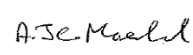
UK EPR	
	Title: UK EPR GDA Submission Master List
	UKEPR-0018-001 Issue 03

Total number of pages: 245	Page No.: I / III
Author: A. FUTTER *Name/Initials*  *Date* *05/12/2012*	Reviewer: F. SAUVAGE *Name/Initials*  *Date* *05/12/2012*
Approved for EDF by: A. MARECHAL *Name/Initials*  *Date* *06/12/2012*	Approved for AREVA by: G. CRAIG *Name/Initials*  *Date* *06/12/2012*

REVISION HISTORY

Issue	Description	Date
00	First issue: Submission Master List 31-03-2011	15/05/2011
01	Step 4 Submission Master List	18/11/2011
02	Final Consolidated Submission Master List	30/11/2012
03	Final Consolidated Submission Master List – Inclusion of “UK EPR - Specification for System Design Manuals Update post GDA” and “UK EPR - Handover document for GDA design changes”	06/12/2012

For information address:

AREVA NP SAS
Tour AREVA
92084 Paris La Défense Cedex
France

EDF
Division Ingénierie Nucléaire
Centre National d'Equipement Nucléaire
165-173, avenue Pierre Brossolette
BP900
92542 Montrouge
France

TABLE OF CONTENTS

1. INTRODUCTION

The Submission Master List is a list which defines the complete UK EPR GDA Submission configuration. The objective of the Submission Master List is to provide a clear record of the perimeter of the UK EPR GDA Submission. The Submission Master List is organised according to the documentation levels reflecting the Submission structure.

The UK EPR GDA Submission is comprised of a Safety, Security and Environmental Report (SSER) and Supporting Documentation. The SSER is comprised of a Pre-Construction Safety Report (PCSR), a Pre-Construction Environmental Report (PCER), a Conceptual Security Arrangements (CSA) document and an SSER Introduction head document. The structure of the UK EPR GDA Submission is:

(i) SSER documents (defined as Level 1 documents)

(ii) Supporting references cited within the SSER (defined as Level 2 documents),

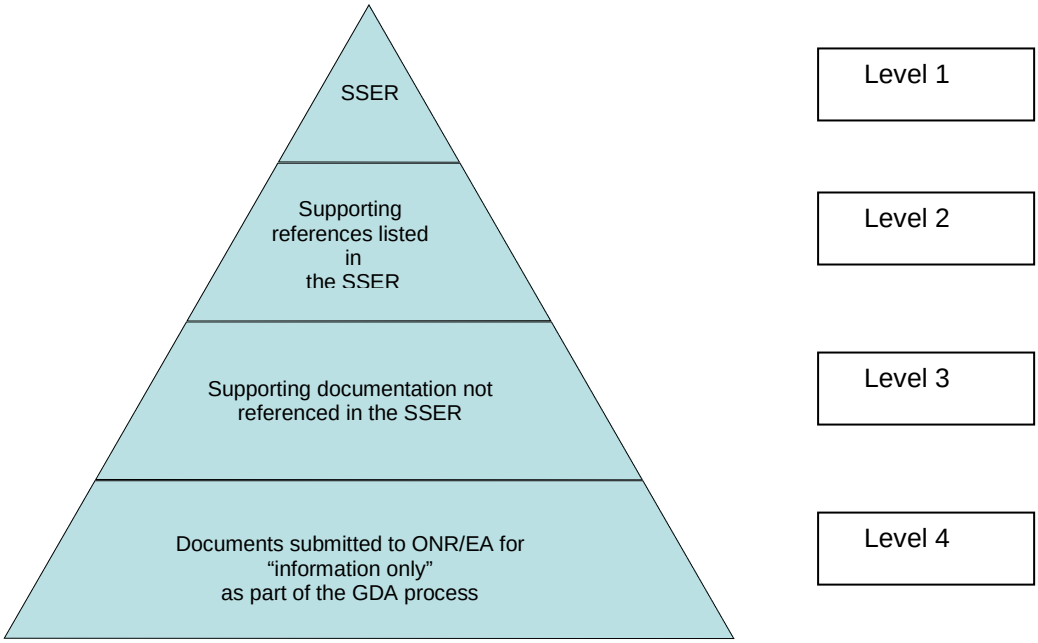
(iii) Submission supporting documents not referenced in the SSER (defined as Level 3 documents).

(iv) Submission supporting documents transmitted to ONR/EA for “information only” to give confidence in the deliverability of the design intent but not to be referenced in the GDA DAC/SoDA (defined as Level 4 documents).

The scope of the GDA Submission and associated Regulator’s assessment is represented by Submission Levels 1 to 4 documents.

The scope of the GDA DAC/SoDA will be represented by the Submission Level 1 to 3 documents at the end of GDA.

Submission Levels 1 to 3 of the Submission Master List are listed in Appendix 1 to 7.



A Security Submission Master List [Ref 1] containing UK Restricted and Confidential information has been produced and submitted to CNS in a separate document

2. REFERENCES

[1] UK EPR GDA Security Submission Master List, November 2011.

APPENDIX 1: UK EPR GDA LEVEL 1 DOCUMENTS: SSER INTRODUCTION HEAD DOCUMENT

Document title	Document number / identifier	Document revision number	Document approval date
Introduction to the Safety, Security and Environmental Report (SSER)	UKEPR-0001-001	06	10-Oct-2012

APPENDIX 2: UK EPR GDA LEVEL 1 DOCUMENTS: PCSR SUBMISSION

Document title	Document number / identifier	Document revision number	Document approval date
PCSR – Chapter 1 – Introduction and General Description			
PCSR – Sub-chapter 1.1 – Introduction	UKEPR-0002-011	04	10-Oct-2012
PCSR – Sub-chapter 1.2 – General description of the unit	UKEPR-0002-012	03	08-Aug-2012
PCSR – Sub-chapter 1.3 – Comparison with reactors of similar design	UKEPR-0002-013	05	10-Oct-2012
PCSR – Sub-chapter 1.4 – Compliance with regulations	UKEPR-0002-016	04	08-Aug-2012
PCSR – Sub-chapter 1.5 – Safety assessment and international practice	UKEPR-0002-017	04	08-Aug-2012
PCSR – Chapter 2 – Generic Site Envelope and Data			
PCSR – Sub-chapter 2.1 – Site Data used in the Safety Analyses	UKEPR-0002-021	04	29-Jun-2012
PCSR – Sub-chapter 2.2 – Site environmental characteristics	UKEPR-0002-022	04	01-Jun-2012
PCSR – Chapter 3 – General Design and Safety Aspects			
PCSR – Sub-chapter 3.1 – General Safety Principles	UKEPR-0002-031	04	08-Nov-2012
PCSR – Sub-chapter 3.2 – Classification of structures, equipment and systems	UKEPR-0002-032	04	16-Nov-2012
PCSR – Sub-chapter 3.3 – Design of Safety Classified Civil Structures	UKEPR-0002-035	05	29-Oct-2012
PCSR – Sub-chapter 3.4 – Mechanical systems and components	UKEPR-0002-036	05	31-Oct-2012
PCSR – Sub-chapter 3.5 – Safety related interfaces	UKEPR-0002-018	05	29-Oct-2012
PCSR – Sub-chapter 3.6 – Qualification of electrical and mechanical equipment for accident conditions	UKEPR-0002-037	04	26-Jun-2012
PCSR – Sub-chapter 3.7 – Conventional Risks of Non-Nuclear Origin	UKEPR-0002-038	02	29-Jun-2009
PCSR – Sub-chapter 3.8 – Codes & standards used in the EPR design	UKEPR-0002-039	05	14-Sep-2012
PCSR – Appendix 3 – Computer Codes used in Chapter 3	UKEPR-0002-310	02	15-Oct-2012

Document title	Document number / identifier	Document revision number	Document approval date
PCSR – Chapter 4 – Reactor and Core Design			
PCSR – Sub-chapter 4.1 – Summary description	UKEPR-0002-041	04	19-Jul-2012
PCSR – Sub-chapter 4.2 – Fuel System Design	UKEPR-0002-042	04	19-Jul-2012
PCSR – Sub-chapter 4.3 – Nuclear Design	UKEPR-0002-043	05	19-Jul-2012
PCSR – Sub-chapter 4.4 – Thermal and hydraulic design	UKEPR-0002-044	04	19-Jul-2012
PCSR – Sub-chapter 4.5 – Functional design of reactivity control	UKEPR-0002-045	03	26-Mar-2011
PCSR – Appendix 4 – Computer codes used in Chapter 4	UKEPR-0002-046	04	19-Jul-2012
PCSR – Chapter 5 – Reactor Coolant System and Associated Systems			
PCSR – Sub-chapter 5.0 – Safety Requirements	UKEPR-0002-050	04	31-Oct-2012
PCSR – Sub-chapter 5.1 – Description of the Reactor Coolant System	UKEPR-0002-051	04	06-Jul-2012
PCSR – Sub-chapter 5.2 – Integrity of the Reactor Coolant Pressure Boundary (RCPB)	UKEPR-0002-052	05	31-Oct-2012
PCSR – Sub-chapter 5.3 – Reactor Vessel	UKEPR-0002-053	06	23-Nov-2012
PCSR – Sub-chapter 5.4 – Components and Systems Sizing	UKEPR-0002-054	05	31-Oct-2012
PCSR – Sub-chapter 5.5 – Reactor Chemistry	UKEPR-0002-055	01	29-Oct-2012
PCSR – Chapter 6 – Containment and Safeguard Systems			
PCSR – Sub-chapter 6.1 – Materials	UKEPR-0002-061	04	26-Mar-2011
PCSR – Sub-chapter 6.2 – Containment Systems	UKEPR-0002-062	04	12-Oct-2012
PCSR – Sub-chapter 6.3 – Safety Injection System (RIS [SIS])	UKEPR-0002-063	04	05-Oct-2012
PCSR – Sub-chapter 6.4 – Habitability of the Control Room	UKEPR-0002-064	04	26-Jun-2012
PCSR – Sub-chapter 6.5 – In-Service Inspection Principles (excluding main primary and secondary systems)	UKEPR-0002-065	02	26-Jun-2009

Document title	Document number / identifier	Document revision number	Document approval date
PCSR – Sub-chapter 6.6 – Emergency Feedwater System (ASG) [EFWS]	UKEPR-0002-066	04	22-Nov-2012
PCSR – Sub-chapter 6.7 – Extra Boration System (RBS) [EBS]	UKEPR-0002-067	04	05-Oct-2012
PCSR – Sub-chapter 6.8 – Main steam relief train system - VDA [MSRT]	UKEPR-0002-068	04	26-Jun-2012
PCSR – Appendix 6 - MER Calculations - BDR Results	UKEPR-0002-069	03	26-Jun-2012
PCSR – Chapter 7 – Instrumentation and Control			
PCSR – Sub-chapter 7.1 – Design principles of the Instrumentation and Control systems	UKEPR-0002-071	04	29-Oct-2012
PCSR – Sub-chapter 7.2 – General architecture of the Instrumentation and Control systems	UKEPR-0002-072	04	05-Nov-2012
PCSR – Sub-chapter 7.3 – Class 1 Instrumentation and Control systems	UKEPR-0002-073	04	05-Nov-2012
PCSR – Sub-chapter 7.4 – Class 2 instrumentation and control systems	UKEPR-0002-074	04	31-Oct-2012
PCSR – Sub-chapter 7.5 – Class 3 Instrumentation and Control Systems	UKEPR-0002-711	01	31-Oct-2012
PCSR – Sub-chapter 7.6 – Instrumentation	UKEPR-0002-075	04	31-Oct-2012
PCSR – Sub-chapter 7.7 – I&C tools, development process and substantiation	UKEPR-0002-076	04	06-Nov-2012
PCSR – Chapter 8 – Electrical Supply and Layout			
PCSR – Sub-chapter 8.1 – External Power Supply	UKEPR-0002-081	04	21-Aug-2012
PCSR – Sub-chapter 8.2 – Power Supply to the Conventional Island and Balance of Plant (BOP)	UKEPR-0002-082	04	21-Aug-2012
PCSR – Sub-chapter 8.3 – Nuclear Island Power Supply	UKEPR-0002-083	06	21-Nov-2012
PCSR – Sub-chapter 8.4 – Specific design principles	UKEPR-0002-084	04	21-Aug-2012
PCSR – Sub-chapter 8.5 – Installation	UKEPR-0002-085	04	21-Aug-2012
PCSR – Sub-Chapter 8.6 – Prevention and Protection against Common Cause Failure	UKEPR-0002-086	01	17-Oct-2012

Document title	Document number / identifier	Document revision number	Document approval date
PCSR – Chapter 9 – Auxiliary Systems			
PCSR – Sub-chapter 9.1 – Fuel Handling and Storage	UKEPR-0002-091	04	21-Nov-2012
PCSR – Sub-chapter 9.2 – Water Systems	UKEPR-0002-092	04	23-Nov-2012
PCSR – Sub-chapter 9.3 – Primary System Auxiliaries	UKEPR-0002-093	04	15-Oct-2012
PCSR – Sub-chapter 9.4 – Heating, Ventilation and Air-Conditioning Systems	UKEPR-0002-094	03	23-Nov-2012
PCSR – Sub-chapter 9.5 – Other supporting systems	UKEPR-0002-095	04	21-Nov-2012
PCSR – Chapter 10 – Main Steam and Feedwater Lines			
PCSR – Sub-chapter 10.1 – General Description	UKEPR-0002-101	02	24-Jun-2009
PCSR – Sub-chapter 10.2 – Turbogenerator Set	UKEPR-0002-102	01	26-Apr-2008
PCSR – Sub-chapter 10.3 – Main steam system (safety classified part)	UKEPR-0002-103	05	31-Oct-2012
PCSR – Sub-chapter 10.4 – Other features of steam and power conversion systems	UKEPR-0002-104	04	26-Jun-2012
PCSR – Sub-chapter 10.5 – Integrity of the main steam lines inside and outside the containment	UKEPR-0002-105	05	31-Oct-2012
PCSR – Sub-chapter 10.6 – Main feedwater system	UKEPR-0002-106	04	27-Jul-2012
PCSR – Chapter 11 – Discharges and Waste - Chemical and Radiological			
PCSR – Sub-chapter 11.0 – Safety Requirements	UKEPR-0002-110	04	21-May-2012
PCSR – Sub-chapter 11.1 – Sources of radioactive materials	UKEPR-0002-111	05	21-Aug-2012
PCSR – Sub-chapter 11.2 – Details of the effluent management process	UKEPR-0002-112	05	28-Aug-2012
PCSR – Sub-chapter 11.3 – Outputs for the operating installation	UKEPR-0002-113	05	23-Aug-2012
PCSR – Sub-chapter 11.4 – Effluent and waste treatment systems design architecture	UKEPR-0002-114	05	22-Oct-2012
PCSR – Sub-chapter 11.5 – Interim storage facilities and disposability for UK EPR	UKEPR-0002-115	03	20-Jun-2012

Document title	Document number / identifier	Document revision number	Document approval date
PCSR – Chapter 12 – Radiation Protection			
PCSR – Sub-chapter 12.0 – Radiation protection requirements	UKEPR-0002-120	04	27-Sep-2012
PCSR – Sub-chapter 12.1 – Radiation protection approach	UKEPR-0002-121	04	27-Sep-2012
PCSR – Sub-chapter 12.2 – Definition of radioactive sources in the primary circuit	UKEPR-0002-122	04	27-Sep-2012
PCSR – Sub-chapter 12.3 – Radiation protection measures	UKEPR-0002-123	04	29-Jun-2012
PCSR – Sub-chapter 12.4 – Dose uptake optimisation	UKEPR-0002-124	04	27-Sep-2012
PCSR – Sub-chapter 12.5 – Post accident accessibility	UKEPR-0002-125	04	27-Sep-2012
PCSR – Chapter 13 – Hazards Protection			
PCSR – Sub-chapter 13.1 – External Hazards Protection	UKEPR-0002-131	06	16-Nov-2012
PCSR – Sub-chapter 13.2 – Internal Hazards Protection	UKEPR-0002-132	05	31-Oct-2012
PCSR – Chapter 14 – Design Basis Analysis			
PCSR – Sub-chapter 14.0 – Assumptions and Requirements for the PCC Accident Analyses	UKEPR-0002-140	05	16-Nov-2012
PCSR – Sub-chapter 14.1 – Plant Characteristics taken into account in the Accident Analyses	UKEPR-0002-141	04	27-Sep-2012
PCSR – Sub-chapter 14.2 – Analysis of the Passive Single Failure	UKEPR-0002-142	04	23-Nov-2012
PCSR – Sub-chapter 14.3 – Analyses of PCC-2 events	UKEPR-0002-143	07	26-Nov-2012
PCSR – Sub-chapter 14.4 – Analyses of the PCC-3 events	UKEPR-0002-144	08	26-Nov-2012
PCSR – Sub-chapter 14.5 – Analyses of the PCC-4 events	UKEPR-0002-145	08	16-Nov-2012
PCSR – Sub-chapter 14.6 – Radiological consequences of design basis accidents	UKEPR-0002-146	06	14-Nov-2012
PCSR – Sub-chapter 14.7 – Fault and Protection Schedule	UKEPR-0002-149	03	26-Nov-2012
PCSR – Appendix 14A – Computer codes used in Chapter 14	UKEPR-0002-147	04	27-Jul-2012
PCSR – Appendix 14B – 4900MW Safety Analyses used in Chapter 14	UKEPR-0002-148	05	26-Mar-2011
PCSR – Appendix 14C – Analysis of single failure for main steam line break	UKEPR-0002-001	01	27-Jul-2012

Document title	Document number / identifier	Document revision number	Document approval date
PCSR – Chapter 15 – Probabilistic Safety Analysis			
PCSR – Sub-chapter 15.0 – Safety requirements and PSA objectives	UKEPR-0002-150	04	16-May-2012
PCSR – Sub-chapter 15.1 – Level 1 PSA	UKEPR-0002-151	05	09-Nov-2012
PCSR – Sub-chapter 15.2 – PSA for internal and external hazards	UKEPR-0002-152	05	06-Nov-2012
PCSR – Sub-chapter 15.3 – PSA of accidents in the spent fuel pool	UKEPR-0002-153	06	16-Nov-2012
PCSR – Sub-chapter 15.4 – Level 2 PSA	UKEPR-0002-154	06	21-Nov-2012
PCSR – Sub-chapter 15.5 – Level 3 PSA: Assessment of off-site risk due to postulated accidents	UKEPR-0002-155	05	09-Nov-2012
PCSR – Sub-chapter 15.6 – Seismic Margin Assessment	UKEPR-0002-156	06	17-May-2012
PCSR – Sub-chapter 15.7 – PSA Discussion and Conclusions	UKEPR-0002-157	06	21-Nov-2012
PCSR – Chapter 16 – Risk Reduction and Severe Accident Analyses			
PCSR – Sub-chapter 16.1 – Risk reduction analysis (RRC-A)	UKEPR-0002-161	07	12-Nov-2012
PCSR – Sub-chapter 16.2 – Severe accident analysis (RRC-B)	UKEPR-0002-162	05	12-Nov-2012
PCSR – Subchapter 16.3 – Practically eliminated situations	UKEPR-0002-163	04	22-Nov-2012
PCSR – Sub-chapter 16.4 – Specific Studies	UKEPR-0002-166	04	28-Nov-2012
PCSR – Sub-chapter 16.5 – Adequacy of the UK EPR design regarding functional diversity	UKEPR-0002-167	01	26-Nov-2012
PCSR – Sub-chapter 16.6 – Analysis of Extreme Beyond Design Basis Events Carried Out in Response to Fukushima	UKEPR-0002-168	00	22-Nov-2012
PCSR – Appendix 16 A – Computer Codes used in Chapter 16	UKEPR-0002-164	04	16-May-2012
PCSR – Appendix 16 B – 4900 MW safety analyses used in Chapter 16	UKEPR-0002-165	06	16-May-2012

Document title	Document number / identifier	Document revision number	Document approval date
PCSR – Chapter 17 – Compliance with ALARP Principle			
PCSR – Sub-chapter 17.1 – Explanation of ALARP Requirement	UKEPR-0002-171	04	24-May-2012
PCSR – Sub-chapter 17.2 – Demonstration of Relevant Good Practice in EPR Design	UKEPR-0002-172	04	24-May-2012
PCSR – Sub-chapter 17.3 – EPR Design Optioneering	UKEPR-0002-173	04	24-May-2012
PCSR – Sub-chapter 17.4 – Review of PSA results: Comparison with Numerical Risk Targets	UKEPR-0002-174	04	30-Aug-2012
PCSR – Sub-chapter 17.5 – Review of possible design modifications to confirm design meets ALARP principle	UKEPR-0002-175	04	30-Aug-2012
PCSR – Sub-chapter 17.6 – Conclusions of EPR ALARP assessment	UKEPR-0002-176	04	24-May-2012
PCSR – Chapter 18 – Human Factors and Operational Aspects			
PCSR – Sub-chapter 18.1 – Human Factors	UKEPR-0002-181	06	15-Nov-2012
PCSR – Sub-chapter 18.2 – Normal Operation	UKEPR-0002-182	06	15-Nov-2012
PCSR – Sub-chapter 18.3 – Abnormal Operation	UKEPR-0002-183	03	15-Nov-2012
PCSR – Chapter 19 – Commissioning			
PCSR – Sub-chapter 19.0 – Commissioning Safety Requirements	UKEPR-0002-190	04	16-May-2012
PCSR – Sub-chapter 19.1 – Plant Commissioning Programme	UKEPR-0002-191	01	27-Mar-2011
PCSR – Chapter 20 – Design Principles Related to Decommissioning			
PCSR – Sub-chapter 20.1 – General Decommissioning Principles – Regulations	UKEPR-0002-201	02	08-Jun-2012
PCSR – Sub-chapter 20.2 – Decommissioning - Implementation for the EPR	UKEPR-0002-202	02	08-Jun-2012
PCSR – Chapter 21 – Quality and Project Management			
PCSR – Sub-chapter 21.1 – Project Organisation	UKEPR-0002-211	02	17-May-2012
PCSR – Sub-chapter 21.2 – Management System	UKEPR-0002-212	02	17-May-2012

APPENDIX 3: UK EPR GDA LEVEL 1 DOCUMENTS: PCER SUBMISSION

Document title	Document number / identifier	Document revision number	Document approval date
PCER – Chapter 1 – Introduction and General Description			
PCER – Sub-chapter 1.1 – Introduction	UKEPR-0003-011	06	18-Oct-2012
PCER – Sub-chapter 1.2 – General description of the unit	UKEPR-0003-012	03	08-Aug-2012
PCER – Sub-chapter 1.3 – Comparison with reactors of similar design	UKEPR-0003-013	05	10-Oct-2012
PCER – Sub-chapter 1.4 – Compliance with regulations	UKEPR-0003-014	03	08-Aug-2012
PCER – Sub-chapter 1.5 – Safety assessment and international practice	UKEPR-0003-015	03	08-Aug-2012
PCER – Chapter 2 – Quality and Project Management	UKEPR-0003-020	04	17-May-2012
PCER – Chapter 3 – Aspects having a bearing on the environment during operation phase	UKEPR-0003-030	05	22-Aug-2012
PCER – Chapter 4 – Aspects having a bearing on the environment during construction phase	UKEPR-0003-040	02	17-Jul-2012
PCER – Chapter 5 – Design principles related to decommissioning	UKEPR-0003-050	05	08-Jun-2012
PCER – Chapter 6 – Discharges and Waste - Chemical and Radiological			
PCER – Sub-chapter 6.0 – Safety requirements	UKEPR-0003-060	03	21-May-2012
PCER – Sub-chapter 6.1 – Sources of radioactive materials	UKEPR-0003-061	05	21-Aug-2012
PCER – Sub-chapter 6.2 – Details of the effluent management process	UKEPR-0003-062	05	28-Aug-2012
PCER – Sub-chapter 6.3 – Outputs for the operating installation	UKEPR-0003-063	05	23-Aug-2012
PCER – Sub-chapter 6.4 - Effluent and waste treatment systems design architecture	UKEPR-0003-064	05	22-Oct-2012
PCER – Sub-chapter 6.5 – Interim storage facilities and disposability for UK EPR	UKEPR-0003-065	04	20-Jun-2012
PCER – Chapter 7 – Measures for monitoring discharges	UKEPR-0003-070	03	04-Aug-2012

Document title	Document number / identifier	Document revision number	Document approval date
PCER – Chapter 8 – Best Available Techniques	UKEPR-0003-080	04	28-Aug-2012
PCER – Chapter 9 – Principles and methods used for environmental approach at the design stage	UKEPR-0003-090	04	08-Jun-2012
PCER – Chapter 10 – Site environmental characteristics	UKEPR-0003-100	05	01-Jun-2012
PCER – Chapter 11 – Radiological impact assessment	UKEPR-0003-110	04	19-Jun-2012
PCER – Chapter 12 – Non radiological impact assessment	UKEPR-0003-120	04	22-Oct-2012

APPENDIX 4: UK EPR GDA LEVEL 2 DOCUMENTS: SSER INTRODUCTION HEAD DOCUMENT REFERENCES

Chapter	Sub-chapter	Reference	Issue	Title
SSER				
SSER	INTRO	REF0001	2	New Nuclear Power Stations, Generic Design Assessment - A guide to the regulatory processes. Version 2. UK Health and Safety Executive and Environment Agency. August 2008. (E)
SSER	INTRO	UKEPRI002		UK EPR GDA Project - Reference Design Configuration. UKEPR-I-002 (E)
SSER	INTRO	REF0002	3	Nuclear Power Station Generic Design Assessment – Guidance to Requesting Parties. Version 3. UK Health and Safety Executive. August 2008 (E)
SSER	INTRO	REF0003	1	Process and Information Document for Generic Assessment of Candidate Nuclear Power Designs. Version 1. UK Environment Agency. January 2007 (E)
SSER	INTRO	REF0004	2	Guidance Document for Generic Design Assessment Activities – Office for Civil Nuclear Safety. Version 2. January 2007 (E)

APPENDIX 5: UK EPR GDA LEVEL 2 DOCUMENTS: PCSR REFERENCES

Chapter	Sub-chapter	Reference	Issue	Title
CHAPTER 1				
SUB-CHAPTER 1.1 – REFERENCES				
1	1.1	UKEPRI002		UK EPR GDA Project - Reference Design Configuration. UKEPR-I-002. EDF/AREVA. (E)
1	1.1	LetterDGSNRSD20729		Direction Générale de la Sûreté Nucléaire et de la Radioprotection, [General Directorate of Nuclear Safety and Radiation, DGSNR]. Letter DGSNR/SD2/N°0729 / 2004
SUB-CHAPTER 1.4 – REFERENCES				
1	1.4	REF0005		Office for Nuclear Regulation Framework Document, as amended on 9 August and 23 November 2011. http://www.hse.gov.uk/nuclear/onr-framework.pdf
1	1.4	ISBN9780102156836		Health and Safety at Work etc Act, 1974. ISBN 978-010215683-6. The Stationery Office Ltd. (E)
1	1.4	ISBN0108502163		The Nuclear Installations Act 1965 (as amended). ISBN 0-10-850216-3. The Stationery Office Ltd. (E)
1	1.4	ISBN9780105425953		Environment Act 1995. ISBN 978-010542595-3. The Stationery Office Ltd. (E)
1	1.4	ISBN0105412937		The Radioactive Substances Act 1993. ISBN 0-10-541293-7. The Stationery Office Ltd (E)
1	1.4	9629EURATOM	Vol39	Council Directive 96/29/EURATOM of 13 May 1996 laying down basic safety standards for the protection of the health of workers and the general public against the dangers arising from ionizing radiation. Official Journal of the European Communities, L159, Vol. 39. June 1996. (E)
1	1.4	ICRP60		1990 Recommendations of the International Commission on Radiological Protection. ICRP Publication 60. Ann. ICRP 21 (1-3). 1991. (E)
1	1.4	ISBN0110856147		The Ionising Radiations Regulations 1999. Statutory Instrument 1999 No. 3232. ISBN 0 11-085614-7. The Stationery Office Ltd (E)
1	1.4	ISBN0717617467		Approved Code of Practice: Work with ionising radiation. Ionising Radiations Regulations 1999. Approved Code of Practice and guidance L121 HSE Books 2000 ISBN 0-71-761746-7. (E)

1	1.4	ISBN0110299086		The Radiation (Emergency Preparedness and Public Information) Regulations 2001 (REPPIR). Statutory Instrument 2001 No. 2975. ISBN 0-11-029908-6.HM Stationery Office. The Stationery Office Ltd (E)
1	1.4	REF0006		Radioactive Substances (Basic Safety Standards) (England and Wales) Direction 2000. Defra, UK. (E)
1	1.4	ISBN9780110996219		The Pollution Prevention and Control Regulations 2000 (PPC). ISBN 978-011099621-9. The Stationery Office Ltd. (E)
1	1.4	ISBN9780110440569		Waste Management Licensing Regulations 1994. ISBN 978-011044056-9. The Stationery Office Ltd. (E)
1	1.4	ISBN9780110726854		Hazardous Waste Regulations 2005. ISBN 978-011072685-4. The Stationery Office Ltd. (E)
1	1.4	ISBN9780105408901		The Town and Country Planning Act (1990) (TCP Act). ISBN 978-010540890-1. The Stationery Office Ltd. (E)
1	1.4	ISBN0110804740		Town and Country Planning EIA (England and Wales) Regulations 1999. ISBN 0-11-080474-0. The Stationery Office Ltd. (E)
1	1.4	ISBN0110186907		Town and Country Planning EIA (England and Wales) (Amended) Regulations 2000. ISBN 0-11-018690-7. The Stationery Office Ltd. (E)
1	1.4	ISBN0110754719		Town and Country Planning EIA (England and Wales) (Amended) Regulations 2006. ISBN 0-11-075471-9. The Stationery Office Ltd. (E)
1	1.4	ISBN0105429890		The Electricity Act 1989 ISBN 978-010542989-0. The Stationery Office Ltd. (E)
1	1.4	ISBN0110996752		The Electricity Works (Environmental Impact Assessment) (England and Wales) Regulations 2000 (SI 1927). ISBN 978-011099675-2. The Stationery Office Ltd. (E)
1	1.4	ISBN0110853956		The Nuclear Reactors (Environmental Impact Assessment for Decommissioning) Regulations 1999 (EIADR) (SI 2892). ISBN 978-011085395-6. The Stationery Office Ltd. (E)
1	1.4	ISBN0110858360		The Pressure Systems Safety Regulations 2000 (PSSR). Statutory Instruments 2000 No. 1281, ISBN 0-11-085836-0. The Stationery Office Ltd, (E)
1	1.4	ISBN0110827902		The Pressure Equipment Regulations 1999 (PER). Statutory Instruments 1999/2001, ISBN 978-011082790-2. The Stationery Office Ltd, (E)
1	1.4	REF0007		The Nuclear Installations (Dangerous Occurrences) Regulations 1965. Statutory Instruments 1965 No.1824. The Stationery Office Ltd. (E)
1	1.4	ISBN9780110966359		The Electricity at Work Regulations 1989. ISBN 9780110966359. The Stationery Office Ltd. (E)
1	1.4	ISBN0110429206		The Electricity Safety, Quality and Continuity Regulations 2002. ISBN 0 11 042920 6. The Stationery Office Ltd. (E)
1	1.4	ISBN0110729455		Regulatory Reform (Fire Safety) Order 2005. ISBN 0-11-072945-5. The Stationery Office Ltd. (E)
1	1.4	ISBN0110429575		Dangerous Substances and Explosive Atmospheres Regulations 2002 (DSEAR). ISBN 0-11-042957-5. The Stationery Office Ltd. (E)
1	1.4	ISBN9780110539997		The Equipment and Protective Systems (EPS) Intended for Use in Potentially Explosive Atmospheres Regulations 1996. ISBN 978-011053999-7. The Stationery Office Ltd. (E)

1	1.4	ISBN9780110821924		The Control of Major Accident Hazards (Amendment) Regulations 1999 (COMAH). ISBN 978-011082192-4. The Stationery Office Ltd. (E)
1	1.4	ISBN0110727665		Control of Major Accident Hazards (Amendment) Regulations 2005 (COMAH). ISBN 0-110-72766-5. The Stationery Office Ltd. (E)
1	1.4	ISBN9780105424017		The Anti-Terrorism, Crime and Security Act 2001 4. ISBN 978-010542401-7. The Stationery Office Ltd. (E)
1	1.4	ISBN0110453972		The Nuclear Industries Security Regulations 2003 (NISR). ISBN 0-11-045397-2. The Stationery Office Ltd. (E)
1	1.4	ISBN9780105457916		Water Resources Act 1991. ISBN 978-010545791-6. The Stationery Office Ltd. (E)
1	1.4	ISBN0110795997		Provision and Use of Work equipment Regulations 1998 (PUWER). ISBN 0-11-079599-7.The Stationery Office (E)
1	1.4	ISBN0110795989		Lifting Operations and Lifting Equipment Regulations 1998. ISBN 0-11-079598-9. The Stationery Office Ltd. (E)
1	1.4	ISBN0110258320		The Personal Protective Equipment at Work Regulations 1992.ISBN 0-11-025832-0. The Stationery Office Lt (E)
1	1.4	ISBN0110856252		Management of Health and Safety at Work Regulations 1999. ISBN 0-11-085625-2. The Stationery Office Ltd. (E)
1	1.4	ISBN0110757896		The Construction (Design And Management) Regulations 2007. ISBN 978-011075789-6. The Stationery Office Ltd. (E)
1	1.4	ICRPPub63		ICRP. Principles for intervention for protection of the public in a radiological emergency. ICRP Publication 63, Annals of the ICRP 22 (4). 1993 (E)
1	1.4	ISBN9201042957		IAEA. International Basic Safety Standards for protection from ionising radiation and security with respect to radiation sources. Safety series No 115. ISBN 92-0-104295-7. IAEA Vienna, 1996. (E)
1	1.4	IAEA50CSGQ		IAEA. Quality Assurance for Safety in Nuclear Power Plants and other Nuclear Installations, Code and Safety Guides Q1–Q14. Safety Series No. 50-C/SG-Q, IAEA, Vienna (1996). (E)
1	1.4	IAEANSR1		Safety of Nuclear Power Plants: Design (Requirements for Design). ISSN 1020-525X IAEA Safety Standards Series N° NS-R-1. IAEA. 2000. (E)
1	1.4	REF0008		Technical Guidelines for the design and construction of the next generation of nuclear pressurised water plant units adopted during plenary meetings of the GPR and German experts on the 19 and 26 October 2000. (E)
1	1.4	RFSGB200101		Basic Safety Rule - Fundamental safety rule n°2001-01 concerning basic nuclear installations. RFS-GB-2001-01. ASN. (E)
1	1.4	RFS12d		Basic Safety Rule – Rule 1.2.d. Protection against risks associated to the industrial environment and the roads. May 1982. (E)
1	1.4	RFS12e		Basic Safety Rule – Rule 1.2.e. Protection against external flooding. April 1984. (E)
1	1.4	ASNGUIDE201		ASN Guide 2/01. Considering the risk of an earthquake when designing civil engineering structures of nuclear installations, excluding longterm storage of radioactive waste. May 2006. (E)
SUB-CHAPTER 1.5 – REFERENCES				

1	1.5	2007AV0016		Statement concerning the draft decree authorising the creation of the basic Flamanville 3 nuclear installation, including an EPR type nuclear reactor, on the Flamanville (Channel) site. Statement No. 2007-AV-0016. (French) Nuclear Safety Authority, 16 February 2007.(Avis relatif au projet de décret autorisant la création de l'installation nucléaire de base dénommée Flamanville 3, comportant un réacteur nucléaire de type EPR, sur le site de Flamanville (Manche). Avis n° 2007-AV- 0016. Autorité de Sûreté Nucléaire. 16 février 2007)
1	1.5	REF0008		"Technical Guidelines for the design and construction of the next generation of nuclear pressurized water plant units" adopted during plenary meetings of the GPR and German experts on the 19 and 26 October 2000. (French) Nuclear Safety Authority, 20 October 2000. (E)
1	1.5	2008DC0114		Decision with regard to Électricité de France - Société Anonyme (EDF-SA) setting the requirements concerning the nuclear power reactor site at Flamanville (Manche department) for the design and construction of the "Flamanville 3" reactor (BNI 167) and for operation of the "Flamanville 1" (BNI 108) and "Flamanville 2" (BNI 109) reactors. Decision No. 2008-DC-0114. (French) Nuclear Safety Authority, 26 September 2008. (Décision fixant à Électricité de France – Société Anonyme (EDF-SA) les prescriptions relatives au site électronucléaire de Flamanville (Manche) pour la conception et la construction du réacteur « Flamanville 3 » (INB n°167) et pour l'exploitation des réacteurs « Flamanville 1 » (INB n°108) et « Flamanville 2 » (INB n°109). Décision n°2008-DC-0114. ASN (Autorité de sûreté nucléaire). 26 septembre 2008)
1	1.5	ECEP102828		Robert Pays. EPR FA3 – Transmission of the first working draft version of the start-up authorisation file. Letter ECEP102828. EDF Direction Production Ingénierie. 29 October 2010. (Robert Pays. EPR FA3 – Transmission de la première version de travail du dossier de demande de mise en service. Lettre ECEP102828. EDF Direction Production Ingénierie. 29 Octobre 2010)
1	1.5	REF0009		WENRA Reactor Safety Reference Levels. Western European Nuclear Regulators' Association, Reactor Harmonization Working Group, January 2007. (E)
1	1.5	EURVOL1	B	European Utility Requirements for LWR nuclear power plants) Volume 1, Chapter C3, Revision B. EUR, Villeurbanne, France. (E)
1	1.5	EURVOL3EPR	B	European Utility Requirements for LWR nuclear power plants) Volume 3 – Standard EPR Subset. Revision B. EUR, Villeurbanne, France. July 2009. (E)
1	1.5	EURVOL1	C	European Utility Requirements for LWR nuclear power plants) Volume 1, Revision C. EUR, Villeurbanne, France. April 2001. (E)
CHAPTER 2				
SUB-CHAPTER 2.1 – REFERENCES				
2	2.1	EUR2001	C	European Utility Requirements for LWR nuclear power plants. Volume 2 - Generic nuclear island requirements. Revision C. EUR document. April 2001. (E)

2	2.1	TDNSREP0243B	1	Nuclear Electric. Safety Review Guidebook for Gas Cooled Reactors, Nuclear Electric Report TD/NS/REP/0243/B, Issue 1, September 1993. (E)
2	2.1	REF0010		Civil Aviation Authority. Structure and operation of UK airspace. Environmental Information Sheet No.3. UK Civil Aviation Authority, Directorate of Airspace Policy: 2006. (E)
2	2.1	CRR1501997		Health and Safety Executive. The calculation of aircraft crash risk in the UK, prepared by AEA Technology plc for the HSE, Contract Research Report 150/1997. 1997 (E)
2	2.1	REF0011		UK Met Office website - http://www.metoffice.gov.uk/
2	2.1	SAP2006	1	UK Health and Safety Executive (HSE). Safety Assessment Principles for Nuclear Facilities. 2006 Edition Revision 1. January 2008. (E)
2	2.1	REF0012		TORRO Organisation website - HTTP://WWW.TORRO.ORG.UK/TORRO/ (E)
SUB-CHAPTER 2.2 – REFERENCES				
2	2.2	REF0003	1	Process and Information Document for Generic Assessment of Candidate Nuclear Power Plant Designs. Version 1. The Environment Agency. January 2007. (E)
2	2.2	WFD2009		The River Basin Districts Surface Water and Groundwater Classification (Water Framework Directive) (England and Wales) Direction 2009. The Stationery Office Ltd. (E)
2	2.2	WFD2010		The River Basin Districts Typology, Standards and Groundwater threshold values (Water Framework Directive) (England and Wales) Direction 2010. The Stationery Office Ltd. (E)
2	2.2	REF0013		Wildlife and Countryside Act 1981 (Chapter 69). The UK Statute Law Database. Office of Public Sector Information, London (E)
2	2.2	ISBN0110457161		The Conservation (Natural Habitats, &c.) Regulations 1994. The UK Statute Law Database. Office of Public Sector Information, London. ISBN 0 11045 716 1. (E)
2	2.2	ISBN0859515133		K R Smith and A L Jones. Generalised Habit Data for Radiological Assessments. NRPB-W41. National Radiological Protection Board. ISBN 0 85951 513 3. May 2003 (E)
2	2.2	ISBN0080411545		ICRP Publication 66: Human Respiratory Tract Model for Radiological Protection, 66. Annals of the ICRP Volume 24/1-3. International Commission on Radiological Protection. ISBN 978-0-08-041154-5. Elsevier. 1995. (E)
CHAPTER 3				
SUB-CHAPTER 3.1 – REFERENCES				
3	3.1	IAEANSR1		Safety of Nuclear Power Plants: Design (Requirements for Design). ISSN 1020-525X IAEA Safety Standards Series N° NS-R-1. IAEA. 2000. (E)

3	3.1	DSIN132193		Joint declaration of the French and German Authorities on a common safety approach for future pressurized water reactors. DSIN letter no. 1321/93. 2 September 1993
3	3.1	CSFRF0893		Conceptual Safety Features Review File (CSFRF). EPR-CSFRF - 08/93; report jointly prepared by EdF, German Utilities, NPI, Siemens, Framatome. (E)
3	3.1	BDR1997		Basic Design Report. Report Jointly prepared by Electricite De France, German Utilities, Framatome, Siemens AG, Nuclear Power International. Issue October 1997. (E)
3	3.1	BDR1999		Basic Design Report. Report Jointly prepared by Electricite De France, German Utilities, Framatome, Siemens AG, Nuclear Power International. Issue February 1999. (E)
3	3.1	DGSNRSD20729		Option de sûreté du projet de réacteur EPR [Safety options for the EPR reactor project]. Letter DGSNR/SD2/n°0729/2004. 28 September 2004.
3	3.1	NEPSFDC400	B	T Bruyères, F Pairot. UK EPR Generic Design Assessment Fault and Protection Schedule. NEPS-F DC 400 Revision B. AREVA. February 2011. (E)
3	3.1	ENSNEA080058	A	EPR - External hazards – inventory of combined events with internal faults and/or other (internal and external) hazards taken into account in design. ENSNEA080058 Revision A. EDF. November 2008. (E) ENSNEA080058 Revision A is the English translation of ENSNEA050062 Revision A.
3	3.1	IPSNDEC1999		Rapport sur l'inondation du site du Blayais survenu le 27 décembre 1999, rapport IPSN, [Report on the flooding at Blayais site which occurred on 27 December 1999, IPSN report].January 2000
3	3.1	SAP2006	1	Safety Assessment Principles for Nuclear Facilities. 2006 Edition Revision 1. UK Health and Safety Executive (HSE). January 2008. (E)
3	3.1	ISBN0110856147		The Ionising Radiations Regulations 1999, Statutory Instrument 1999 No. 3232, HM Stationery Office, ISBN 0 11 085614 7. (E)
3	3.1	REF0006		Radioactive Substances (Basic Safety Standards) (England and Wales) Direction 2000 (E)
3	3.1	REF0008		Technical Guidelines for the design and construction of the next generation of nuclear pressurised water plant units, adopted during plenary meetings of the GPR and German experts on the 19 and 26 October 2000. (E)
SUB-CHAPTER 3.2 – REFERENCES				
3	3.2	NEPSFDC557	D	Classification of Structures, Systems and Components. NEPS-F DC 557 Revision D. AREVA. October 2012. (E)
3	3.2	ISBN9201007078		IAEA Safety Glossary. Terminology Used in Nuclear Safety and Radiation Protection. ISBN 92-0-100707-8. 2007 Edition. (E)
3	3.2	DS367	6.3	IAEA Safety Standards – Safety Classification of Structures, Systems and Components in Nuclear Power Plants. Draft Safety Guide DS 367. Draft 6.3 June 2012. (E)
3	3.2	IAEANSR1		IAEA Safety Standards - Safety of Nuclear Power Plants: Design. NS-R-1. 2000. (E)

3	3.2	BSIEC61226		Nuclear power plants – Instrumentation and control important to safety – Classification of instrumentation and control functions. BS IEC 61226:2009. (E)
3	3.2	PEPSFDC90	C	Safety principles applied to the UK EPR I&C architecture in terms of the requirements for diversity and independence. PEPS-F DC 90 Revision C. AREVA. August 2012. (E)
3	3.2	SAP2006	1	UK Health and Safety Executive (HSE). Safety Assessment Principles for Nuclear Facilities. 2006 Edition Revision 1. January 2008. (E)
3	3.2	IEC61513		Nuclear power plants – Instrumentation and control for systems important to safety – General requirements for systems. IEC 61513. Edition 2001. (E)
3	3.2	TAST005	4	UK Health and Safety Executive (HSE). Technical Assessment Guide, ND Guidance on the Demonstration of ALARP (As Low As is Reasonably Practicable). T/AST/005 Issue 4. January 2009. (E)
3	3.2	ENSNDR100088	A	F. Romanet. UK EPR ALARP methodology to support the design modification process. ENSNDR100088 Revision A. EDF/SEPTEN. July 2010. (E)
3	3.2	ENSECC110102	B	Justification of smart devices for nuclear safety applications. ENSECC110102 Revision B. EDF. May 2012. (E)
3	3.2	ECECC111134	C	UK EPR Guideline for Application of Production Excellence and Independent Confidence Building. ECECC111134 Revision C. EDF. July 2012. (E)
3	3.2	ENSN110130	A	UK EPR – Safety Class 2 Structures – Definition of C2 Safety Requirements. ENSN110130 Issue A. EDF. January 2012. (E)
3	3.2	ENGSGC110254	B	Civil Works Dedicated Rules for Buildings Classified C2 'Main Structures. ENGSGC110254 Issue B. EDF. June 2012. (E)
3	3.2	ENGSGC110157	B	UK EPR – GDA – Good Practice - Seismic Detailing Rules for Safety Classified Reinforced Concrete and Steel Structures. ENGSGC110157 Revision B. EDF. April 2012. (E)
3	3.2	ECEIG112228	A	UK EPR – GDA – Identification of C1 'Other Structures' (Removal Parts). ECEIG112228 Revision A. EDF. February 2012. (E)
SUB-CHAPTER 3.3 – REFERENCES				
3	3.3	ECEIG111110	C	EPR Nuclear Island Civil Engineering Design Process. ECEIG111110 Revision C. EDF. October 2012. (E) *Supersedes Revision B June 2012 and Revision A August 2011.*
3	3.3	ENGSGC110254	B	Civil Works Dedicated Rules for Buildings Classified C2 'Main Structures. ENGSGC110254 Issue B. EDF. June 2012. (E)
3	3.3	ENSN110130	A	UK EPR – Safety Class 2 Structures – Definition of C2 Safety Requirements. ENSN110130 Issue A. EDF. January 2012. (E)

3	3.3	ECEIG021405	H1	EPR - General Hypotheses Note for Civil Engineering Design of Nuclear Island Buildings. ECEIG021405 Revision H1 (TR 07/312). EDF. October 2008. (E) ECEIG021405 Revision H1 is the English translation of ECEIG021405 Revision H.
3	3.3	ENGSGC100140	C	Common Foundation Raft and Seismic Analysis - GDA Scope. ENGSGC100140 Revision C. EDF. December 2011. (E) *Supersedes Revison B January 2011 and Revision A June 2010.*
3	3.3			Reference for design report and drawing package for aircraft shell; *this reference contains Security Sensitive material*
3	3.3	ECEIG102182	A	EPR Nuclear Island Detailed Design Report. ECEIG102182 Revision A. EDF. December 2010. (E)
3	3.3	ECEIG102044	B	EPR Inner Containment Wall detailed design report ("route map"). ECEIG102044 Revision B. EDF. February 2011. (E)
3	3.3	KWUNAT1996017	B	EPR Technical Report TR 96/25. Protection against internal hazards. KWU NA-T/1996/017 B. Siemens. April 1996. (E) Alternative reference number: ENSN960623 (TR 96-25). EDF
3	3.3	KWUNAT1996033	C	Protection against External Hazards. KWU NA-T/1996/033 Revision C (TR96-49). AREVA. December 1996. (E)
3	3.3	ENSN030450	A	C Clement. Safety approach adopted in the design of the EPR containment with a steel liner. ENSN030450 Revision A. EDF. October 2003. (E)
3	3.3	ENGSGC050076	B	ETC-C (EPR Technical Code for Civil works). ENGSGC050076 Revision B. EDF. April 2006. (E)
3	3.3	ENGSGC090193	A	Design criteria for a double-walled containment with a steel liner. ENGSGC090193 Revision A. EDF. September 2009. (E) ENGSGC090193 Revision A is the English translation of ENGSGC030167 Revision A.
3	3.3	UKEPRCMF018		Design Change Management Form - Modification of liner floor design for future EPRs. UKEPR-CMF- 018. (E).
3	3.3	ENGSGC100010	C	Safety Justification of the Prestressing System of the UK EPR Inner Containment. ENGSGC100010 Revision C. EDF. January 2011 (E)
3	3.3	ENGSGC100267	B	Principles of an optimum instrumentation system (DAO) for the UK EPR inner containment. ENGSGC100267 Revision B. EDF. January 2011. (E)
3	3.3	PEPSPF11051		Target Reliabilities for UK EPR Structures built with ETC-C. PEPSPF/11.051. AREVA. January 2011. (E).
3	3.3	ENGSDS100093	B	Reliability of the EPR Inner containment to Earthquake. ENGSDS100093 Revision B. EDF. January 2011. (E) *Supersedes Revision A April 2010.*
3	3.3	ENGSGC100106	B	Study of the behaviour of the EPR inner containment wall beyond design-basis conditions. ENGSGC100106 Revision B. EDF. February 2011. (E) *Supersedes Revision A April 2010.*
3	3.3	ECEF040241	A	EPR - Equipment hatch and airlocks operating principles. ECEF040241 Revision A (TR 04-140). EDF. April 2004. (E)

3	3.3	SFLEYRC003022	F1	Hypothesis note on reactor building containment internals. SFL-EYRC-003022 Revision F1. Sofinel. October 2008. (E)
3	3.3	11787YR1221NT28B010001	D1	Reactor Building - Containment Internals – Hypothesis and methodology note. 11787 YR1221 NT 28B01 0001 Revision D1. EDF. October 2008. (E). Revision D1 is the English translation of 11787 YR1221 NT 28B01 0001 Revision D
3	3.3	ECEIG050051	L1	EPR Civil Engineering Standard. ECEIG050051 Revision L1. EDF. October 2009. (E)
3	3.3	SFLEZC008016	B	Concept Design of the MCR for ERP-France Specific Arrangements. SFLEZC008016 Revision B. SOFINEL. February 2008. (E)
3	3.3	ECEIG110015		MCR Acoustic Study Justification – Answers to 35-CE-1 and 35-CE-3. ECEIG110015. EDF. December 2010. (E)
3	3.3	ENGSGC110243	B	Pools and Tanks Liner Design Requirements and Methodology. ENGSGC110243 Revision B. EDF. February 2012. (E)
3	3.3	ECEIG060543	C1	Design, Construction and Testing of EPR Pool Liners. ECEIG060543 Revision C1. EDF. June 2009. (E) ECEIG060543 Revision C1 is the English translation of document ECEIG060543 Revision C.
3	3.3	ECEIG051339	B1	EPR General Hypothesis to Design Steel Structures. ECEIG051339 Revision B1. EDF. March 2009. (E) ECEIG051339 Revision B1 is the English translation of ECEIG051339 Revision B.
3	3.3	SFLEZC008003	C	Hypothesis note for the Nuclear Auxiliary Building SFL-EZC-00-8003 Revision C. Sofinel. March 2007. (E)
3	3.3	SFLEYRC0030020	E1	Hypothesis note on civil engineering design of the EPR FA3 Diesel Buildings. SFL-EYRC-0030020 Revision E1 (TR 08/549). Sofinel. October 2008. (E) SFL-EYRC-0030020 Revision E1 is the English version of document SFL-EYRC-0030020 Revision E
3	3.3	ENGSDS100268	B	UK EPR - Seismic Analysis of Foundation Raft, Design Report. ENGSDS100268 Revision B. EDF. April 2012. (E) *Supersedes Revison A February 2011.*
3	3.3	ENGSDS100269	B	UK EPR - Methodology for Seismic Analysis of NI buildings. ENGSDS100269 Revision B. EDF. April 2012. (E) *Supersedes Revison A February 2011.*
SUB-CHAPTER 3.4 – REFERENCES				
3	3.4	ENSNDR090183	D	Identification of High Integrity Components – components whose gross failure is discounted. ENSNDR090183 Revision D. EDF. October 2012. (E) *Supersedes Revison B December 2010.*
3	3.4	NEERF102070	D	Demonstration of integrity of High Integrity Components against fast fracture. Fracture Mechanic Analyses – Non Destructive Testing – Fracture Toughness. NEER-F 10.2070 Revision D. AREVA. August 2012. (E)

3	3.4	EPRRDC1705	B	Preliminary Safety Analysis Report, Section 3.6.1.1 “Design Transients”. EPRR DC 1705 Revision B. AREVA. (E)
3	3.4	PEPSFDC105	A	Mechanical Systems - Definition of Loadings and Criteria. PEPS-F DC 105 Revision A. AREVA. April 2012. (E)
3	3.4	REF0008		Technical guidelines for the design and construction of the next generation of nuclear power plants with pressurised water reactors. Adopted during the GPR / German experts plenary meetings held on October 19th and 26th, 2000. (E)
3	3.4	KT132012		Nuclear Safety Standards Commission (KTA) – Components of the reactor coolant pressure boundary of light water reactors. KT1 3201.2. June 1996. (E)
3	3.4	RCCM2007		Design and Construction Rules for mechanical components of PWR nuclear islands (RCC-M), Subchapters B 3100 and C 3100. AFCEN. 2007 Edition. (E)
3	3.4	PMLDC98	D	C Canteneur. Synthesis of ANSYS qualification report. PMLC DC 98 Revision D. AREVA NP. January 2009. (E)
3	3.4	NEERFDC28	D	F Barrère. EPR FA3 - Loop analyses - Dimensioning loadings applied to primary components supports and pipes. NEER-F DC 28 Revision D. AREVA. July 2007. (E)
3	3.4	REGGUIDE161	1	US Nuclear regulatory commission - Regulatory guide 1.61. Damping values for seismic design of nuclear power plants. Revision 1. March 2007. (E)
3	3.4	NEEGFDC41	D	S Francke. EPR 79/19 TE Steam Generator dynamic model and spring mass model of the partition plate. NEEG-F.DC.41 Revision D. AREVA. July 2007. (E)
3	3.4	6CS20059	C	P Hatquet. Reactor coolant pump parameters for the dynamic analysis. 6 CS 20059 Revision C. AREVA.July 2007. (E)
3	3.4	NFPMRDC68	E	S. Courtin. Physical validation synthesis report for SYSTUS computer code. NFPMR DC 68 Revision E. AREVA. 2008. (E)
3	3.4	NEPRG2007en0122		G. Hilber. Validation of ROLAST. NEPR-G/2007/en/0122. AREVA NP. May 2007. (E)
3	3.4	NFPMRDB1207	G	EPR FA3 – Primary Loops – Assembly. AREVA-NP drawing. NFPMR DB 1207 Revision G. AREVA. March 2009. (E)
3	3.4	NGPS12005en0076		E Brehm. Validation report S-TRAC. NGPS1/2005/en/0076. AREVA NP. (E)
3	3.4	KWUNSS96E2514	A	L Obereisenbuchner. Calculation of fluid dynamic loads for the loop after LOCA at 100% Power Operation and stretch-out. KWU NS-S/96/E2514 Revision A. AREVA. 1996. (E)
3	3.4	KWUNSS97E2505		L Obereisenbuchner. Fluid dynamic loads on the internals of the RPV after LOCA at 83.3% Power Operation (stretch-out). KWU NS-S/97/E2505. AREVA. 1997. (E)
3	3.4	ENGSDS050211	A1	E Champain. Design basis floor response spectra: EPR plant series. ENG SDS 05 0211 Revision A1. EDF. June 2009. ENG SDS 05 0211 Revision A1 is the English translation of ENG SDS 05 0211 Revision A.
3	3.4	NEERFDC30	B	F Barrère. EPR FA3 - Reactor block calculations - Preliminary load values on RPV internals. NEER-F DC 30 Revision B. October 2008. (E)
3	3.4	PEDCE9723		European Directive for Pressure Equipment. PED-CE 97/23. (E)

3	3.4	NEPRFDC457	A	Design of the overpressure protection CPP [RCPB] / CSP [MSS] for the FA3 EPR NEPR-F DC 457 Revision A. AREVA. 2009. (E)
3	3.4	NFPSDDC85	D	MANTA Code synthetic qualification assessment. NFPSD DC 85 Revision D. AREVA.
3	3.4	PSRRDC0020	B	Preliminary Safety Analysis Report, Section 3.6.1.8.1 “Overpressure Protection Analyses at Power”. PSRR DC 0020 Revision B. AREVA. (E)
3	3.4	NFPSDDC89	A	Science V2 Nuclear Code Package - Qualification Report. NFPSD DC 89 Revision A. AREVA. March 2004. (E)
3	3.4	NFPSDDC188	A	M N Jullion. Qualification Report FLICA IIIF - Version 3. NFPSD DC 188 Revision A. AREVA. January 2006. (E) and EP/TC/DC 1469 Revision A. AREVA. (E)
3	3.4	PMSCDC23	B	Preliminary Safety Analysis Report, Section 3.6.1.8.2 “Overpressure Protection Analyses in Cold Conditions”. PMSC DC 23 Revision B. AREVA.
3	3.4	EPRRDC1701	A	EPR Operation at 4250 MWth. EPRR DC 1701 Revision A. AREVA. (E)
3	3.4	EPRRDC1690	C	RPV Thermal-hydraulic Design: Rod Drop Time. EPRR DC 1690 Revision C. AREVA. (E)
3	3.4	NEERF100298	C	Methodology to determine the critical defects in terms of fast fracture for High Integrity Components. NEER-F 10.0298 Revision C. AREVA. December 2010. (E)
3	3.4	RSEM1997		RSE-M – In-Service Inspection Rules for the Mechanical Components of PWR Nuclear Power Islands. Appendix 5.4 – Determination of the main parameters. AFCEN. 1997 Edition supplemented by 1998, 2000 and 2005 modification. (E)
3	3.4	PEEMF101124	A	Manufacturing non destructive testing personnel qualification. PEEM-F101124 Revision A. AREVA. June 2010. (E)
3	3.4	NEERFDC231	B	E Abib. EPR CRGA validation test (MAGALY tests). NEER-F DC 231 B BPE. AREVA. April 2008. (E)
3	3.4	RCCM2007FM1060		Design and Construction Rules for Mechanical Components of PWR Nuclear Islands. RCC-M. AFCEN. June 2007 supplemented by RCC-M Modification Sheet FM 1060. (E)
3	3.4	NLEFDC176	A	System Design Manual – Full Length Rod Control System (RGL [CRDM]), P1. NLE-F DC 176 Revision A. AREVA. April 2008. (E)
3	3.4	NLEFDC147	A	System Design Manual – Full Length Rod Control System (RGL [CRDM]), P2 - System Operation. NLE-F DC 147 Revision A. AREVA. March 2008. (E)
3	3.4	NLEFDC148	A	System Design Manual – Full Length Rod Control System (RGL [CRDM]), P3 - System and component sizing. NLE-F DC 148 Revision A. AREVA. March 2008. (E)
3	3.4	NLEFDC149	A	System Design Manual – Full Length Rod Control System (RGL [CRDM]), P4 – Flow Diagrams. NLE-F DC 149 Revision A. AREVA. February 2008. (E)
3	3.4	NEERG2006en1583	F	R Faltermeier. Equipment Specification Control Rod Drive Mechanisms D142. NEER-G/2006/en/1583 Revision F. AREVA. September 2008. (E)
3	3.4	NEERFDC208	A	EPR KOPRA Tests - Rod Drop Times during Stability Test. NEER F DC 208 Revision A. AREVA. (E)

3	3.4	NEERFDC211	A	EPR KOPRA Tests - Short Drive Rod Configuration - Performance Tests - Synthesis Report for Validation Purposes. NEER F DC 211 Revision A. AREVA. (E)
3	3.4	NEERFDC40	D	E Abib. RPV Internals Equipment Specification. NEER-F DC 40 Revision D BPE. AREVA. April 2008. (E)
3	3.4	NEERFDB1308	D	Raimbault, C Hazelard. RPV Internals - General view of the internals and part list. NEER-F DB 1308 Revision D. June 2009. (E)
3	3.4	NEERFDB1227	E	E Abib, Denis. RPV Internals - Upper internals assembly. NEER-F DB 1227 Revision E BPE. June 2009. (E)
3	3.4	NEERFDB1281	E	Abib, Denis. RPV Internals - Control Rod Guide Assembly. NEER-F DB 1281 E BPE. AREVA. March 2008. (E)
3	3.4	NEERFDC47	D	Control Rod Guide Assembly Equipment Specification. NEER-F DC 47 Revision D. AREVA. (E)
3	3.4	NEERFDC61	C	J Jouniaux Corriger. EPR FA3 Contract A - RPV internals mechanical dimensioning - Upper internals 1st and 2nd categories analyses. NEER-F DC 61 Revision C BPE. AREVA. October 2007. (E)
3	3.4	NEERFDC74	B	J-L Chambrin. EPR FA3 Contract A – RPV Internal Mechanical Dimensioning 4th Category Analysis. NEER-F DC 74 Revision B BPE. AREVA. June 2007. (E)
3	3.4	NEPRFDC61	E	P Bertrand. RPV hydraulic design - Flow distribution and pressure losses. NEPR-F DC 61 Revision E BPE. AREVA. July 2008. (E)
3	3.4	NEPRFDC10	D	P Bertrand. RPV Thermal-hydraulic design: Core bypass. NEPR-F DC 10 Revision D BPE. AREVA. June 2008. (E)
3	3.4	NEPRFDC242	B	P Bertrand. RPV and Internals hydraulic design (upper plenum). NEPR-F DC 242 Revision B BPO. AREVA. May 2008. (E)
3	3.4	NEERG2007en1365	C	J Giebel, K MINI. Equipment specification for the nozzle closure of the in core instrumentation. NEER-G/2007/en/1365 Revision C BPE. AREVA. October 2008. (E)
3	3.4	NEERG2007en1367	C	J Giebel, K MINI. Equipment specification for the instrumentation lances of the in core instrumentation. NEER-G/2007/en/1367 Revision C BPE. AREVA. November 2008. (E)
3	3.4	NEERG2007en1368	C	J Giebel, K.MINI. Equipment specification for the RPV level and dome temperature measurement probes of the in core instrumentation. NEER-G/2007/en/1368 Revision C BPE. AREVA November 2008. (E)
3	3.4	NEERG00100291	E	Giebel, Muller. In core instrumentation - Nozzle closure. NEER-G- 00-100291 Revision E BPE. AREVA. February 2009. (E)
3	3.4	NEERG00100290	C	Giebel, Muller. In core instrumentation - Instrumentation lances. NEER-G-11-100290 Revision C. AREVA. January 2008 (E)
3	3.4	NEERG11100289	D	Giebel, Muller. In core instrumentation - Instrumentation lances Yoke types 1,2,3,4. NEER-G- 00-100289 Revision D BPE. AREVA. February 2009. (E)
3	3.4	NEERG00100629	D	Giebel, Muller. RPV level measurement probes with thermocouples. NEER-G- 00-100629 Revision D BPE. AREVA. February 2009. (E)
3	3.4	NEERG00100628	C	Giebel, Muller. RPV level measurement probes with thermocouples. NEER-G- 00-100628 Revision C BPE. AREVA. February 2009. (E)

3	3.4	NEERFDB1226	H	E Abib, Denis. RPV Internals - Lower internals assembly. NEER-F DB 1226 Revision H BPE. AREVA. September 2009 (E)
3	3.4	NEERFDB1229	G	E Abib, Denis. RPV Internals - Heavy reflector assembly drawing. NEER-F DB 1229 Revision G BPE. AREVA. June 2009 (E)
3	3.4	NEERFDB1231	C	E Abib, Denis. Lower RPV Internals - Flow distribution device in lower plenum. NEER-F DB 1231 Revision C BPE. AREVA. March 2008 (E)
3	3.4	NEELFDC21	B	E Messina. EPR FA3 Contract A - RPV internals mechanical dimensioning - Lower internals - Core barrel upper part - 1st and 2nd categories. NEEL-F DC 21 Revision B BPE. AREVA. November 2007. (E)
3	3.4	NEELFDC22	B	E Messina. EPR FA3 Contract A - RPV internals mechanical dimensioning - Lower internals - Lower support plate - 1st and 2nd categories. NEEL-F DC 22 Revision B BPE. AREVA. November 2007 (E)
3	3.4	NEERFDC76	C	J-L Chambrin. EPR FA3 Contract A - RPV internals mechanical dimensioning - Heavy reflector analysis - Design validation - Categories 1 and 2. NEER-F DC 76 Revision C BPE. AREVA. September 2007 (E)
3	3.4	NEPDFDC67	D	O Cartier. Thermo-hydraulic justification of FA3 Heavy reflector. NEPD-F DC 67 Revision D BPE. AREVA. January 2009. (E)
3	3.4	ENFCAE080186	A	Design policy concerning the periodic safety tests for the EPR plant. ENFCAE080186 Revision A. EDF. December 2008 (E). ENFCAE080186 Revision A is the English translation of ENFCRI050121 Revision B
SUB-CHAPTER 3.5 – REFERENCES				
3	3.5	RCCM2007		Design and Construction Rules for Mechanical Components of PWR Nuclear Islands. RCC-M 3rd modification. AFCEN. June 2007. (E)
3	3.5	NFEN199211		Eurocode 2 - Design of concrete structures. NF EN 1992-1-1. AFNOR. 2005. (E)
3	3.5	ETCC2010		EPR Technical Code for Civil works. ETC-C 2010 Edition. AFCEN. 2010. (E)
3	3.5	ENGSGS110015	E	UKEPR - GDA - UK Companion Document to AFCEN ETC-C. ENGSGC110015 Revision E. EDF. August 2012. (E) *Supersedes Revision D April 2012.*
SUB-CHAPTER 3.6 – REFERENCES				
3	3.6	ECEF0100515	F1	Summary Report on Functional Requirements Analyses per Plant System. ECEF0100515 Revision F1. EDF. June 2010. (E)
3	3.6	RCCEDEC2005		Design and Construction Rules for Electrical Components of Nuclear Islands. RCC-E. AFCEN Edition. December 2005. (E)

3	3.6	ENSEMD050222	C	Cahier des données de projet complétant les exigences du RCC-E déc. 2005 pour EPR. [Project data book completing the RCC-E December 2005 requirements for EPR.] ENSEMD050222 revision C. EDF. May 2007.
3	3.6	ECEF040759	D1	EPR - Preliminary list of plant functions to be qualified for accident ambient conditions. ECEF040759 Revision D1. EDF. 2008 (E)
3	3.6	CEI60780		Nuclear power plants – Electrical safety equipment – Qualification. CEI 60780. IEC. 1998 (E)
3	3.6	KTA3501		Reactor Protection System and Monitoring Equipment of the Safety System. KTA 3501. TUV. 1985 (E)
3	3.6	KTA3701		General Requirements for the Electrical Power Supply of the Safety System in Nuclear Power Plants. KTA 3701. TUV. 1999 (E)
3	3.6	IEEE323		IEEE Standard for Qualifying Class 1E Equipment for Nuclear Power Generating Stations. IEEE Standard 323. IEEE. 1983 (E)
3	3.6	EUR16246E		A comparison of European practices for the qualification of electrical and I & C equipment important to safety for European LWR power plant. EUR 16246 E. Reactor Safety Working Group. 1996 (E)
SUB-CHAPTER 3.8 – REFERENCES				
3	3.8	RCCM2007FM1060		Design and Construction Rules for Mechanical Components of PWR Nuclear Islands. RCC-M 3rd modification. AFCEN. June 2007 supplemented by RCC-M Modification Sheet FM 1060. (E)
3	3.8	RSEM1997		RSE-M – In-Service Inspection Rules for the Mechanical Components of PWR Nuclear Power Islands. Appendix 5.4 – Determination of the main parameters. AFCEN. 1997 Edition supplemented by 2005 modification. (E)
3	3.8	RCCEDEC2005		Design and Construction Rules for Electrical Components of Nuclear Islands. RCC-E. AFCEN Edition. December 2005. (E)
3	3.8	ENSEMD050222	C	Book of Project data completing the RCC-E December 2005 requirements for EPR. ENSEMD050222 revision C. EDF. May 2007. (E)
3	3.8	ETCC2010		EPR Technical Code for Civil works. ETC-C 2010 Edition. AFCEN. 2010. (E)
3	3.8	ENGSIN050312	B	ETC-F. EPR Technical Code for Fire Protection. ETC-F Version G. ENGSIN050312 Revision B. EDF. August 2007. (E)
3	3.8	NESPF090480		UKEPR. NESP-F.09.0480. AREVA. September 2009. (E)
3	3.8	NEEMF091383	A	UK EPR. RCCM requirements for design analysis of class 1/2/3 piping. Review of ASME reports. NEEM-F 09 1383 Revision A. AREVA. December 2009. (E)
3	3.8	REGGUIDE1124		Service Limits and Loading Combinations for Class 1 Linear-Type Supports. Regulatory Guide 1.124. NRC. 2007
3	3.8	REGGUIDE1130		Service Limits and Loading Combinations for Class 1 Plate-And-Shell-Type Supports. Regulatory Guide 1.130. NRC. 2007

3	3.8	NFEN473		Essais non destructifs — Qualification et certification du personnel END — Principes généraux. [Non-destructive testing – Qualification and certification of END employees – General principles.] NF EN 473. AFNOR, 2008
3	3.8	ENGSGC090215	A	Development Path for ETC-C (Rules for EPR Civil Works) ENGSGC090215 Revision A. EDF. September 2009. (E)
3	3.8	ENGSGC050076	B	ETC-C (EPR Technical Code for Civil works). ENGSGC050076 Revision B. EDF. April 2006. (E)
3	3.8	ENGSGS110015	E	UK Companion Document to AFCEN ETC-C - Part 1. ENGSGC110015 Revision E. EDF. August 2012. (E) *Supersedes Revision D April 2012.*
3	3.8	NFEN1990		Eurocode: Basis of Structural Design. EN 1990:2002. (E)
3	3.8	NFEN199111		Eurocode 1: Actions on structures. EN 1991-1-1:2002/AC:2009. (E)
3	3.8	NFEN199211		Eurocode 2: Design of concrete structures. EN 1992-1-1:2004/AC:2010. (E)
3	3.8	NFEN199311		Eurocode 3: Design of steel structures. EN 1993-1-1: 2005/AC:2009. (E)
3	3.8	NFEN19971		Eurocode 7 – Geotechnical design. EN 1997-1:2004/AC:2009. (E)
3	3.8	IAEANSG17		IAEA Safety Standards Series. Protection against Internal Fires and Explosions in the Design of Nuclear Power Plants. Safety Guide No. NS-G-1.7. IAEA. 2004. (E)
3	3.8	FGO2004		Arrêté du 22 mars 2004 portant approbation de dispositions complétant et modifiant le règlement de sécurité contre les risques d'incendie et de panique dans les établissements recevant du public (dispositions relatives au désenfumage). [Order of 22 March 2004 approving provisions supplementing and amending the rules of safety against fire and panic in establishments open to the public (provisions relating to smoke).] French Government order, 2004
3	3.8	NFEN135012		Classement au feu des produits de construction et éléments de bâtiment - Partie 2: classement à partir des données d'essais de résistance au feu à l'exclusion des produits utilisés dans les systèmes de ventilation. [Fire classification of construction products and building elements - Part 2: classification from the test data of fire resistance with the exclusion of products used in ventilation systems.] NF EN 13501-2. AFNOR. May 2004
3	3.8	HI8204022	B	Qualification file of Fire Code Magic version 4.1.1. HI-82/04/022 Revision B. EDF. December 2005. (E)
3	3.8	PEERF101989	A	Summary of the FMA approaches introduced in RSE-M Appendix 5.4 and RCC-MR Appendix A16 - illustration of their pertinence through an example. PEER-F 10.1989 Revision A. AREVA. December 2010. (E)
3	3.8	FGO1974		Arrêté du 26 février 1974 relatif à la construction du circuit primaire principal des chaudières nucléaires à eau. [Order of 26 February 1974 for the construction of the main primary circuit nuclear boiler.] French Government order (1974) modified in 2000

APPENDIX 3 - REFERENCES

3	App 3	CODEASTER1		Description of the code (http://www.code-aster.org). (E)
3	App 3	ENGSDS080130 / A002047	A	Qualification :ENGSDS080130 A and A0.02.047 A (available at http://www.code-aster.org). (E)
3	App 3	EUROPLEXUS1		Description of the code (http://europlexus.jrc.ec.europa.eu/). (E)
3	App 3	NFPMRDC159	A	I Duvernoy. JUPITER qualification summary report. NFPMR DC 159 Revision A. AREVA. July 2005. (E)
3	App 3	ENGSOFTN4		O. Ibidapo-Obe, A. Sofoluwe, O. Abass. Householder/QI algorithm for the functional eigenproblem. Advances in Engineering software, volume 4, issue 4. October 1982. (E)
3	App 3	AIAAVOL32	Vol 3(2)	R J. Guyan. Reduction of stiffness and mass matrices. AIAA Journal, volume 3(2). 1965. (E)
3	App 3	HI8204024	B	MAGIC software version 4.1.1: Mathematical model. HI-82/04/024 Revision B. EDF-SA. 2005. (E)
3	App 3	HI8204022	B	Qualification file of Fire Code Magic version 4.1.1. HI-82/04/022 Revision B. EDF. December 2005. (E)
3	App 3	PRECONT1		PRECONT qualification documents and users guide are available via EDF at COB office on demand.
3	App 3	60590011UM008	A	Allong Module: Calculation of cable elongation during tensioning. 60590011-UM-008 Revision A. October 1997. (E)
3	App 3	09456010RP004	A	Calpdom Module: Decomposition of dome cables into straight segments and circular arcs. 09456010-RP-004 Revision A. October 2008. (E)
3	App 3	09456010RP001	A	Coth Module: Calculation of intersection point elevations of horizontal cables with regularly spaced radial planes. 09456010-RP-001 Revision A. October 2008. (E)
3	App 3	09456010RP002	A	Cotv Module: Calculation of vertical cable spacing at a given elevation. 09456010-RP-002 Revision A. October 2008. (E)
3	App 3	09456010RP003	A	Cotdom Module: Calculation of elevation of a dome cable at the intersection with a cylinder with a given radius. 09456010-RP-003 Revision A. October 2010. (E)
3	App 3	09456010RP007	A	Defcab Module: Encoding of horizontal and vertical cables (original version replaced by EDICAB). 09456010-RP-007 Revision A. October 2010. (E)
3	App 3	09456010RP005	A	Defcom Module: Encoding and discretisation of dome cables (original version replaced by EDICAB). 09456010-RP-005 Revision A. October 2010. (E)
3	App 3	60590011UM007	A	Destens Module: Calculation and design of cable tension as a function of curvilinear abscissa. 60590011-UM-007 Revision A. October 1997. (E)
3	App 3	60590011UM007	A	Discr Module: Discretisation of an encoded horizontal or vertical cable (see EDICAB). 60590011-UM-007 Revision A. October 1997. (E)
3	App 3	09456010RP006	A	Discrdom Module: Discretisation of encoded dome (using DEFDOM). 09456010-RP-006 Revision A. October 2008. (E)
3	App 3	09456010RP008	A	Ecartdom/distdom Module: Calculation of spacing (on a plane) between points of intersection of a dome cable layer with cylinder of a given radius. 09456010-RP-008 Revision A. October 2008.

3	App 3	60590011UM002	B	Edicab Module: Encoding and discretisation of all types of cables with graphic aid (replaces DEFCAB, DEFDOM, DISCR and DISCRDOM). 60590011-UM-002 Revision B. February 1998. (E)
3	App 3	60590011UM006	A	Fnodal Module: Calculation of vector loading due to pre-stressing applied to finite element model. 60590011-UM-006 Revision A. December 1997. (E)
3	App 3	09456010RP009	A	Mistens Module: Calculation of cable tension as a function of curvilinear abscissa. 09456010-RP-009 Revision A. October 2008. (E)
3	App 3	09456010RP010	A	Recol Module: Re-fusing of vertical and gamma cables. 09456010-RP-010 Revision A. October 2008. (E)
3	App 3	ProMISS3D		Features ProMISS3D (http://www.mssmat.ecp.fr/structures/perso/miss) (E)
3	App 3	HT6205009	A1	Quality sheet and qualification file ProMISS 3D V1.3 and chaining with Code_Aster version 7. H-T62-05-009 Revision A1. EDF-SA. July 2009. (E) H-T62-05-009 Revision A1 is the English translation of H-T62-05-009 Revision A.
3	App 3	HT62200903724EN	1	Quality sheet and qualification file for ProMISS 3D version 1.4 and its chaining with Code_Aster version 9. H-T62-2009-03724-EN Revision 1.0. EDF-SA. April 2012. (E) H-T62-2009-03724-EN Revision 1.0 is the English translation of H-T62-2009-03724-FR Revision 1.0.
3	App 3	300511	A1	Validation of calculation tools used in soil-structure interaction – Summary Report. 30-05/11 Revision A1. August 2010. (E)
3	App 3	PEERFDC89	A	S Lame. ROCOCO Programme Qualification Summary Report. PEER-F DC 89 Revision A FIN. AREVA. October 2012. (E)
3	App 3	NEPRG2007en0122	A	G. Hilbert. Validation of ROLAST. NEPR-G/2007/en/0122 Revision A. AREVA. 2007. (E)
3	App 3	NGPS12005en0076		Validation of S-TRAC. NGPS1/2005/en/0076. AREVA. 2005. (E)
3	App 3	STRUDYN1		STRUDYN. Theoretical Manual. Version 1/1996. (E)
3	App 3	STRUDYN2		STRUDYN. User Information Manual. Version 1/2000. (E)
3	App 3	STRUDYN3		STRUDYN. Verification Manual. Part 1 – 5, 1/1996. (E)
3	App 3	USNRC192	Rev2	Regulatory Guide 1.92 Revision 2. USNRC. July 2006. (E)
3	App 3	R62120282	P1	Benchmark for the computer code STRUDYN (Quality Assurance) Part 1. KWU-Arbeitsbericht R621/202/82. (E)
3	App 3	R621830166	P2	Benchmark for the computer code STRUDYN (Quality Assurance) Part 2. KWU-Arbeitsbericht R621/83/0166. (E)
3	App 3	PEERFDC88	A	S Lame. SYSTAR Programme Qualification Summary Report. PEER-F DC 88 Revision A FIN. AREVA. October 2012. (E)
3	App 3	SYSTUS		ESI-group web site – description of the code. (http://www.esi-group.com/products/multiphysics/systus). (E)
3	App 3	NFPMRDC68	E	S. Courtin. Qualification synthesis report of SYSTUS computer code, integrating SYS* tools, the F.E. CRACK BLOCK and the NUKE module. NFPMR DC 68 Revision E. AREVA. 2008. (E)
3	App 3	ENSGG0000355	A1	P A Naze. Principles of the CYBERQUAKE computer code. E-N-S-GG/00-00355 Revision A1. May 2000. (E)

3	App 3	ENSGC0000356	A1	P A Naze. CYBERQUAKE computer code validation report. E-N-S-GC/00-00356 Revision A1. May 2000. (E)
3	App 3	ANSYS		Description of the code (http://www.ansys.com). (E)
3	App 3	COBEF		Description of the code (http://www.coyne-et-bellier.fr/dun/dsc/logiciels.html). (E)
3	App 3	0500NT001	A	S. Cazadieu. Hercule Computer Code Description. 0500 NT 001 Revision A. IOSIS. May 2009. (E)
3	App 3	0500NT002	A	S. Cazadieu. Hercule Finite Element System – Validation Tests. 0500 NT 002 Revision A. IOSIS. May 2009. (E)
3	App 3	NASTRAN		Description of the code (http://www.mscsoftware.com). (E)
3	App 3	NASTRAN2		Complement for the description of the code (http://www.openchannelfoundation.org/projects/NASTRAN) (E)
CHAPTER 4				
SUB-CHAPTER 4.1 REFERENCES				
4	4.1	NFPSCDC1042	C	S Laurent. EPR FA3 - NSSS Operating Parameters. NFPSC DC 1042 Revision C. AREVA. December 2006. (E)
4	4.1	FFDC05182	B	EPR HARMONI® RCCA - Description, Functional Requirements and Materials Properties. FF DC 05182 Revision B. AREVA. July 2009. (E)
SUB-CHAPTER 4.2 – REFERENCES				
4	4.2	FS10001387	1.0	N Sins. Summary report of handling tests on improved AFA 3G Grids. FS1-0001387 Revision 1.0. AREVA. July 2010. (E) (This document contains proprietary information)
4	4.2	FS10001493	2.0	F Laudarin. AFA 3GLE fuel assembly for the EPRTM reactor – Synthesis of the mechanical tests performed on a fuel assembly mock-up. FS1-0001493 Revision 2.0. AREVA. December 2011. (E) (This document contains proprietary information)
4	4.2	FS10004218	2.0	J Pacull. Synthesis of AFA 3GLE endurance tests in Hermes P. FS1-0004218 Revision 2.0. AREVA. January 2012. (E) (This document contains proprietary information)
4	4.2	FS10004489	1.0	J-F Desseignes. AFA 3GLE fuel assembly for EPR reactor pressure loss measurements synthesis report. FS1-0004489 Revision 1.0. AREVA. February 2012. (E) (This document contains proprietary information)
4	4.2	FS10005295	1.0	L Lalli. Buckling strength of the AFA 3GLE grid in accident conditions – Test synthesis report. FS1-0005295 Revision 1.0. AREVA. March 2012. (E) (This document contains proprietary information)
4	4.2	FS10000096EN	3.0	F DeThe. AFA 3GLE fuel assembly for FA3 EPRTM reactor - Product Sheet. FS1-0000096-EN Issue 3.0. July 2009. (E) FS1-0000096-EN is the English translation of FS1-0000096
4	4.2	TFJCDC01191	A	V.Rebeyrolle. Presentation of M5 cladding tube manufacturing route. TFJC DC 01191. AREVA. July 1998. (E) (This document contains proprietry information)

4	4.2	FFDC00267	D	JP.Mardon. M5™ Properties Data BE . FF DC 00267. AREVA. January 2005. (E) (This document contains proprietry information)
4	4.2	FFDC05182	B	EPR HARMONI® RCCA Description, Functional Requirements and Materials Properties. FF DC 05182 Revision B. AREVA. July 2009. (E)
4	4.2	FFDC5549	A	R Batail. Fuel Rod Thermal/Mechanical Design Bases. FF DC 5549 Revision A. AREVA. July 2009. (E)
4	4.2	FFDC05063EN	C	F Regnier. Fuel Assembly Mechanical Design General Methodology AFA® and AGORA® Fuel Assembly Types. FF DC 05063-EN Revision C. AREVA. July 2009. (E)
4	4.2	FS10000607	3.0	Nuclear Fuel - Specification and Justification of Design and Safety Criteria. FS1-0000607 Revision 3.0. AREVA. October 2010. (E)
4	4.2	ENCNCA100114	B	Long term storage of spent fuel - Design criteria. ENCNCA100114 Revision B. EDF. August 2010. (E)
4	4.2	FS10000108EN	4	A. Dalmais. Reactor EPR™ FA3 - Evaluation of Thermo-mechanical Behaviour of AFA 3GLE UO2 and UO2-Gd203 Fuel Rods. FS1-0000108-EN Issue 4. AREVA. July 2009. (E) FS1-0000108-EN is the English translation of FS1-0000108
4	4.2	PEPCFDC18	A	C Yver and F Pairot. Pellet-Clad Interaction – Transients Analysis. PEPC-F DC 18 Revision A. AREVA. October 2010. (E)
4	4.2	FS10000099EN	3	L.Lalli. AFA 3GLE Fuel assembly for FA3 EPR reactor - Mechanical design analysis report. FS1-0000099-EN Issue 3. AREVA. July 2009. (E) FS1-0000099-EN is the English translation of FS1-0000099
4	4.2	FS10000103EN	3	J.F. Desseignes. Thermal-Hydraulic Design Report AFA 3GLE Fuel assembly for FA3 EPR reactor. FS1-0000103-EN Issue 3. AREVA. July 2009. (E)
4	4.2	FS10001671	1	E Mery de Montigny. AFA3GLE fuel assembly for UK EPR Expected in-core behaviour regarding fuel assembly bow. FS1-0001671 Revision 1.0. AREVA. September 2010. (E)
4	4.2	RCCC2005		Design and construction rules for fuel assemblies of nuclear plants. RCC-C. AFCEN. 2005. (E)
4	4.2	FS10001318	1.0	Status on Crud Monitoring and Acceptability. FS1-0001318 Revision 1.0. AREVA. July 2010. (E)
4	4.2	PEPCFDC32	A	EPR™ UK - GDA Assembly Bow - Impact on Safety Studies. PEPC-F DC 32 Revision A. AREVA. December 2010. (E)
SUB-CHAPTER 4.3 – REFERENCES				
4	4.3	NFPSDDC89	A	S. Rauck. Science V2 Nuclear Code Package - Qualification Report. NFPSD DC 89 Revision A. AREVA. March 2004. (E)
4	4.3	NFPSCDC285	A	S Laurent. Fuel Management - Neutronic Design Report (Science Calculations) (Update 4500 MWth). NFPSC DC 285 Revision A. AREVA. September 2004. (E)
4	4.3	NFPSCDC284	B	C. Hove. Core Reactivity Control (update 4500 MWth). NFPSC DC 284 Revision B. AREVA. January 2006. (E)

4	4.3	NFPSCDC286	B	S Laurent. Neutronic Data for Transient Analyses (Update 4500 MWth). NFPSC DC 286 Revision B. AREVA. February 2006. (E)
4	4.3	NFPSCDC283	C	A. Delumley. Residual Decay Heat Curves for System Design and Accident Analysis (Update 4500 MWth). NFPSC DC 283 Revision C. AREVA. November 2005. (E)
4	4.3	EPRRDC1685	C	R Gagner. EPR sizing at 4500MWth. EPRR DC 1685 Revision C. AREVA. February 2004. (E)
SUB-CHAPTER 4.4 – REFERENCES				
4	4.4	TFJCDC906	A	A Chotard. Melting point of UO2 and (U,Gd)O2 fuels. TFJC DC 906EN Revision A. AREVA. September 1997. (E)
4	4.4	FFDC03429		A Chotard and K Sawyer. Study on the melting point of oxide fuels and its changes with burnup. FFDC03429. AREVA. December 2006 (E)
4	4.4	NEPRFDC10	D	P Bertrand. RPV T/H design: Core bypass. NEPR-F DC 10 Revision D. AREVA. June 2008. (E)
4	4.4	FS10000531	1.0	Void fraction for Fuel Assembly equipped with M5® cladding tube. FS1-0000531 Revision 1.0 BPE. AREVA. March 2010. (E)
4	4.4	EPDDC511	A	J P Bourteele. FC - FRAMATOME Critical Heat Flux correlation for AFA-2G and AFA-3G fuel assemblies description. EPD DC 511 Revision A. AREVA. April 2001. (E)
4	4.4	ISBN139781560324850		L S Tong and Y S Tang. Boiling Heat Transfer and Two-Phase Flow. ISBN-13: 9781560324850. Taylor & Francis (CRC Press). 2007. (E)
4	4.4	NFPSCDC1042	C	S Laurent. EPR - FA3 NSSS Operating Parameters. NFPSC DC 1042 Revision C. AREVA. December 2006 (E)
4	4.4	FS10001318	1.0	Status on Crud Monitoring and Acceptability. FS1-0001318 Revision 1.0. AREVA. July 2010. (E)
4	4.4	FS10001366	2.0	F Filhol. UK EPR Reactor - CHF in Peripheral Region. FS1-0001366 Revision 2.0. AREVA. July 2010. (E).
4	4.4	NFPSDDC188	A	M N Jullion. Qualification Report FLICA IIIF - Version 3. NFPSD DC 188 Revision A. AREVA. January 2006. (E)
APPENDIX 4 – REFERENCES				
4	App 4	NFPSDDC89	A	Science V2 Nuclear Code Package - Qualification Report. NFPSD DC 89 Revision A. AREVA. March 2004. (E)
4	App 4	EPDSDC150	C	S Ebalard. ORIGEN-S: Qualification manual. EPDS DC 150. Revision C. AREVA. November 2005. (E)
4	App 4	NFPSDDC188	A	M N Jullion. Qualification Report FLICA IIIF - Version 3. NFPSD DC 188 Revision A. AREVA. January 2006. (E)
4	App 4	NFPSDDC119	-	P Dolleans. STAR-CD Qualification On Dilution Phenomena. NFPSD DC 119. AREVA. October 2004. (E).
4	App 4	NEPDFDC10160	B	B. Durand. STAR-CD Version 4.02 - Physical Validation Synthesis Report. NEPD-F DC 10160 Revision B. AREVA. February 2010. (E)
CHAPTER 5				

SUB-CHAPTER 5.1 – REFERENCES				
5	5.1	NESSFDC534	A	System Design Manual - Reactor Coolant System (RCS) - Part 3: System and component sizing. NESS-F DC 534 Revision A. AREVA. April 2009 (E)
5	5.1	NESSFDC538	A	System Design Manual - Reactor Coolant System (RCS) - Part 2: System Operation. NESS-F DC 538 Revision A. AREVA. May 2009 (E)
5	5.1	NESSFDC545	A	System Design Manual - Reactor Coolant System (RCS) - Part 4: Flow diagrams. NESS-F DC 545 Revision A. AREVA. June 2009 (E)
5	5.1	NESSFDC613	A	System Design Manual - Reactor Coolant System (RCS) Part 5 – Instrumentation and Control. NESS-F DC 613 Revision A. AREVA. November 2009. (E)
SUB-CHAPTER 5.2 – REFERENCES				
5	5.2	ECEMA040920	C1	Application of the break preclusion assumption in the main reactor coolant and steam lines of the EPR FA3. ECEMA040920 Revision C1. EDF. (E)
5	5.2	ENSNDR080245	A	Break Preclusion in reactor main coolant lines and main steam lines. Positioning of the concept and associated safety requirements . ENSNDR080245 Revision A. EDF. (E)
5	5.2	NFPMTDC39	C	EPR RF002 - Alloy 690 material data file. NFPMT DC 39 Revision C. AREVA. December 2004. (E)
5	5.2	SAP2006	1	UK Health and Safety Executive (HSE). Safety Assessment Principles for Nuclear Facilities. 2006 Edition Revision 1. January 2008 (E)
5	5.2	TAGTAST16	3	UK Health and Safety Executive (HSE). Technical Assessment Guide. Integrity of Metal Components and Structures. TAG T/AST/16 Issue 3. August 2008 (E)
5	5.2	IJPVP78		R. Bullough, F. M. Burdekin, O. J. V. Chapman, V. R. Green, D. P. G. Lidbury, J. N. Swingler, R. Wilson. 'The Demonstration of Incredibility of Failure in Structural Integrity Safety Cases', International Journal of Pressure vessels and Piping, 78, 2001, 539 – 552 (E)
5	5.2	RCCM2007FM1060		Design and Construction Rules for mechanical components of PWR nuclear islands (RCC-M). (See RCCM Subsection B) AFCEN. June 2007 edition supplemented by RCC-M Modification Sheet FM 1060. (E).
5	5.2	ECEMA101028	A	F Chavigny. Demonstration of the accessibility and controllability for in-service inspection of the structural integrity components. ECEMA101028 Revision A. EDF. April 2010. (E)
5	5.2	PEEMF110505	C	Ultrasonic examination of MCL homogeneous and dissimilar metal welds. PEEM-F 11.0505 Revision C. AREVA. March 2012. (E)
SUB-CHAPTER 5.3 – REFERENCES				
5	5.3	NEERFDC19	C	EPR - RPV Mechanical dimensioning. NEER-F DC 19 Revision C. AREVA. May 2007. (E)
5	5.3	NFPMRDC1145	J	FA3 RPV Equipment Specification. NFPMR DC 1145 Revision J. AREVA. March 2009. (E)

5	5.3	NEERFDB1206	D	FA3 RPV equipment drawing. NEER-F DB 1206. Revision D. AREVA. 2008 (E)
5	5.3	NEERFDB1204	D	Reactor Pressure Vessel detailed drawing - CRDM and Instrumentation Adaptors. NEER-F DB 1204 Revision D. AREVA. September 2008. (E)
5	5.3	PEERF100134	A	Definition of the hierarchy of the main reports relating to design and manufacture primary component. PEER-F 100134 Revision A. AREVA. March 2010. (E)
5	5.3	NEERFDC77	C	FA3 - RPV Assessment of Sensitive Areas. NEER-F DC 77 Revision C. AREVA. April 2008. (E)
5	5.3	NFPMRDB1200	G	Reactor Pressure Vessel: General Assembly. NFPMR DB 1200 Revision G. AREVA. January 2009. (E)
5	5.3	NFPMRDB1201	F	Reactor Pressure Vessel detailed drawing - closure. NFPMR-DB 1201 Revision F. AREVA. November 2008. (E)
5	5.3	NFPMRDB1202	G	Reactor Pressure Vessel: Inlet Nozzle - Outlet Nozzle. NFPMR-DB 1202 Revision G. AREVA. January 2009. (E)
5	5.3	NEERFDB1203	D	Reactor Pressure Vessel detailed drawing - radial guides. NEER-F DB 1203 Revision D. AREVA. September 2008 (E)
5	5.3	NEERFDB1205	D	Reactor Pressure Vessel detailed drawing - vent pipe and dome thermocouple. NEER-F DB 1205. Revision D. AREVA. September 2008. (E)
5	5.3	NEERF102070	D	Demonstration of integrity of High Integrity Components against fast fracture. Fracture Mechanic Analyses – Non Destructive Testing – Fracture Toughness. NEER-F 10.2070 Revision D. AREVA. August 2012. (E)
5	5.3	PEEMF101134	D	UK Technical report on the manufacturing non destructive testing to be qualified. PEEM-F.101134 Revision D. AREVA. December 2010. (E)
5	5.3	PEEMF110505	C	Ultrasonic examination of MCL homogeneous and dissimilar metal welds. PEEM-F 11.0505 Revision C. AREVA. March 2012. (E)
5	5.3	PEERFDC78	A	UK EPR ALARP justification of manufacturing inspection techniques proposed for MCL welds of the UK EPR. PEER-F DC 78 Revision A. AREVA. March 2012. (E)
5	5.3	NEEMF100179	E	UK EPR™ Avoidance of Fracture- Fracture mechanics and minimum fracture toughness aspects: Materials Data. NEEM-F 10.0179 Revision E. AREVA. November 2010. (E)
5	5.3	NEERFDC25	C	RPV Irradiation Surveillance Program. NEER-F DC 25 Revision C. AREVA. 2008. (E)
5	5.3	PEPDF100428		Evaluation of dpa rate on UK EPR™ RPV. PEPD-F 10.0428. AREVA. December 2010. (E)
5	5.3	HT27201200181	1.0	RPV irradiation surveillance programme (ISP) - Design Principles for UK. EPR H-T27-2012-00181 Revision 1.0. EDF. February 2012. (E)
5	5.3	NEERF100697	A	Relation between operating domain and (P, T) curve issued from fast fracture analysis. NEER-F 100697 Revision A. AREVA. April 2010. (E)
SUB-CHAPTER 5.4 – REFERENCES				
5	5.4	NEEGFDC4	D	EPR FA3 – Equipment specification for the RCP (Reactor Coolant Pumps). NEEG-F DC 4 Revision D. AREVA NP. 2008. (E)

5	5.4	PEEOFDC167	A	UK EPR - Reactor Coolant Pump Characteristics. PEEO-F DC 167 Revision A. AREVA. August 2012. (E). PEEO-F DC 167 Revision A is the English translation of JSPM JSR 6GA20735 Revision B.
5	5.4	NEERF102070	D	Demonstration of integrity of High Integrity Components against fast fracture. Fracture Mechanic Analyses – Non Destructive Testing – Fracture Toughness. NEER-F 10.2070 Revision D. AREVA. August 2012. (E)
5	5.4	PEEMF110567	A	Fracture toughness properties of repair welds in cast pump casing. PEEM-F 11.0567 Revision A. AREVA. March 2011. (E)
5	5.4	PEEMF102218	B	UK EPR Technical Proposition for the NDT examination of major repair welds of the primary pump casing. PEEM-F 10.2218 Revision B. AREVA. September 2011. (E)
5	5.4	PEEOF100715	C	Non destructive test performed on the Reactor Coolant Pump flywheel during manufacturing. PEEO-F 10.0715 Revision C. AREVA. December 2011. (E)
5	5.4	ECEMA111847	A	Principles for the RCP Flywheel In Service Inspection. ECEMA111847 Revision A. EDF. August 2011. (E)
5	5.4	NEEMF100179	E	UK EPR Avoidance of Fracture- Fracture mechanics and minimum fracture toughness aspects: Materials Data. NEEM-F 10 0179 Revision E. AREVA. November 2010. (E)
5	5.4	1GA20230	D	Reactor Coolant Pumps. EPR FA3 – Outline drawing. JSPM JSR 1GA20230 Revision D. AREVA NP. 2008. (E)
5	5.4	1GA20341	B	GMPP [RCP] EPR FA3 – Ensemble joint d'arbre et DEA. [Reactor Coolant Pumps EPR FA3 - Shaft Sealing System] JSPM JSR 1GA20341 Revision B. AREVA NP. 2008.
5	5.4	NEEGFDB1200	H	C Hazelard. Générateur de vapeur 79/19TE - Plan d'ensemble - GV droit et gauche. [Steam generator 79/19TE - Layout – SG right and left] NEEG-F DB 1200 Revision H. AREVA NP - FA3 Project. November 2008
5	5.4	NFPMGDB1201	G	C Hazelard. Générateur de vapeur 79/19TE - Vue extérieure - GV droit et gauche. [Steam generator 79/19TE - Exterior view – Steam Generator right and left] NFPMG DB 1201 Revision G. AREVA NP - FA3 Project. November 2008
5	5.4	NFPMGDB1202	H	C Hazelard. Générateur de vapeur 79/19TE - Vue extérieure – Coupes et vues - GV droit et gauche. [Steam generator 79/19TE - Exterior view – Steam Generator – cross-sections and layout - right and left] NFPMG DB 1202 Revision H. AREVA NP - FA3 Project. November 2008
5	5.4	NFPMGDB1204	H	C Hazelard. Générateur de vapeur 79/19TE - Boîte à eau primaire – Ensemble et details - GV droit. [Steam generator 79/19TE – Primary water box – General and details – Steam Generator right] NFPMG DB 1204 Revision H. AREVA NP - FA3 Project. November 2008
5	5.4	NFPMGDB1205	H	C Hazelard. Générateur de vapeur 79/19TE - Boîte à eau primaire – Ensemble et details - GV gauche. [Steam generator 79/19TE – Primary water box – General and details – Steam Generator left] NFPMG DB 1205 Revision H. AREVA NP - FA3 Project. November 2008

5	5.4	NFPMGDB1208	G	C Hazelard. Générateur de vapeur 79/19TE - Enceinte secondaire – Vue générale - GV droit. [Steam generator 79/19TE – Secondary side casing – General view – Steam Generator right] NFPMG DB 1208 Revision G. AREVA NP - FA3 Project. November 2008
5	5.4	NFPMGDB1209	G	C Hazelard. Générateur de vapeur 79/19TE - Enceinte secondaire – Vue générale - GV gauche. [Steam generator 79/19TE – Secondary side casing – General view – Steam Generator left] NFPMG DB 1209 Revision G. AREVA NP - FA3 Project. November 2008
5	5.4	NEEGFDB1212	D	D Venturi. Générateur de vapeur 79/19TE – Faisceau tubulaire et nomenclature - GV droit et gauche. [Steam generator 79/19TE – Tube bundle and nomenclature - Steam Generator right and left] NEEG-F DB 1212 Revision D. AREVA NP - FA3 Project. June 2007
5	5.4	NEEGFDB1213	E	D Venturi. Générateur de vapeur [Steam generator] 79/19TE – Lower internals assembly – Right and left hand SGs. NEEG-F DB 1213 Revision E. AREVA NP - FA3 Project. April 2008
5	5.4	NEEGFDB1225	G	C Hazelard. Générateur de vapeur [Steam generator] 79/19TE – Upper internals assembly – Right and left hand SGs. NEEG-F DB 1225 Revision G. AREVA NP - FA3 Project. November 2008
5	5.4	NEEGFDB1226	F	C Hazelard. Générateur de vapeur [Steam generator] 79/19TE – Upper internals – Sections - Right and left hand SGs. NEEG-F DB 1226 Revision F. AREVA NP - FA3 Project. November 2008
5	5.4	NFPSRDC1144	A	T Millequant. Functional requirements on the Steam Generator – NFPSR DC 1144 Revision A. AREVA NP – FA3 Project. May 2006. (E)
5	5.4	NEEGFDC5	F	A Nicoli. EPR 79/19 TE – Steam Generator operating parameters. NEEG-F DC 5 Revision F. AREVA. January 2009. (E)
5	5.4	NEEGFDC35	D	S Francke. EPR - Steam Generator 79/19 TE - Primary and Secondary Sizing. NEEG-F DC 35 Revision D. AREVA NP - FA3 Project. July 2007 (E)
5	5.4	ECEMA0051005	A1	Steam Generator Cleaning – Lancing (SNL) - System Specification. ECEMA0051005 Revision A1. EDF. December 2009. (E)
5	5.4	EYTF2008FR0042	A	Dossier de Système Elémentaire – Système de lançage des générateurs de vapeurs (SNL), P4 – Schémas mécaniques détaillés. [System Design Manual - Steam Generator Cleaning – Lancing (SNL) System. P4 – Detailed flow diagrams.] EYTF/2008/FR/0042 Revision A. Sofinel. October 2008.
5	5.4	EYTF2008FR0043	A	Dossier de Système Elémentaire – Système de lançage des générateurs de vapeurs (SNL), P4.2 – Schémas mécaniques détaillés. [System Design Manual - Steam Generator Cleaning – Lancing (SNL) System. P4 – Detailed flow diagrams.] EYTF/2008/FR/0043 Revision A. Sofinel. October 2008.
5	5.4	NEEGFDC232	B	C Blanchard. EPR 79/19 TE – Tube bundle linear vibration analysis. NEEG-F DC 232 Revision B. AREVA NP – FA3 Project. May 2008. (E)
5	5.4	NFPMGDB1207	J	C Hazelard. Générateur de vapeur 79/19TE - Boîte à eau primaire – Nomenclature - GV droit et gauche. [Steam generator 79/19TE – Primary water box – Nomenclature – Steam Generator right and left] NFPMG DB 1207 Revision J. AREVA NP - FA3 Project. November 2008

5	5.4	NFPMGDB1211	G	Priol. Générateur de vapeur 79/19TE - ¨Enceinte secondaire – Nomenclature - GV droit et gauche. [Steam generator 79/19TE – Secondary side casing – Nomenclature – Steam Generator right and left] NFPMG DB 1211 Revision G. AREVA NP - FA3 Project. January 2009
5	5.4	RCCM1060		UK EPRTM - RCC-M Modification Sheet FM 1060. Introduction of 20 MND 5 steel grade. AFCEN. (E)
5	5.4	PEEGF100403	1	UK EPR – Design Change Management stage 2 – CMF017 - 20MND5 characteristics. PEEG-F 10.0403 Revision 1. AREVA. April 2010. (E)
5	5.4	PEEMF101062	C	UK EPR™- Comparison of mechanical properties of 20MND5 with those of 18MND5 and 16MND5 grades. PEEM-F 10.1062 Revision C. AREVA. June 2010. (E)
5	5.4	PEERF100134	A	Definition of the hierarchy of the main reports relating to design and manufacture primary component. PEER-F 100134 Revision A. AREVA. March 2010. (E)
5	5.4	NEEGFDC155	D	J-C Devoisin. G.V. EPR - 79/19 TE - Steam Generator Main Feedwater Nozzle -Assessment of Fatigue Damage. NEEG-F DC 155 Revision D. AREVA. August 2009. (E)
5	5.4	NEEGFDC88	D	C Canteneur. G.V. EPR - 79/19 TE - Steam Generator Emergency Feedwater Nozzle -Assessment of Fatigue Damage. NEEG-F DC 88 Revision D. AREVA. August 2008. (E)
5	5.4	NFEMGDC80	I	EPR Steam generator – Equipment specification. NFEMG DC 80 Revision I. AREVA NP – FA3 Project. June 2007. (E)
5	5.4	PEEMF101134	D	Manufacturing Non Destructive Testing to be qualified. PEEM-F.10.1134 Revision D. AREVA. December 2010. (E)
5	5.4	PEEMF110505	C	Ultrasonic examination of MCL homogeneous and dissimilar metal welds. PEEM-F 11.0505 Revision C. AREVA. March 2012. (E)
5	5.4	PEERFDC78	A	UK EPR - ALARP justification of manufacturing inspection techniques proposed for Main Coolant Line welds of the UK EPR. PEER-F DC 78 Revision A. AREVA. March 2012. (E)
5	5.4	NEEGFDC30	B	FP Le Moal. Steam Generator components – Volumes and masses. NEEG-F DC 30 Revision B. AREVA NP – FA3 project. September 2006. (E)
5	5.4	PEERFDC60	A	UK EPR Main Coolant Lines Design Basis Report (excluding surge line). PEER-F DC 60 Revision A. EDF. September 2011. (E)
5	5.4	NFPMRDB1207	G	EPR FA3 – Primary Loops – Assembly. AREVA-NP drawing. NFPMR DB 1207 Revision G. AREVA. 2009. (E)
5	5.4	PEERFDB0022	A	EPR UK – MCL Crossover lowering. AREVA-NP drawing PEER-F DB 0022 Revision A. AREVA. 2011. (E)
5	5.4	NEERFDC422	A	EPR UK – Main Coolant Lines – Dimensioning report. NEER-F DC 422 Revision A. AREVA. December 2009. (E) NEER-F DC 422 Revision A is the English translation of NEERF DC 31 Revision D.
5	5.4	NFPMRDB1208	F	EPR FA3 – Primary Loops – Pressuriser Surge Line. AREVA-NP drawing NFPMR DB 1208 Revision F. AREVA. October 2009.
5	5.4	NEERFDC9	H	EPR FA3 – Equipment Specification – MCL procurement. NEER-F DC 9 Revision H. AREVA. 2007. (E)

5	5.4	NEEMFDC13	B	EPR OL3 - Austenitic Filler Metal for Primary and Auxiliary Piping Material Data File. AREVA NP Report NEEM-F DC 13 Revision B. AREVA. 2006. (E)
5	5.4	PEERFDC73	B	UK EPR MCL Optimisation – U-leg lowering feasibility study. PEER-F DC 73 Revision B. AREVA. February 2012. (E)
5	5.4	PEERFDC79	A	UK EPR MCL optimisation – counterbore length increase feasibility study. PEER-F DC 79 Revision A. AREVA. March 2012. (E)
5	5.4	NEERFDB1225	E	C Borrel. PZR Part list. NEER-F DB 1225 Revision E. AREVA. April 2009. (E)
5	5.4	NEERFDC68	C	V. Prevert. PZR Assessment of Sensitive Areas. NEERF DC 68 Revision C. AREVA. September 2009. (E)
5	5.4	NESSFDC545	A	System Design Manual - Reactor Coolant System (RCS) - Part 4: Flow diagrams. NESS-F DC 545 Revision A. AREVA. June 2009. (E)
5	5.4	NEERFDC11	E	V. Prevert. FA3 - Pressurizer Design Finalisation. NEER-F DC 11 Revision E. AREVA NP. September 2009. (E)
5	5.4	NEERFDC18	E	J P Potonnier. Pressuriser Equipment Specification. NEER-F DC 18 Revision E. AREVA NP. July 2009. (E)
5	5.4	NESSFDC534	A	System Design Manual - Reactor Coolant System (RCS) - Part 3: System and component sizing. NESS-F DC 534 Revision A. AREVA. April 2009. (E)
5	5.4	NEERFDB1221	E	C. Borrel. Pressuriser - General Assembly. NEER-F DB 1221 Revision E. AREVA NP. February 2009. (E)
5	5.4	NEEGFDC74	C	A Chatain. Valves datasheet for pressurizer safety relief valves (PSRV). NEEG-F DC 74 Revision C. AREVA NP – FA3 Project. September 2008. (E)
5	5.4	NEEGFDC9	E	M N Farina. Datasheet for pressurizer relief tank. NEEG-F DC 9 Revision E. AREVA NP – FA3 Project. July 2008. (E)
5	5.4	ECEMA020665	B1	Engineering Rules ENG 2-49: Selection rules for valves and their actuator. ECEMA020665 Revision B1. EDF. December 2009. (E)
5	5.4	NEEGFDC39	E	F Vialard. Equipment specification for Q1 and Q2 valves. NEEG-F DC 39 Revision E. AREVA NP. September 2008. (E)
5	5.4	NESSFDC538	A	System Design Manual - Reactor Coolant System (RCS) - Part 2: System Operation. NESS-F DC 538 Revision A. AREVA. May 2009. (E)
5	5.4	NEEGFDC52	E	F Pedrak. Data sheet - Situations and loads for dedicated PZR gate valve. NEEG-F DC 52 Revision E. AREVA NP. November 2008. (E)
5	5.4	NEEGFDC55	E	F Pedrak. Data sheet - Situations and loads for dedicated PZR globe valve. NEEG-F DC 55 Revision E. AREVA NP. November 2008. (E)
5	5.4	NEERFDB1247	C	Brouttelande & Ghebbi. Reactor pressure vessel support, assembly. NEER-F DB 1247 Revision C. AREVA NP. March 2007. (E)
5	5.4	NEERFDB1248	B	Brouttelande & Ghebbi. Steam generator supports, general assembly. NEER-F DB 1248 Revision B. AREVA NP. March 2007. (E)
5	5.4	NEERFDB1258	C	Dufour & Ghebbi. Reactor coolant pump supports, general assembly. NEER-F DB 1258 Revision C. AREVA NP. September 2007. (E)

5	5.4	NEERFDB1264	C	Dufour & Ghebbi. Pressurizer supports, general assembly. NEER-F DB 1264 Revision C. AREVA NP. June 2008. (E)
SUB-CHAPTER 5.5 – REFERENCES				
5	5.5	ECECS121408	A	Ex-core Radiation Minimisation and Control in UK-EPR™ Reactor. ECECS121408 Revision A. EDF. September 2012. (E)
5	5.5	ECEF091642	B1	EPR FA3 – Report on Identifying Chemical and Radiochemical Parameters for Operating Technical Specification (Level 1 Specifications). ECEF091642 Revision B1. EDF. October 2010. (E)
5	5.5	EDECME100781	A	Identification of the 'Level 2' Chemical and Radiochemical Parameters, as Part of the RGE Rules of the Flamanville EPR. EDECME100781 Revision A. EDF. November 2010. (E)
5	5.5	PEEMFDC4	A	Reactor Chemistry Topical Report. PEEM-F DC 4 Revision A. AREVA. June 2010. (E)
5	5.5	ECEF110448	A	A Tigeras. Analysis of UK EPR™ Source Term: Identification, Quantification and Characterisation. ECEF110448 Revision A. EDF. July 2011. (E)
5	5.5	ECEF110449	A	Activity management at UK-EPR Auxiliary Systems : system performance and control actions. ECEF110449 Revision A. EDF. June 2012. (E)
5	5.5	EPRI1014986	2	Pressurized Water Reactor Primary Water Chemistry Guidelines, EPRI Report 1014986 Revision 2. EPRI. December 2007 (E)
5	5.5	NEEMFDC143	C	Reduction in Primary Circuit Radioactivity SFAIRP Based Upon the Primary Circuit Chemistry. NEEM-F DC 143 Revision C. AREVA. March 2011. (E)
5	5.5	FS10001318	1.0	Status on Crud Monitoring and Acceptability. FS1-0001318 Revision 1.0. AREVA. July 2010. (E)
5	5.5	ECEF111022	A	A Tigeras, M Bachet. Corrosion product characterization under PWR/EPR primary coolant conditions: thermodynamic assessments and power plant feedback. ECEF111022 Revision A. EDF. July 2011. (E)
5	5.5	NGPS62006en1008	G	D Gschrey. System Design Manual – Chemical and Volume Control System (RCV [CVCS]), P2 – Operation of the System. NGPS6/2006/en/1008. Revision G. AREVA. December 2008. (E)
5	5.5	ECEF110140	A	Chemistry and Radiochemistry monitoring at UK-EPR primary coolant. ECEF110140 Revision A. EDF. March 2011 (E)
5	5.5	PEEMFDC19	A	Reactor Chemistry – Zinc Assessment for EPR UK. PEEM-F DC 19 Revision A. AREVA. September 2010. (E)
5	5.5	EDECME100223	A	Zinc injection benefits and its implementation feasibility for the UK-EPR. EDECME100223 Revision A. EDF. February 2010. (E)
5	5.5	ECEF110139	A	Zinc Injection claims, arguments and evidences: overall balance for UK-EPR. ECEF110139 Revision A. EDF. March 2011 (E)
5	5.5	ECEF110138	A	Zinc Injection Implementation at UK-EPR. ECEF110138 Revision A. EDF. March 2011 (E)

5	5.5	EPRI1013522	0	F Garzarolli, and GP Sabol, Assessment of the Effect of Elevated Reactor Coolant Hydrogen on the Performance of PWR Zr-Based Alloys. EPRI report, 1013522 Revision 0. EPRI. May 2006. (E)
5	5.5	ECEF110141	A	Hydrogen risk management at UK-EPR. ECEF110141. Revision A. EDF. March 2011 (E)
5	5.5	ECEF110142	A	Shutdown and start-up primary coolant chemistry/radiochemistry strategy for UK-EPR. ECEF110142. Revision A. EDF. March 2011 (E)
5	5.5	ECEF102664	A	Activity Management Methodology Applied to UK EPR. ECEF102664 Revision A. EDF. December 2010. (E)
5	5.5	EPRI1019107	2	Fuel Integrity Monitoring and Failure Evaluation Handbook, EPRI 1019107 Revision 2. EPRI. November 2010. (E)
5	5.5	NEEMFDC140	A	Reduction in primary circuit radioactivity SFAIRP based upon the primary circuit materials. NEEM-F DC 140 Revision A. AREVA. April 2010. (E)
5	5.5	ECEF110161	A	Tritium management - provisions and limits - EPR UK. ECEF110161 Revision A. EDF. February 2011. (E)
5	5.5	SFLEZS030008	G	M Drechsler, C Hamann. System Design Manual – Coolant Purification, Degasification, Storage and Treatment System (TEP [CSTS]), P2 –System Operation, SFL-EZS-030008 Revision G. Sofinel. October 2008. (E)
5	5.5	SFLEFMF2006712	G1	System Design Manual - PTR Fuel Pool Cooling System - Part 2 System Operation, SFL–EF-MF 2006-712 Revision G1. Sofinel. August 2009. (E)
5	5.5	PEEMFDC5	A	Secondary Coolant Chemistry – Topical Report. PEEM-F DC 5 Revision A. AREVA. June 2010. (E)
5	5.5	ETDOPN100090	A1	Preliminary description of the ATD, CEX, CVI, SIT and SRI systems. E.T.DOPN/100090 Revision A1. EDF. June 2010. (E)
CHAPTER 6				
SUB-CHAPTER 6.2 – REFERENCES				
6	6.2	EPRRDC1673	C	F. Lefevre. Mass and Energy Releases in the containment in case of a Loss of Coolant Accident (LOCA). EPRR DC 1673 Revision C. Framatome ANP. December 2001. (E)
6	6.2	NGPS42003en0045	D	Dr Eyink. EPR Pressure and temperature loads relevant for the steel liner. NGPS4/2003/en/0045 Revision D. AREVA NP. July 2003. (E)
6	6.2	BDR1999CH621211		Basic Design Report. Chapter 6.2.1.2.1.1, Leaktightness requirements for PCCs and RRC-A. Report jointly prepared by EDF, German utilities, Framatome, Siemens AG, Nuclear power international. February 1999. (E)
6	6.2	BDR1999CH621323		COLIGNON. Basic Design Report. Chapter 6.2.1.3.2.3, Initial and Boundary Conditions. Report jointly prepared by EDF, German utilities, Framatome, Siemens AG, Nuclear power international. February 1999. (E)
6	6.2	BDR1999CH621333		COLIGNON. Basic Design Report. Chapter 6.2.1.3.3.3, Calculations assumptions. Report jointly prepared by EDF, German utilities, Framatome, Siemens AG, Nuclear power international. February 1999. (E)

6	6.2	NEPSG2007en0035		K-G Petzold. EPR Basic Containment Model for the "Lumped Parameter" code COCOSYS including the Combustible Gas Control System. NEPS-G/2007/en/0035. AREVA NP. December 2007. (E)
6	6.2	BDR1999CH62152		L. KRISSIAN, F. ROY. Basic Design Report. Chapter 6.2.1.5.2, List of input data for P&T calculation under Surge Line Break, 2A LOCA and SLB conditions – Containment data. Report jointly prepared by EDF, German utilities, Framatome, Siemens AG, Nuclear power international. February 1999. (E)
6	6.2	BDR1999CH62163		L. KRISSIAN, F. ROY. Basic Design Report. Chapter 6.2.1.6.3, P&T in case of Steam Line Break. Report jointly prepared by EDF, German utilities, Framatome, Siemens AG, Nuclear power international. February 1999. (E)
6	6.2	BDR1999CH6215		L. KRISSIAN, F. ROY. Basic Design Report. Chapter 6.2.1.5, List of input data and Chapter 6.2.1.6, Preliminary calculations of P&T transients. Report jointly prepared by EDF, German utilities, Framatome, Siemens AG, Nuclear power international. February 1999. (E)
6	6.2	EYTS2008FR0033	B1	System Design Manual - EDE (Annulus Ventilation) system description - Part 1 system description records. EYTS/2008/fr/0033 Revision B1. Sofinel. August 2009. (E)
6	6.2	EYTS2006fr0010	B1	System Design Manual - Annulus Ventilation System (EDE [AVS]), Part 2, System Operation (Stage 1). EYTS2006/fr/0010 Revision B1. Sofinel. August 2009. (E)
6	6.2	EYTS2007fr0110	B1	System Design Manual - Annulus Ventilation System (EDE [AVS]), Part 3, System and Component Design (Stage 1). EYTS2007/fr/0110 Revision B1. Sofinel. August 2009. (E)
6	6.2	EYTS2007fr0026	B1	System Design Manual - Annulus Ventilation System (EDE [AVS]), Part 4 – Flow diagrams. EYTS2007/fr/0026 Revision B1. Sofinel. August 2009. (E)
6	6.2	ECEF071054	A	Dossier de Système Elémentaire - EDE Ventilation espace entre enceintes, P4.1 Schéma mécanique fonctionnel. [System Design Manual - Annulus Ventilation System (EDE [AVS]), P4.1 Functional Flow Diagram] ECEF071054 Revision A. EDF. April 2008.
6	6.2	EYTS2007fr0024	B	Dossier de Système Elémentaire - EDE Ventilation espace entre enceintes, P4.2 Schéma mécanique détaillé. [System Design Manual - Annulus Ventilation System (EDE [AVS]), P4.2 Detailed Flow Diagram] EYTS2007/fr/0024 Revision B. EDF. September 2007.
6	6.2	EYTS2007fr0003	B1	System Design Manual - Annulus Ventilation System (EDE [AVS]), Part 5 – Instrumentation and Control. EYTS2007/fr/0003 Revision B1. Sofinel. August 2009. (E)
6	6.2	NFEN779		General ventilation air filters for the elimination of particles. NF EN 779. AFNOR. (E)
6	6.2	NFM62206		Nuclear energy, Nuclear ventilation installations, Control method for the purification of iodine traps. NF M 62-006. AFNOR. January 1982. (E)
6	6.2	D5710ESD1999005128011	01	Study of boron crystallisation as a function of temperature D5710/ESD/1999/005128/01-1. EDF. December 2009. (E) D5710/ESD/1999/005128/01-1 is the English translation of D5710/ESD/1999/005128/01
6	6.2	ECEIG0001089	C1	A Meziere, P. Bily. Reactor Building Specifications. ECEIG0001089 Revision C1. EDF. September 2009. (E) ECEIG0001089 Revision C1 is the English translation of ECEIG0001089 Revision C.

6	6.2	EYRC2007FR0243		Dimensions of the penetration sleeves in the inner containment wall. EYRC 2007 FR 0243. Sofinel. 2008. (E)
6	6.2	SFLEYRC0030037	G	List of penetrations through inner and outer containment walls. SFL EYRC 0030037 Revision G. Sofinel. March 2009. (E)
6	6.2	ECEIG061113	A1	A Meziere. System Design Manual - Leak Rate Control and Testing (EPP) System Specification. ECEIG061113 Revision A1. EDF. March 2009. (E)
6	6.2	ECEIG080098	B1	Layout rules for Piping and fittings used for 3D Modelling. ECEIG080098 Revision B1. EDF. Aprl 2009. (E) ECEIG080098 Revision B1 is the English translation of ECEIG080098 Revision B.
6	6.2	ECEIG061563	A	Mechanical Analysis of Piping. ECEIG061563 Revision A. EDF. October 2007. (E)
6	6.2	ECEIG021405	H1	EPR - General hypotheses note for Civil Engineering Design of the Nuclear Island buildings.. ECEIG021405 Revision H1 (TR 07/312). EDF. October 2008. (E)
6	6.2	ECEF070060	A1	Combustible Gas Control System (ETY [CGCS]): ETY System Specifications. ECEF070060 Revision A1. EDF. August 2008. (E)
6	6.2	EZS2008en0080	B	System Design Manual - Combustible Gas Control System (ETY [CGCS]), Part 2, System Operation. EZS/2008/en/0080 Revision B. Sofinel. January 2009. (E)
6	6.2	EZS2008en0081	B	System Design Manual - Combustible Gas Control System (ETY [CGCS]) Part 3, Sizing of the System and Components. EZS/2008/en/0081 Revision B. Sofinel. April 2009 (E)
6	6.2	EZS2008en0082	B	System Design Manual - Combustible Gas Control System (ETY [CGCS]). Part 4.2 Detailed Flow Diagrams. EZS/2008/en/0082 Revision B. Sofinel. January 2009. (E)
6	6.2	EZS2008en0083	A	System Design Manual - Combustible Gas Control System (ETY [CGCS]). Part 5 Instrumentation and Control. EZS/2008/en/0083 Revision A PREL. Sofinel. October 2008. (E)
6	6.2	GRSP31		Klein Hessling. COCOSYS V 1.2 User Manual, Reference Manual. GRS-P-3/1. 2000. (E)
6	6.2	NESBG2008en0037		Short Description and qualification of the computer code WAVCO. NESB_G/2008/en/0037. AREVA NP GmbH. February 2008. (E) (not applicable for EPR)
6	6.2	IAEAOECD2002		P Royl, J R Travis. Status of Development, Validation, and Application of the 3D CFD Code GASFLOW at FZK. IAEA-OECD Technical Meeting, Pisa, 11-14 November 2002. (E)
6	6.2	FZK5994		J R Travis, et al. GASFLOW: A Computational Fluid Dynamic Code; GASES, Aerosols, and Combustion, Volume 1: Theory and Computational Models, Volume 2: User's Manual, Volume 3: Assessment Manual. LA-13357-M, FZK-5994. October 1998. (E)
6	6.2	96003	0	O Braillard. Etude des performances du recombineur d´hydrogène Siemens FR90/1-150 en presence d´aspersion. [Study of the performance of hydrogen recombiner Siemens FR90/1-150 in a water spray environment.] CEA, 96/003 HO-400-5020, Indice 0.
6	6.2	OECD1996		E Studer et al. The H2PAR facility for hydrogen research. OECD/NEA/CSNI workshop on the implementation of hydrogen mitigation techniques. Winnipeg, Manitoba, Canada. May 1996. (E)

6	6.2	TR107517	1	Generic Tests of Passive Autocatalytic Recombiners (PARs) for Combustible Gas Control in Nuclear Power Plants, Volume 1 Program Description. TR-107517-V1, 4424-01. EPRI/EDF. June 1997
6	6.2	EUROSAFE2007		F Payotand, al. PHEBUS FPT3: Main relevant results likely to impact safety. EUROSAFE 2007, Berlin, November 5 and 6, 2007
6	6.2	EYTF2008FR0029	B1	System Design Manual - Containment Leak off and Seal Monitoring System (EPP) P4 – Flow Diagrams. EYTF/2008/fr/0029 Revision B1. Sofinel. September 2009. (E)
6	6.2	EYTF2008FR0030	B	Dossier de Système Elémentaire - EPP Etanchéité et Contrôle des Fuites Enceinte des Bâtiments HR- HK- HLA & HLI, P4.2 Schéma mécanique détaillé. [System Design Manual - Leak Rate Control and Testing System (EPP), P4.2 - Detailed Flow Diagrams] EYTF/2008/fr/0030 Revision B. EDF. December 2008.
6	6.2	EYTF2008FR0045	B1	System Design Manual - Containment Leak off and Seal Monitoring System (EPP) P5 – Instrumentation and Control. EYTF/2008/fr/0045 Revision B1. Sofinel. September 2009. (E)
6	6.2	USNRC10		Appendix J to Part 50 - Primary Reactor Containment Leakage Testing for Water-Cooled Power Reactors. US NRC Regulations Title 10, Code of Federal Regulations. (E)
6	6.2	NEPSG2008en1116	A	M Fischer. System CMSS: CMSS System Description. NEPS-G/2008/en/1116 Revision A. AREVA NP. November 2008. (E)
6	6.2	NEEGG2007en1037	B	A Henning, M Fischer. Equipment Data Sheet Cooling Elements of Core Catcher. NEEG-G/2007/en/1037 Revision B. AREVA NP. November 2008. (E)
6	6.2	NGPS42004en0185	A	M Nie, M Fischer. Parametric examination of the sacrificial concrete layer thickness in the spreading compartment. NGPS4/2004/en/0185 Revision A (TR04/166). AREVA NP. September 2004. (E)
6	6.2	EYTF2006FR0010	B1	System Design Manual - Containment Heat Removal System (EVU [CHRS]) P1 - Plant system file history. EYTF/2006/FR/0010 Revision B1. Sofinel. October 2009. (E)
6	6.2	SFLEFMF2006626	E1	Containment Heat Removal System (EVU [CHRS]) P2 – System operation (Stage 1) SFL–EF-MF 2006-626 Revision E1. Sofinel. October 2009. (E)
6	6.2	SFLEFMF2006694	F1	Containment Heat Removal System (EVU [CHRS]) P3 – System design (Stage 1) SFL–EF-MF 2006-694 Revision F1. Sofinel. October 2009. (E)
6	6.2	SFLEFMF2006627	C1	EVU [CHRS] Containment Heat Removal System - Plant System File - P4 Flow diagrams (Stage 1) SFL-EF-MF 2006-627 Revision C1. November 2009. (E)
6	6.2	ECEF080869	A	Dossier de Système Elémentaire - EVU Système évacuation de puissance enceinte, P 4.1 [System Design Manual - EVU Containment Heat Removal System, P4.1 flow diagrams] ECEF080869 Revision A. EDF. May 2008.
6	6.2	EYTF2006FR001	E	Dossier de Système Elémentaire - Système d'évacuation de la puissance enceinte, P4.2 Schémas mécaniques détaillés [System Design Manual - EVU Containment Heat Removal System, P4.2 Detailed flow diagrams] EYTF-2006-FR-001 Revision E. EDF. September 2008.

6	6.2	SFLEFMF2006752	D1	Containment Heat Removal System (EVU [CHRS]) - Plant System File - P5 Instrumentation and Control. SFL-EF-MF 2006-752 Revision D1. October 2009. EDF. (E)
6	6.2	ECEF061432	A	Alkalinisation of the IRWST: EVU [CHRS] soda injection. ECEF061432 Revision A. EDF. 2006. (E)
SUB-CHAPTER 6.3 – REFERENCES				
6	6.3	NESSFDC539	A	System Design Manual - Safety Injection System and Residual Heat Removal System - Part 2: System Operation. NESS-F DC 539 Revision A. AREVA. May 2009. (E)
6	6.3	NESSFDC540	A	System Design Manual - Safety Injection System and Residual Heat Removal System - Part 3: System Sizing. NESS-F DC 540 Revision A. AREVA. May 2009. (E)
6	6.3	NESSFDC546	A	System Design Manual Safety Injection and Residual Heat Removal System - Part 4: Flow diagrams. NESS-F DC 546 Revision A. AREVA. June 2009. (E)
6	6.3	NESSFDC614	A	System Design Manual - Safety Injection System - Residual Heat Removal (SIS-RHR) P5 – Instrumentation & Control. NESS-F DC 614 Revision A. AREVA. November 2009. (E)
SUB-CHAPTER 6.4 – REFERENCES				
6	6.4	ICRP60		1990 Recommendations of the International Commission on Radiological Protection. ICRP Publication 60. Ann. ICRP 21 (1-3). 1991. (E)
6	6.4	9629EURATOM	Vol39	Council directive 96/29/EURATOM of May 1996.laying down basic safety standards for the protection of health of workers and general public against the dangers arising from ionising radiation. EURATOM 96/29. May 1996. (E)
SUB-CHAPTER 6.6 – REFERENCES				
6	6.6	SFLEFMF2006829	E1	F Fioravanti, C. Gibowski. System Design Manual - Emergency Feedwater System (ASG [EFWS]) P2 – System operation SFL–EFMF 2006.829 Revision E1. Sofinel. September 2009. (E)
6	6.6	EYTF2007fr0052	C1	C. Gibowski. System Design Manual - Emergency Feedwater System (ASG [EFWS]) P3 – System and component design. EYTF/2007/fr/0052 Revision C1. Sofinel. September 2009. (E)
6	6.6	SFLEFMF2006746	D1	C Gibowski, F Fioravanti. System Design Manual - Emergency Feedwater System (EFWS), P4. – Flow diagrams. SFL-EF MF 2006.746 Revision D1. Sofinel. September 2009. (E)
6	6.6	ECEF072091	A	Dossier de Système Elémentaire - ASG (EFWS) Alimentation de secours GV, P4.1 Schéma mécanique fonctionnel. [System Design Manual - Emergency Feedwater System (EFWS), P4.1 Functional Flow Diagram] ECEF072091 Revision A. EDF. December 2007.

6	6.6	EYTF2007FR0031	E	Dossier de Système Elémentaire - ASG (EFWS) Alimentation de secours GV, P4.2 Schémas mécaniques détaillés. [System Design Manual - Emergency Feedwater System (EFWS), P4.2 Detailed Flow Diagrams] EYTF2007FR0031 Revision E. EDF. September 2008.
6	6.6	EYTF2006fr0011	F1	C Gibowski. System Design Manual - SG Emergency Feedwater System (EFWS) P5 – Instrumentation and Control. EYTF/2006/fr/0011 Revision F1. Sofinel. September 2009. (E)
SUB-CHAPTER 6.7 – REFERENCES				
6	6.7	NESSFDC535	A	System Design Manual. Extra Boration System - Part 2: System Operation. NESS-F DC 535 Revision A. AREVA. April 2009. (E)
6	6.7	NESSFDC536	A	System Design Manual. Extra Boration System - Part 3: System and component sizing. NESS-F DC 536 Revision A. AREVA. May 2009. (E)
6	6.7	NESSFDC537	A	System Design Manual. Extra Boration System - Part 4: Flow diagrams. NESS-F DC 537 Revision A. AREVA. May 2009. (E)
6	6.7	NESSFDC615	A	System Design Manual. Extra Boration System Part 5: – Instrumentation & Control. NESS-F DC 615 Revision A. AREVA. November 2009. (E)
SUB-CHAPTER 6.8 – REFERENCES				
6	6.8	NESSFDC624	A	System Design Manual – Atmospheric Steam Dump System (VDA [MSRT]) - P1. NESS-F DC 624 Revision A. AREVA. December 2009. (E)
6	6.8	NESSFDC580	A	System Design Manual - Main Steam Relief Train - Part 2: System Operation. NESS-F DC 580 Revision A. AREVA. November 2009. (E)
6	6.8	NESSFDC581	A	System Design Manual Main Steam Relief Train - Part 3: System Design NESS-F DC 581 Revision A. AREVA. November 2009. (E)
6	6.8	NESSFDC595	A	System Design Manual Main Steam Relief Train - Part 4: Flow diagrams NESS-F DC 595 Revision A. AREVA. September 2009. (E)
6	6.8	NESSFDC630	A	System Design Manual – Atmospheric Steam Dump System (VDA [MSRT]) P5 – Instrumentation & Control. NESS-F DC 630 Revision A. AREVA. December 2009. (E)
APPENDIX 6 – REFERENCES				
6	App6	BDR1999		Basic Design Report. Report jointly prepared by EdF, German utilities, Framatome, Siemens AG, Nuclear power international. February 1999. (E).
6	App6	EPDDC490	C	A Barbier. CATHARE 2 – Code synthetic qualification assessment. EPD DC 490 Revision C. AREVA. September 2007. (E)

6	App6	NGPS12004en0507	A	Topical Report - COCO containment code - Model description and validation. NGPS1/2004/en/0507 Revision A. AREVA. October 2004. (E)
6	App6	EPRXDM77	A	B Marchal. CONPATE4 – Qualification Report. EPRX DM 77 Revision A. AREVA November 1993. (E)
6	App6	EPDSDC150	C	S Ebalard. ORIGEN-S: Qualification manual. EPDS DC 150. Revision C. AREVA. November 2005. (E)
6	App6	EPTADM482	B	F Goux. THEMIS - Qualification report. EP/TA/DM.482 Revision B. AREVA. August 1995. (E)
6	App6	PSRRDC5	B	N Nicaise. PSAR Section 6.2.1.3.1 to 4 – Large Break LOCA (PCC) 2A-LOCA, Steam Line Break (PCC), 2A-SLB. PSRR DC 5 Revision B. AREVA. December 2003. (E)
CHAPTER 7				
SUB-CHAPTER 7.1 – REFERENCES				
7	7.1	ECECC100744	A	Plant I&C requirement specification. ECECC100744 Revision A. EDF. June 2010. (E)
7	7.1	NEPSFDC557	D	Classification of structure systems and components. NEPS-F DC 557 Revision D AREVA. October 2012. (E)
7	7.1	RCCEDEC2005		Design and Construction Rules for Electrical Components of Nuclear Islands. RCC-E. AFCEN Edition. December 2005. (E)
7	7.1	16626709000RPT0028	3	UKEPR GDA I&C System CAE Document. 16626-709-000-RPT-0028 Issue 03. AMEC. June 2012. (E)
7	7.1	16626709000RPT0031	2	Update of Claims-Argument-Evidences trail for satisfaction of SAPs relevant to I&C. 16626-709-000-RPT-0031 Issue 02. AMEC. June 2012. (E)
7	7.1	PEPSFDC90	C	Safety principles applied to the UK EPR I&C architecture in terms of the requirements for diversity and independence. PEPS-F DC 90 Revision C. AREVA. August 2012. (E)
7	7.1	ECP3070095	E	UK EPR GDA Project Quality Plan. ECP3070095 Revision E. EDF. August 2010. (E)
7	7.1	NLNFDC132	A	Overall I&C System Quality Plan. NLN-F DC 132 Revision A. AREVA. June 2010. (E)
7	7.1	ECECC100831	B	Architecture of instrumentation and control system UK EPR: Design principles and defence in depth. ECECC100831 Revision B. EDF. October 2012. (E)
7	7.1	ECECC010055	F1	EPR - Technical Specifications and Conditions (CSCT) for Standard Instrumentation & Control General presentation report - CCF 01. ECECC010055 Revision F1. EDF. October 2009. (E) ECECC010055 Revision F1 is the English translation of ECECC010055 Revision F.
SUB-CHAPTER 7.2 – REFERENCES				
7	7.2	ECEF091489	E	UK EPR Generic Design Assessment - Classification of I&C safety features. ECEF091489 Revision E. EDF. October 2012. (E)

7	7.2	PEPSFDC90	C	Safety principles applied to the UK EPR I&C architecture in terms of the requirements for diversity and independence. PEPS-F DC 90 Revision C. AREVA. August 2012. (E)
7	7.2	NEPSFDC557	D	Classification of structure systems and components. NEPS-F DC 557 Revision D AREVA. October 2012. (E)
7	7.2	SAP2006	1	UK Health and Safety Executive (HSE). Safety Assessment Principles for Nuclear Facilities. 2006 Edition Revision 1. January 2008. (E)
7	7.2	ECECC121458	A	Substantiation of Independence between PICS and SAS. ECECC121458 Revision A. EDF. July 2012. (E)
7	7.2	ECEF021069	E1	Sizing of SICS. ECEF021069 Revision E1. EDF. December 2010. (E)
7	7.2	ECECC111829	B	Class 1 Control and Display Facilities in the Main Control Room and the Remote Shutdown Station. ECECC111829 Revision B. EDF. August 2012. (E)
7	7.2	SY710	V6	Kristel. System specification file. Annex 8: Interlock between HMIs (MCP-SICS-PDR). SY710 Version 6.0. Siemens. March 2009. (E)
7	7.2	ECECC060019	A1	System Specification for KSC Plant System: Safety Information and Control System and layout of Main Control Room, Remote Shutdown Station and Emergency Technical Centre. ECECC060019 Revision A1. EDF. July 2010. (E)
7	7.2	ECECC070760	B1	C Botta. System Design Manual - Main Control Room (KSC [MCR]), Part 5: Control and Instrumentation System (KSC [MCR]) EPR FA3 (Stage 2). ECECC070760 Revision B1. EDF. November 2009. (E)
7	7.2	TA2062484	C	NCSS System Specification. TA-2062484 Revision C. AREVA. July 2012. (E)
7	7.2	DN2214	2.0	Analysis Network Qualification - Part 2 Plant Bus and Island Bus. DN 2.2.14 Revision 2.0. Siemens. September 2009. (E)
7	7.2	ECECC111368	B	Justification of time response end to end on Terminal Bus Plant Bus. ECECC111368 Revision B. EDF. August 2012 (E)
7	7.2	ANPG49	1	TELEPERM XS – System Overview. ANP:G-49 V1.0. AREVA. 2006. (E)
7	7.2	NLNFDC193	C	Protection System – System Description NLN-F DC 193 Revision C. AREVA. April 2012. (E)
7	7.2	NLPG2006en1007	H	RCSL - Detailed Specification. NLP-G\2006\en\1007 Revision H. AREVA. June 2009. (E)
7	7.2	PELLFDC82	C	Diversity Criteria for Sensors & Conditioning. PELL-F DC 82 Revision C. AREVA. October 2012. (E)
7	7.2	PEPRFDC83	C	Functional Analysis for Sensors' Common Cause Failure. PEPR-F DC 83 Revision C. AREVA. October 2012. (E)
7	7.2	PELAFDC3	C	Diversity Implementation plan for sensors & conditioning. PELA-F DC 3 Revision C. AREVA. October 2012. (E)
7	7.2	PELAFDC7	B	UK EPR GDA - Basis of Substantiation for the Reliability Claims for Sensors and Conditioning Modules. PELA-F DC 7 Revision B. AREVA. October 2012. (E)
7	7.2	ECECC121662	A	UKEPR Basis of Substantiation for the Reliability Claims for the PACS Modules. ECECC121662 Revision A. EDF. August 2012. (E)
7	7.2	ECECC120443	B	Diversity criteria definition for Priority Actuation Control (PAC) module. ECECC120443 Revision B. EDF. August 2012. (E)

7	7.2	ECESN120472	A	EPR UK – Diversity implementation plan for PAC modules. ECESN120472 Revision A. EDF. July 2012. (E).
7	7.2	ECECC100831	B	Architecture of instrumentation and control system UK EPR: Design principles and defence in depth. ECECC100831 Revision B. EDF. October 2012. (E)
7	7.2	ECECC111963	C	Independence of the Class 1 Protection System (PS), the Safety Automation System (SAS) and the Non-Computerised Safety System (NCSS). ECECC111963 Revision C. EDF. August 2012. (E)
7	7.2	PTLFDC3	B	Diversity Criteria Between Protection System And Safety Automation System. PTL-F DC 3 Revision B. AREVA. August 2012. (E)
7	7.2	PELLFDC11	C	Non Computerised Safety System – Diversity Criteria. PELL-F DC 11 Revision C. AREVA. August 2012. (E)
7	7.2	PELZFDC2	B	Justification of Diversity Between I&C Systems Implemented In I&C Platforms. PELZ-F DC 2 Revision B. AREVA. October 2012. (E)
7	7.2	TA2060143	C	UNICORN Project - Platform Specification. TA-2060143 Revision C. AREVA. May 2012. (E)
7	7.2	ECECC121713	A	Overall Approach to Diversity of UK EPR I&C Systems. ECECC121713 Revision A. EDF. August 2012. (E)
7	7.2	PTLFDM1	B	Methodology and Organization for Diversity Management between I&C platforms and I&C systems. PTL-F DM 1 Revision B. AREVA. June 2012. (E)
7	7.2	PELLFDC233	B	Justification of PS reliability. PELL-F DC 233 Revision B. AREVA. June 2012. (E)
7	7.2	QU018	0	Reliability Analysis SPPA-T2000/S7. QU018.Revision 0, Siemens. February 2012. (E)
7	7.2	PTLFDC5	A	Non-Computerised Safety System- Basis of Safety Case. PTL-F DC 5. Revision A. August 2012. (E)
7	7.2	TA2082935	B	Justification of Platform Reliability & Response Time on a Typical Automatic Function. TA-2082935 Revision B. AREVA TA. July 2012. (E)
7	7.2	ECEMA071141	A	Interfaces entre systèmes de contrôle commande et cellules actionneurs HTA et BT. ECEMA071141 Revision A. EDF. November 2007. Note: this document is in French and is provided for information only.
7	7.2	ECEMA080091	A	Schémas type (relais) des cellules de tableaux 10kV de la distribution électrique de la tranche EPR FA3. ECEMA080091 Revision A. EDF. October 2008. Note: this document is in French and is provided for information only.
7	7.2	RCCEDEC2005		Design and Construction Rules for Electrical Components of Nuclear Islands. RCC-E. AFCEN Edition. December 2005. (E)
7	7.2	ECECC070931	B1	E.Marotte. KC Plant System File - Level 1 Instrumentation and Control Equipment Document Part 1 - DSE History. ECECC070931 Revision B1. EDF September 2009. (E)
7	7.2	ECECC070539	B1	E.Marotte, G. Saoutieff. KC Plant System File - Part 2: KC. System operations. ECECC070539 Revision B1. EDF. September 2009. (E)
7	7.2	ECECC070902	B1	E.Marotte, G. Saoutieff. KC Plant System File – Part 3: Designing KC. system. ECECC070902 Revision B1. EDF. September 2009. (E)
7	7.2	ECECC070935	B1	E.Marotte, G. Saoutieff. KC Plant System File – Part 4: KC. system Mechanical Diagrams. ECECC070935 Revision B1. EDF. September 2009. (E)

7	7.2	ECECC070903	B1	E.Marotte, G. Saoutieff. KC Plant System File - Part 5: KC. System I&C. ECECC070903 Revision B1. EDF. September 2009. (E)
7	7.2	ENSECC090082	A1	D. Soaret. EMC (immunity) standards and requirements to be specified for electrical equipment. ENSECC090082 Revision A1. EDF. June 2009. (E). ENSECC090082 Revision A1 is the English translation of ENSECC060193 Revision A.
7	7.2	ENGSIN050312	B	ETC-F. EPR Technical Code for Fire Protection. ENGSIN050312 Revision B. EDF. August 2007. (E)
7	7.2	ECEF051003	A1	J Ferrari. Habitability of the main control room in the event of fire. ECEF051003 Revision A1. EDF. October 2008. (E)
7	7.2	80C012EPRUK00	A	R Delhomme. Test specification for control equipment. BTR 80.C.012.EPRUK.00. Revision A. EDF (SEPTEN). July 2011. (E)
7	7.2	NLFFDC14	D	System Qualification Program. NLF-F DC14 Revision D. AREVA. March 2009. (E)
7	7.2	NLZFDC3	C	I&C TXS Cabinets Qualification Program. NLZ-F DC 3 Revision C. July 2007. (E)
SUB-CHAPTER 7.3 – REFERENCES				
7	7.3	PELFDC7	A	PS (incl. RPI sw) / RCSL / SA I&C / PIPS TELEPERM XS I&C System Engineering Quality Plan. PEL-F DC 7 Revision A. AREVA. June 2012. (E)
7	7.3	PELVFDC28	A	TXS I&C Systems Verification and Validation Plan. PELV-F DC 28 Revision A. AREVA. June 2012. (E)
7	7.3	NLZFDC3	C	I&C TXS Cabinets Qualification Program. NLZ-F DC 3 Revision C. AREVA. July 2007. (E)
7	7.3	NLEFDC25	B	PS dependability program. NLE-F DC 25 Revision B. AREVA. October 2006. (E)
7	7.3	NLNFDC193	C	Protection System – System Description NLN-F DC 193 Revision C. AREVA. April 2012. (E)
7	7.3	NLNFDC89	A	J. Latour. Protection System - Functional Diagrams. NLN-F DC 89 Revision A. AREVA. December 2009. (E) NLN-F DC 89 Revision A is the English translation of NLE-F DC 44 Revision E.
7	7.3	NLEFDC33	C	Protection System – Concept for I&C failure handling. NLE-F DC 33 Revision C. AREVA. June 2007 (E)
7	7.3	NLEFDC34	E	Protection System – Concept for periodic test. NLE-F DC 34 Revision E. AREVA. March 2010. (E)
7	7.3	NLEFDC35	F	Protection System – Concept for Signal Annunciation. NLE-F DC 35 Revision F. AREVA. July 2009. (E)
7	7.3	NLEFDC42	B	Protection System – Concept for manual commands. NLE-F DC 42 Revision B. AREVA. March 2008. (E)
7	7.3	NLEFDC124	B	P. Martinet. Reactor Trip Concept. NLE-F DC 124 Revision B. AREVA. June 2008. (E).
7	7.3	NLEFDC59	C	System Design Manual - Reactor Protection System (RPR), Part 2 – System operation. NLE-F DC 59 Revision C. AREVA. June 2007. (E)
7	7.3	NLEFDC58	C	System Design Manual - Reactor Protection System (RPR), Part 3 – System sizing. NLE-F DC 58 Revision C. AREVA. April 2007. (E)
7	7.3	NLEFDC77	B	System Design Manual - Reactor Protection System (RPR), Part 4 – Flow Diagrams. NLE-F DC 77 Revision B. AREVA. March 2007. (E)

7	7.3	NLEFDC66	E	System Design Manual - Reactor Protection System (RPR), Part 5 – Instrumentation and Control. NLE-F DC 66 Revision E. AREVA. July 2008. (E)
7	7.3	NLEFDM10014	C	TELEPERM XS Engineering Procedure Calculation of Response Time and Accuracy of TELEPERM XS channels. NLE-F DM 10014 Revision C. AREVA. September 2010. (E)
7	7.3	NLEFDM10032	A	TELEPERM XS Engineering Procedure - Methodology for RAMS Studies. NLE-F DM 10032 Revision A. AREVA. June 2010. (E)
7	7.3	PELAFDC7	B	UK EPR GDA - Basis of Substantiation for the Reliability Claims for Sensors and Conditioning Modules. PELA-F DC 7 Revision B. AREVA. October 2012. (E)
7	7.3	IEC61513		Massini. IEC 61513 ed. 2001 §6.1.1 Mapping to FA3 PS documentation. AREVA. June 2012. (E)
7	7.3	ENSECC080054	A1	Analysis of the digital CCF within systems supporting F1A safety-class functions (PS) in the instrumentation & control architecture of the FA3 EPR. ENSECC080054 Revision A1. EDF. August 2009. (E)
7	7.3	ECECC111963	C	Independence of the Class 1 Protection System (PS), the Safety Automation System (SAS) and the Non-Computerised Safety System (NCSS). ECECC111963 Revision C. EDF. August 2012. (E)
7	7.3	ECECC121713	A	Overall Approach to Diversity of UK EPR I&C Systems. ECECC121713 Revision A. EDF. August 2012. (E)
7	7.3	PELLFDC233	B	Justification of PS reliability. PELL-F DC 233 Revision B. AREVA. June 2012. (E)
7	7.3	NLTCG2008en0079	B	TXS Self-monitoring and fail safe behaviour. NLTC-G/2008/en/0079 Revision B. AREVA. November 2008. (E)
7	7.3	ECECC120489	A	Protection System Operator Terminal Basis of Safety Case. ECECC120489 Revision A. EDF. May 2012. (E)
7	7.3	ECECC120711	A	Protection System Operator Terminal (PSOT) – Functional Scope. ECECC120711 Revision A. EDF. July 2012. (E)
7	7.3	PTLFDC3	B	Diversity Criteria Between Protection System And Safety Automation System. PTL-F DC 3 Revision B. AREVA. August 2012. (E)
7	7.3	PTLFDM1	B	Methodology and Organization for Diversity Management between I&C platforms and I&C systems. PTL-F DM 1 Revision B. AREVA. June 2012. (E)
7	7.3	NLEFDC249	E	TELEPERM XS based systems. Concept for Electrical Separation. NLE-F DC 249 Revision E. AREVA. January 2011. (E)
7	7.3	PELLFDC252	A	Analysis of the non disturbance of the Protection System by lower classified signals coming for systems in interface. PELL-F DC 252 Revision A. AREVA. April 2012. (E)
7	7.3	ECECC060019	A1	System Specification for KSC Plant System: Safety Information and Control System and layout of Main Control Room, Remote Shutdown Station and Emergency Technical Centre. ECECC060019 Revision A1. EDF. July 2010. (E)
7	7.3	ECEF021069	E1	Sizing of SICS. ECEF021069 Revision E1. EDF. December 2010. (E)
7	7.3	ECECC070760	B1	C Botta. System Design Manual - Main Control Room (KSC [MCR]), Part 5: Control and Instrumentation System (KSC [MCR]) EPR FA3 (Stage 2). ECECC070760 Revision B1. EDF. November 2009. (E)

7	7.3	SY710	6	Kristel. System specification file (DSS). SY710 Version 6.0. Siemens. March 2009. (E)
7	7.3	NLNFDC91	A	J. Latour. Overall Architecture Drawing. NLN-F DC 91 Revision A. Areva. December 2009. (E) NLN-F DC 91 Revision A is the English translation of NLF-F DC 10 Revision E.
7	7.3	ECECC111058	B	Generic Rules for the Electrical Isolation of EPR UK I&C Systems (internal connections and interfaces). ECECC111058 Revision B. EDF. June 2012. (E)
SUB-CHAPTER 7.4 – REFERENCES				
7	7.4	NLFFDC11	C	Project Development Plan relating to FA3 Standard I&C. NLF-F-DC 11 Revision C. AREVA. June 2007. (E)
7	7.4	NLFFDC82	C	Quality Plan for engineering of FA3 standard I&C based on SPPA-T2000. NLF-F DC 82 Revision C. AREVA. April 2008. (E)
7	7.4	NLFFDC14	D	System Qualification Program. NLF-F DC14 Revision D. AREVA. March 2009. (E)
7	7.4	SY710	6	System specification file (DSS). SY710 Version 6.0. Siemens. March 2009. (E)
7	7.4	NLNFDC91	A	Overall Architecture Drawing. NLN-F DC 91 Revision A PREL. AREVA. December 2009. (E) NLN-F DC 91 Revision A is the English translation of NLF-F DC 10 Revision E.
7	7.4	NLFFDC89	C	Application Software Test program. NLF-F-DC 89 Revision C. AREVA. December 2009. (E)
7	7.4	QU017	0.2	Definition of the predictability model of SPPA-T2000 /S7. SIE QU017 Revision 0.2. Siemens. May 2012. (E)
7	7.4	HP1A200801857FR	3	DCE Project: Analysis of Common Cause Failures in the Instrumentation and Control System Based on the SPPA-T2000 Platform that Perform F1B (SAS) Safety Class Functions. H-P1A-2008-01857-FR Revision 3.0. EDF. August 2009. (E)
7	7.4	ECECC070931	B1	KC Plant System File - Level 1 Instrumentation and Control Equipment Document Part 1 - DSE History. ECECC070931 Revision B1. EDF September 2009. (E)
7	7.4	ECECC070539	B1	KC Plant System File - Part 2: KC. System operations. ECECC070539 Revision B1. EDF. September 2009. (E)
7	7.4	ECECC070902	B1	KC Plant System File – Part 3: Designing KC. system. ECECC070902 Revision B1. EDF. September 2009. (E)
7	7.4	ECECC070935	B1	KC Plant System File – Part 4: KC. system Mechanical Diagrams. ECECC070935 Revision B1. EDF. September 2009. (E)
7	7.4	ECECC070903	B1	KC Plant System File - Part 5: KC. System I&C. ECECC070903 Revision B1. EDF. September 2009. (E)
7	7.4	QU019	0	Module dependability analysis for SPPA-T2000 (S7) AS620B/SPPA-T2000 - OM components/Safety parameter determination approach. QU019. Revision 0. Siemens. February 2012 (E)
7	7.4	ECECC121435	A	UKEPR: SAS IEC 61513 System Requirements Specification (SRS) Equivalence. ECECC121435 Revision A. EDF. August 2012. (E)

7	7.4	ECECC080586	B1	Justification Report on the Independence of I&C Systems based on the SPPA T2000 Platform. ECECC080586 Revision B1. EDF. July 2009. (E) ECECC080586 Revision B1 is the English translation of ECECC080586 Revision B.
7	7.4	QU003	0.1	Self Test Coverage Analysis. QU003 Revision 0.1. Siemens. February 2012. (E)
7	7.4	ECECC111058	B	Generic Rules for the Electrical Isolation of EPR UK I&C Systems (internal connections and interfaces). ECECC111058 Revision B. EDF. June 2012. (E)
7	7.4	PELFDC13	A	Basis of Safety Case of SPPA-T2000. PEL-F DC 13 Revision A. AREVA. June 2012. (E)
7	7.4	ANPG49	1	TELEPERM XS – System Overview. ANP:G-49 V1.0. AREVA. 2006. (E)
7	7.4	PELFDC7	A	PS (incl. RPI sw) / RCSL / SA I&C / PIPS TELEPERM XS I&C System Engineering Quality Plan. PEL-F DC 7 Revision A. AREVA. June 2012. (E)
7	7.4	PELVFDC28	A	TXS I&C Systems Verification and Validation Plan. PELV-F DC 28 Revision A. AREVA. June 2012. (E)
7	7.4	NLZFDC3	C	I&C TXS Cabinets Qualification Program. NLZ-F DC 3 Revision C. AREVA. July 2007. (E)
7	7.4	NLPG2006en1007	H	RCSL - Detailed Specification. NLP-G\2006\en\1007 Revision H. AREVA. June 2009. (E)
7	7.4	NLEFDC179	B	RCSL – Concept for Periodic Tests. NLE-F DC 179 Revision B. AREVA. May 2009. (E)
7	7.4	NLEFDC166	B	RCSL – Concept for Signal Annunciation. NLE-F DC 166 Revision B. AREVA. August 2009. (E)
7	7.4	NLNFDC90	A	RCSL - Functional Diagrams. NLN-F DC 90 Revision A. AREVA. December 2009. (E) NLN-F DC 90 Revision A PREL is the English translation of NLE-F DC 167 Revision D.
7	7.4	NLEFDM10014	C	TELEPERM XS Engineering Procedure - Calculation of Response Time and Accuracy of TELEPERM XS channels. NLE-F DM 10014 Revision C. AREVA. September 2010. (E)
7	7.4	NLEFDM10032	A	TELEPERM XS Engineering Procedure - Methodology for RAMS Studies. NLE-F DM 10032 Revision A. AREVA. June 2010. (E)
7	7.4	NLEFDC249	E	TELEPERM XS based systems. Concept for Electrical Separation. NLE-F DC 249 Revision E. AREVA. January 2011. (E)
7	7.4	ECEF100659	A	Principles to be used for the implementation of the NCSS for Emergency Operating Procedures. ECEF100659 Revision A. EDF. March 2010. (E)
7	7.4	TA2062484	C	NCSS System Specification. TA-2062484 Revision C. AREVA. July 2012. (E)
7	7.4	NEPRFDC551	C	EPR UK Functional Requirements on Non-Computerised Safety I&C Functions. NEPR-F DC 551 Revision C. AREVA. July 2012 (E)
7	7.4	NEPSFDC555	D	Safety Requirements for Non-Computerised Safety System (NCSS). NEPS-F DC 555 Revision D. June 2012. (E)
7	7.4	PELLFDC11	C	Non Computerised Safety System – Diversity Criteria. PELL-F DC 11 Revision C. AREVA. August 2012. (E)
7	7.4	TA2065953	C	NCSS System Verification and Validation Plan. TA-2065953 Revision C. AREVA. July 2012. (E)
7	7.4	TA2061589	C	NCSS Quality Plan. TA-2061589 Revision C. AREVA. July 2012. (E)

7	7.4	TA2057230	D	UNICORN Project - Platform Quality Plan. TA-2057230 Revision D. AREVA. June 2012. (E)
7	7.4	TA2073805	D	UNICORN Project - Platform Qualification Plan. TA-2073805 Revision D. AREVA. July 2012. (E)
7	7.4	TA2082935	B	UNICORN Project - Justification of Platform Reliability & Response Time on a Typical Automatic Function. TA-2082935 Revision B. AREVA. July 2012. (E)
7	7.4	TA2060143	C	UNICORN Project - Platform Specification. TA-2060143 Revision C. AREVA. May 2012. (E)
7	7.4	PTLFDC5	A	Non-Computerised Safety System- Basis of Safety Case. PTL-F DC 5. Revision A. August 2012. (E)
7	7.4	PTIDC2	E	Requirements for non-computerized I&C platform. PTI DC 2. Revision E. AREVA. April 2012. (E)
7	7.4	PEPRFDC105	A	EPR UK – Functional Justification of the Non Computerised Safety System Design. PEPR-F DC 105 Revision A. AREVA. July 2012. (E)
SUB-CHAPTER 7.5 – REFERENCES				
7	7.5	NLFFDC11	C	Project Development Plan relating to FA3 Standard I&C. NLF-F-DC 11 Revision C. AREVA. June 2007. (E)
7	7.5	NLFFDC82	C	Quality plan for engineering of FA3 standard I&C based on SPPA-T2000. NLF-F DC 82 Revision C. AREVA. April 2008. (E)
7	7.5	ECECC010055	F1	EPR - Technical Specifications and Conditions (CSCT) for Standard Instrumentation & Control General presentation report - CCF 01 and associated documents CCF02 to CCF17, ECECC010055 Revision F1. EDF. October 2009. (E) ECECC010055 Revision F1 is the English translation of ECECC010055 Revision F.
7	7.5	NLFFDC14	D	System Qualification Program. NLF-F DC 14 Revision D. AREVA. March 2009. (E)
7	7.5	SY710	6	Kristel. System specification file (DSS). SY710 Version 6.0. Siemens. March 2009. (E)
7	7.5	NLNFDC91	A	J. Latour. Overall Architecture Drawing. NLN-F DC 91 Revision A PREL. AREVA. December 2009. (E) NLN-F DC 91 Revision A is the English translation of NLF-F DC 10 Revision E.
7	7.5	NLFFDC89	C	Application Software Test program. NLF-F-DC 89 Revision C. AREVA. December 2009. (E)
7	7.5	ECECC121435	A	UKEPR: SAS IEC 61513 System Requirements Specification (SRS) Equivalence. ECECC121435 Revision A. EDF. August 2012. (E)
7	7.5	ECECC121609	A	SRS Equivalence Justification Note for PAS and PACS. ECECC121609 Revision A. EDF. August 2010. (E)
7	7.5	PELFDC13	A	Basis of Safety Case of SPPA-T2000. PEL-F DC 13 Revision A. AREVA. June 2012. (E)
7	7.5	ECECC040729	A1	Specification for the KIC [PICS] Plant System: Instrumentation and Control Systems for Computerised Control. ECECC040729 Revision A1. EDF. June 2010. (E)
7	7.5	ECECC080097	B1	Process Information and Control System (KIC [PICS]) Part 5: Control and Instrumentation System EPR FA3 (Stage 2). ECECC080097 Revision B1. EDF. June 2010. (E)
7	7.5	RCCEDEC2005		Design and Construction Rules for Electrical Components of Nuclear Islands. RCC-E. AFCEN Edition. December 2005. (E)

7	7.5	ECECC111368	B	Justification of time response end to end on Terminal Bus Plant Bus. ECECC111368 Revision B. EDF. August 2012 (E)
7	7.5	ECECC060019	A1	System Specification for KSC Plant System: Safety Information and Control System and layout of Main Control Room, Remote Shutdown Station and Emergency Technical Centre. ECECC060019 Revision A1. EDF. July 2010. (E)
7	7.5	ECECC070760	B1	C Botta. System Design Manual - Main Control Room (KSC [MCR]), Part 5: Control and Instrumentation System (KSC [MCR]) EPR FA3 (Stage 2). ECECC070760 Revision B1. EDF. November 2009. (E)
7	7.5	ECECC070931	B1	E.Marotte. KC Plant System File - Level 1 Instrumentation and Control Equipment Document Part 1 - DSE History. ECECC070931 Revision B1. EDF September 2009. (E)
7	7.5	ECECC070539	B1	E.Marotte, G. Saoutieff. KC Plant System DES File - Part 2: KC. System operations. ECECC070539 Revision B1. EDF. September 2009. (E)
7	7.5	ECECC070902	B1	E.Marotte, G. Saoutieff. KC Plant System File – Part 3: Designing KC. system. ECECC070902 Revision B1. EDF. September 2009. (E)
7	7.5	ECECC070935	B1	E.Marotte, G. Saoutieff. KC Plant System File – Part 4: KC. system Mechanical Diagrams. ECECC070935 Revision B1. EDF. September 2009. (E)
7	7.5	ECECC070903	B1	E.Marotte, G. Saoutieff. KC Plant System File - Part 5: KC. System I&C. ECECC070903 Revision B1. EDF. September 2009. (E)
7	7.5	ANPG49	1	TELEPERM XS – System Overview. ANP:G-49 V1.0. AREVA. 2006. (E)
7	7.5	PELVFDC28	A	TXS I&C Systems Verification and Validation Plan. PELV-F DC 28 Revision A. AREVA. June 2012. (E)
7	7.5	NLEFDC106	C	SA I&C - Detail Specification File. NLE-F DC 106 Revision C. AREVA. August 2009. (E)
7	7.5	NLEFDC169	C	SA I&C - Functional Diagrams. NLE-F DC 169 Revision C. AREVA. April 2009
7	7.5	NLEFDM10014	C	TELEPERM XS Engineering Procedure Calculation of Response Time and Accuracy of TELEPERM XS channels. NLE-F DM 10014 Revision C. AREVA. September 2010. (E)
7	7.5	NLEFDM10032	A	TELEPERM XS Engineering Procedure - Methodology for RAMS Studies. NLE-F DM 10032 Revision A. AREVA. June 2010. (E)
7	7.5	PELFDC7	A	PS (incl. RPI sw) / RCSL / SA I&C / PIPS TELEPERM XS I&C System Engineering Quality Plan. PEL-F DC 7 Revision A. AREVA. June 2012. (E)
7	7.5	NLEFDC249	E	TELEPERM XS based systems. Concept for Electrical Separation. NLE-F DC 249 Revision E. AREVA. January 2011. (E)
SUB-CHAPTER 7.6 – REFERENCES				
7	7.6	RCCEDEC2005		Design and Construction Rules for Electrical Components of Nuclear Islands. RCC-E. AFCEN Edition. December 2005. (E)

7	7.6	ENSECC110102	B	Justification of smart devices for nuclear safety applications. ENSECC110102 Revision B. EDF. May 2012. (E)
7	7.6	ENSECC110106	B	Lifecycle approach to qualify Smart Devices used in nuclear safety applications. ENSECC110106 Revision B. EDF. March 2012. (E)
7	7.6	PELLFDC82	C	Diversity Criteria for Sensors & Conditioning. PELL-F DC 82 Revision C. AREVA. October 2012. (E)
7	7.6	PELAFDC3	C	Diversity Implementation plan for sensors & conditioning. PELA-F DC 3 Revision C. AREVA. October 2012. (E)
7	7.6	PELAFDC7	B	UK EPR GDA - Basis of Substantiation for the Reliability Claims for Sensors and Conditioning Modules. PELA-F DC 7 Revision B. AREVA. October 2012. (E)
7	7.6	NLEFDC11	E	Fixed Core Outlet Thermocouple Instrumentation System Specification. NLE-F DC 11 Revision E. AREVA. January 2008. (E)
7	7.6	NLEFDC15	F	Self Powered Neutron Flux Detector Instrumentation SPND System Specification. NLE-F DC 15 Revision F. AREVA. December 2009. (E)
7	7.6	NLLNG2007en1020	D	Reactor Pressure Vessel Dome Temperature Measurement System Specification. NLLN-G/2007/en/1020 Revision D. AREVA. November 2008. (E)
7	7.6	NLLNG2008en1087	A	System Design Manual – In-core Instrumentation System (RIC), P1. NLLN-G/2008/en/1087 Revision A. AREVA. July 2008. (E)
7	7.6	NLLNG2008en1088	A	System Design Manual – In-core Instrumentation System (RIC) P2 - System Operation. NLLN-G/2008/en/1088 Revision A. AREVA. August 2008. (E)
7	7.6	NLLNG2008en1089	A	System Design Manual – In-core Instrumentation System (RIC) P3 – System sizing. NLLN-G/2008/en/1089 Revision A. AREVA. August 2008.(E)
7	7.6	NLLNG2008en1090	A	System Design Manual – In-core Instrumentation System (RIC) P4 – Flow Diagrams. NLLN-G/2008/en/1090 Revision A. AREVA. August 2008. (E)
7	7.6	NLLNG2008en1091	A	System Design Manual – In-core Instrumentation System (RIC) P5 – Instrumentation and Control. NLLN-G/2008/en/1091 Revision A. AREVA. August 2008. (E)
7	7.6	NLEFDC13	G	Reactor Pressure Vessel Level Measurement System Specification. NLE-F DC 13 Revision G. AREVA. February 2008. (E).
7	7.6	NLLNG2008en1006	A	System Quality Plan for RIC/RPN Core Instrumentation. NLLN G/2008/en/1006 Revision A. AREVA. February 2010. (E)
7	7.6	NLLNG2008en1092	A	System Design Manual – Nuclear Instrumentation System (RPN) P1. NLLN-G/2008/en/1092 Revision A. AREVA. July 2008. (E)
7	7.6	NLLNG2008en1093	A	System Design Manual – Nuclear Instrumentation System (RPN), P2 - System Operation. NLLN-G/2008/en/1093 Revision A. AREVA. July 2008. (E)
7	7.6	NLLNG2008en1094	A	System Design Manual – Nuclear Instrumentation System (RPN), P3 – System sizing. NLLN-G/2008/en/1094 Revision A. AREVA. July 2008. (E)

7	7.6	NLLNG2008en1095	A	System Design Manual – Nuclear Instrumentation System (RPN), P4 – Flow Diagrams. NLLN-G/2008/en/1095 Revision A. AREVA. July 2008. (E)
7	7.6	NLLNG2008en1096	A	System Design Manual – Nuclear Instrumentation System (RPN) P5 – Instrumentation and Control. NLLN-G/2008/en/1096 Revision A. AREVA. July 2008. (E)
7	7.6	NLEFDC14	F	Ex-core Instrumentation System Specification. NLE-F DC 14 Revision F. AREVA. November 2008. (E)
7	7.6	IEC61502		IEC 61502 Standard - Nuclear power plants - Pressurised water reactors - Vibration monitoring of internal structures. IEC 61502. November 1999. (E)
7	7.6	HP1D05045	A1	R Chevalier. Additional instrumentation needs for monitoring the EPR electrical rotating machines. HP-1D/05/045 Revision A1. EDF. October 2005. (E) HP-1D/05/045 Revision A1 is the English translation of HP-1D/05/045 Revision A
7	7.6	HP1D200702400FR	1	J C Benas. Design requirements for the French EPR Loose Part Monitoring System (LPMS). H-P1D-2007-02400-FR Issue 1 EDF / R&D. November 2007. (E)
SUB-CHAPTER 7.7 – REFERENCES				
7	7.7	ANPG49	1	TELEPERM XS – System Overview. ANP:G-49 V1.0. AREVA. 2006. (E)
7	7.7	TXS100876	4.1	TELEPERM XS - System Data TXS-1008-76 V4.1. AREVA. January 2012. (E)
7	7.7	EMF2110NPA	1	TELEPERM XS - A Digital Reactor Protection System. EMF-2110(NP)(A) v1.0. AREVA. July 2000. (E)
7	7.7	NGLT2004en0023	A	S. Richter. Architecture of TXS Online Software. NGLT/2004/en/0023 Revision A. AREVA. October 2004. (E)
7	7.7	PTLDG2010en0355	A	Operation principles and safety features of the TXS system platform Release 3.5.x PTLD-G/2010/en/0355 Revision A. AREVA. December 2010 (E)
7	7.7	NGLP2004en0094	A	J-U Wittig. TELEPERM XS Simulation - Concept of Validation and Verification. NGLP/2004/en/0094 Revision A. AREVA. July 2004. (E)
7	7.7	ISTecA1085	0	A Lindner. Assessment of application of tools for TELEPERM XS. ISTec-A-1085 Revision 0. ISTec. June 2006. (E)
7	7.7	NLTCG2007en0072	A	C Hessler. Overview of approach for TXS hardware qualification. NLTC-G/2007/en/0072 Revision A. AREVA. November 2007. (E)
7	7.7	PTLCG2011en0059	A	TXS Self-monitoring and Fail-safe behaviour from Core-Software Release 3.6.2. PTLC-G/2011/en/0059 Revision A. AREVA. January 2012. (E)
7	7.7	PELFDC7	A	PS (incl. RPI sw) / RCSL / SA I&C / PIPS TELEPERM XS I&C System Engineering Quality Plan. PEL-F DC 7 Revision A. AREVA. June 2012. (E)
7	7.7	IEC61513		IEC 61513 ed. 2001 §6.1.1 Mapping to FA3 PS documentation. AREVA. June 2012. (E)
7	7.7	ECECC120489	A	Protection System Operator Terminal Basis of Safety Case. ECECC120489 Revision A. EDF. May 2012. (E)
7	7.7	NLSFDC10145	C	Design Principles for QDS Application. NLS-F DC 10145 Revision C. AREVA. May 2012 (E)
7	7.7	NFLSDC215	H	QDS Design Tool Software User Manual. NFLS DC 215 Revision H. AREVA. October 2012 (E)

7	7.7	PELVFDC28	A	TXS I&C Systems Verification and Validation Plan. PELV-F DC 28 Revision A. AREVA. June 2012. (E)
7	7.7	FAWTXS11	A	R. Fehn. Phase model for the Development of Software Components for TELEPERM XS, Engineering Procedure. FAW TXS-1.1 Revision A. AREVA. October 2006. (E)
7	7.7	FAWTXS16	A	Dr. B. Schnitzer. Software Verification and Validation Plan (V&V plan), Engineering Procedure. FAW TXS-1.6. AREVA. March 2006. (E)
7	7.7	PELLFDC233	B	Justification of PS reliability. PELL-F DC 233 Revision B. AREVA. June 2012. (E)
7	7.7	SY710	6	Kristel. System specification file. SY710 Version 6.0. Siemens. March 2009. (E)
7	7.7	DE410	3	Engineering tools detail specification. DE 410 Version 3.0. Siemens. November 2007.(E)
7	7.7	DE403	9	Detailed software specification - Engineering tools. DE 403 Version 9.0. Siemens. November 2008. (E)
7	7.7	80C012EPRUK00	A	Test specification for control equipment. 80.C.012 EPRUK 00 Revision A. EDF. July 2011. (E)
7	7.7	NLFFDC14	D	System Qualification Program. NLF-F DC14 Revision D. AREVA. March 2009. (E)
7	7.7	PELFDC13	A	Basis of Safety Case of SPPA-T2000. PEL-F DC 13 Revision A. AREVA. June 2012. (E)
7	7.7	QU003	0.1	Self Test Coverage Analysis. QU003 Revision 0.1. Siemens. February 2012. (E)
7	7.7	NLFFDC82	C	Quality plan for engineering of FA3 standard I&C based on SPPA-T2000. NLF-F DC 82 Revision C. AREVA. April 2008. (E)
7	7.7	ECECC121435	A	UKEPR: SAS IEC 61513 System Requirements Specification (SRS) Equivalence. ECECC121435 Revision A. EDF. August 2012. (E)
7	7.7	QU018	0	Reliability Analysis SPPA-T2000/S7. QU018.Revision 0, Siemens. February 2012. (E)
7	7.7	QU019	0	Module dependability analysis for SPPA-T2000 (S7) AS620B/SPPA-T2000 - OM components/Safety parameter determination approach. QU019. Revision 0. Siemens. February 2012. (E)
7	7.7	NLFFDC89	C	Application Software Test program. NLF-F-DC 89 Revision C. AREVA. December 2009. (E)
7	7.7	TA2057230	D	Platform Quality Plan. TA-2057230 Revision D. AREVA TA. June 2012. (E)
7	7.7	PTLFDC5	A	Non-Computerised Safety System- Basis of Safety Case. PTL-F DC 5. Revision A. AREVA August 2012. (E)
7	7.7	TA2073805	D	Platform Qualification Plan. TA-2073805 Revision D. AREVA TA. July 2012. (E)
7	7.7	PTIDC2	E	Requirements for Non-Computerized I&C Platform. PTI DC 2. Revision E. AREVA. April 2012. (E)
7	7.7	PELLFDC11	C	Non Computerised Safety System – Diversity Criteria. PELL-F DC 11 Revision C. AREVA. August 2012. (E)
7	7.7	PTIDC7	B	NCSS Characterization for platform sizing evaluation. PTI DC 7 Revision B. AREVA. August 2012 (E)
7	7.7	TA2060143	C	Platform Specification. TA-2060143 Revision C. AREVA TA. May 2012. (E)
7	7.7	TA2084059	A	UNICORN Project - Module Common Requirements Specification. TA-2084059 Revision A. AREVA TA. May 2012. (E)
7	7.7	TA2080785	A	SCAT Module Specification NTA-228830. TA-2080785 Revision A. AREVA TA. May 2012 (E)

7	7.7	TA2080787	A	VOPER Module Specification NTA-228831. TA-2080787 Revision A. AREVA TA. May 2012 (E)
7	7.7	TA2080788	A	AVACT Module Specification NTA-228835. TA-2080788 Revision A. AREVA TA. June 2012 (E)
7	7.7	TA2082935	B	Justification of Platform Reliability and Response Time on a Typical Automatic Function. TA-2082935 Revision B. AREVA TA. July 2012 (E)
7	7.7	TA2096900	A	Justification of Reliability Allocation. TA-2096900 Revision A. AREVA TA. June 2012 (E)
7	7.7	ECECC111134	C	UK EPR Guideline for Application of Production Excellence and Independent Confidence Building. ECECC111134 Revision C. EDF. July 2012. (E)
7	7.7	NEPRFDC551	C	Functional Requirements on Non-Computerised Safety I&C Functions. NEPR-F DC 551 Revision C. AREVA. July 2012 (E)
7	7.7	NEPSFDC555	D	Safety Requirements for Non-Computerised Safety System (NCSS). NEPS-F DC 555 Revision D. June 2012. (E)
7	7.7	TA2061589	C	NCSS Quality Plan. TA-2061589 Revision C. AREVA TA. July 2012. (E)
7	7.7	TA2065953	C	NCSS System Verification and Validation Plan. TA-2065953 Revision C. AREVA. July 2012. (E)
7	7.7	TA2062484	C	NCSS System Specification. TA-2062484 Revision C. AREVA. July 2012. (E)
7	7.7	ECECC111557	B	Justification for Production Excellence and Independent Confidence Building Measures used for Teleperm XS Based Systems. ECECC111557 Revision B. EDF. July 2012. (E)
7	7.7	ENSECC110173	B	UK EPR Protection System - Overall scope of Independent Confidence Building Measures. ENSECC110173 Revision B. EDF. June 2012. (E)
7	7.7	ECECC120398	B	UK EPR Computer Systems Safety Production Excellence Independent Confidence Building SPPA-T2000. ECECC120398 Revision B. EDF. August 2012. (E)
7	7.7	PTLCG2010en0047	B	Compliance of the TXS system platform and development processes with IEC 61513. PTLC-G/2010/en/0047 Revision B. AREVA. June 2012. (E)
7	7.7	PTLCG2010en0383	A	TXS platform: Compliance Analysis IEC 60880 ed 2.0. PTLD-G/2010/en/0383 Revision A. AREVA. December 2011. (E)
7	7.7	NLTCG2008en0053	A	Compliance of the TXS Hardware design and Engineering process with IEC60987 Ed.2 – Platform Part. NLTC-G/2008/en/0053 Revision A. AREVA. July 2008. (E)
7	7.7	NGLT2003en0045	D	Dr. Heinz-Wilhelm Bock. W. Dreeves, S. Richter. Operation Principles and Safety Features of the TELEPERM XS System Platform. NGLT/2003/en/0045 Revision D. AREVA. January 2004. (E)
7	7.7	PTLDG2010en0191	A	Field Failure Rate Calculation and Statistics of TELEPERM XS. PTLD-G/2010/en/0191 Revision A. AREVA. July 2010. (E)
7	7.7	PELAFDC7	B	UK EPR GDA - Basis of Substantiation for the Reliability Claims for Sensors and Conditioning Modules. PELA-F DC 7 Revision B. AREVA. October 2012. (E)
7	7.7	NLTDG2008de0152	B	TXS Test Report Module Test: Runtime Environment 2.7.2. NLTD-G/2008/de/0152 Revision B. AREVA. November 2009. (De)

7	7.7	TXS-AUST-0599-02		Technical Report on the assessment of the plant independent system test of the digital safety instrumentation and control system TELEPERM XS. TXS-AUST-0599-02. ISTec GmbH and TÜV Nord. May 1999 (E)
7	7.7	PELFDC8	A	TELEPERM XS I&C System Compliance Analysis With IEC 61513. PEL-F DC 8 Revision A. AREVA. June 2012. (E)
7	7.7	PELFDC9	A	TELEPERM XS I&C Systems Compliance Analysis with IEC 60880. PEL-F DC 9 Revision A. AREVA. June 2012. (E)
7	7.7	PELFDC10	A	TELEPERM XS I&C Systems Compliance Analysis with IEC 60987. PEL-F DC 10 Revision A. AREVA. June 2012. (E)
7	7.7	ENSECC110123	B	UK EPR Protection System - Scope and programme of work to address functional static analysis and compiler validation. ENSECC110123 Revision B. EDF. June 2012. (E)
7	7.7	ECECC111521	B	UK EPR - Programme of statistical testing activities. ECECC111521 Revision B. EDF. June 2012. (E)
7	7.7	QU042	0.1	SPPA-T2000 IEC 62138 Justification for AS620B. QU042. Revision 0.1 Siemens. April 2012 (E)
7	7.7	QU041	0.2	SPPA-T2000 IEC justification for OM690. QU041. Revision 0.2. Siemens. April 2012 (E)
7	7.7	DN2224	3.0	IEC 61513 and 62138 justification for SAS. DN 2.2.24 Issue 3.0. Siemens. January 2010. (E)
7	7.7	ENSECC110106	B	Lifecycle approach to qualify Smart Devices used in nuclear safety applications. ENSECC110106 Revision B. EDF. March 2012. (E)
7	7.7	ENSECC110102	B	Justification of smart devices for nuclear safety applications. ENSECC110102 Revision B. EDF. May 2012. (E)
7	7.7	ENSECC110110	B	EMPHASIS tool evaluation. ENSECC110110 Revision B. EDF. March 2012. (E)
7	7.7	ECECC121333	A	Assessment Plan for Class1 Smart Device Trial. ECECC121333 Revision A. EDF. August 2012. (E)
7	7.7	ECECC121334	A	Standard Temperature Transmitter - Requirements Identification File. ECECC121334 Revision A. EDF. July 2012. (E)
7	7.7	ECECC121335	A	Standard Temperature Transmitter - Equipment Identification File. ECECC121335 Revision A. EDF. July 2012. (E)
7	7.7	ECECC121336	A	Software Assessment Report for STT1 Temperature Transmitter. ECECC121336 Revision A. EDF. August 2012. (E)
7	7.7	ECECC121337	A	Summary Qualification Report for STT1 Temperature Transmitter. ECECC121337 Revision A. EDF. August 2012. (E)
7	7.7	ECECC111779	A	Qualification plan for Yokogawa DX1000. ECECC111779 Revision A. EDF. February 2012. (E)
7	7.7	ECECC120095	B	SICS chart recorder - Requirements Identification File. ECECC120095 Revision B. EDF. June 2012. (E)
7	7.7	ECECC120096	B	SICS chart recorder - Equipment Identification File. ECECC120096 Revision B. EDF. June 2012. (E)
7	7.7	ECECC121090	A	Report on software assessment of Yokogawa DX1000 series electronic chart recorders. ECECC121090 Revision A. EDF. June 2012. (E)
7	7.7	ECECC121091	A	Summary Qualification Report for Yokogawa DX1000. ECECC121091 Revision A. EDF. June 2012 (E)
CHAPTER 8				
SUB-CHAPTER 8.1 - REFERENCES				

8	8.1	ETDOFC080180	A1	System Design Manual – 10 kV Normal Power Supply System (Conventional Island) (LGi) Part 2 – System operation. ETDOFC080180 Revision A1. EDF. September 2009. (E)
8	8.1	ETDOFC080274	A1	System Design Manual – 10 kV Normal Power Supply System (Conventional Island) (LGi) Part 3 – System sizing. ETDOFC080274 Revision A1. EDF. September 2009. (E)
8	8.1	ETDOFC080275	A1	System Design Manual – 10 kV Normal Power Supply System (Conventional Island) (LGi) Part 4 – Flow diagrams. ETDOFC080275 Revision A1. EDF. September 2009. (E)
8	8.1	ETDOFC070285	B1	System Design Manual – 10 kV Normal Power Supply System (Conventional Island) (LGi) Part 5 - Instrumentation and Control. ETDOFC070285 Revision B1. EDF. November 2009. (E)
8	8.1	RCCEDEC2005		Design and Construction Rules for Electrical Components of Nuclear Islands. RCC-E. AFCEN Edition. December 2005. (E)
8	8.1	ENSEMD090232	A	Detailed design of the conventional island. Study of electrical transients. ENSEMD090232 Revision A. EDF. September 2009. (E) ENSEMD090232 Revision A is the English translation of ENSEMD060315 Revision A.
8	8.1	ENSEMD090233	A	Detailed design for the conventional island. Calculation of three-phase short-circuit currents for the HVA and LV switchboards. ENSEMD090233 Revision A. EDF. September 2009. (E) ENSEMD090233 Revision A is the English translation of ENSEMD070136 Revision A.
SUB-CHAPTER 8.2 - REFERENCES				
8	8.2	ETDOFC080180	A1	System Design Manual – 10 kV Normal Power Supply System (Conventional Island) (LGi) Part 2 – System operation. ETDOFC080180 Revision A1. EDF. September 2009. (E)
8	8.2	ETDOFC080274	A1	System Design Manual – 10 kV Normal Power Supply System (Conventional Island) (LGi) Part 3 – System sizing. ETDOFC080274 Revision A1. EDF. September 2009. (E)
8	8.2	ETDOFC080275	A1	System Design Manual – 10 kV Normal Power Supply System (Conventional Island) (LGi), Part 4 – Flow diagrams. ETDOFC080275 Revision A1. EDF. September 2009. (E)
8	8.2	ETDOFC070285	B1	System Design Manual – 10 kV Normal Power Supply System (Conventional Island) (LGi), Part 5 - Instrumentation and Control. ETDOFC070285 Revision B1. EDF. November 2009. (E)
8	8.2	ETDOFC070288	C1	System Design Manual – 690 V Emergency Power Supply System (Conventional Island) (LJL/LJM), Part 5 - Instrumentation and Control. ETDOFC070288 Revision C1. EDF. December 2009. (E)
8	8.2	ETDOFC070287	C1	System Design Manual – 400 V Emergency Power Supply System (Conventional Island) (LLL/LLM), Part 5 - Instrumentation and Control. ETDOFC070287 Revision C1. EDF. December 2009. (E)
8	8.2	ETDOFC080062	A1	System Design Manual - 2 hour 220 V DC Uninterruptible Power Supply System (LAV/W), Part 2 – System operation. ETDOFC080062 Revision A1. EDF. November 2009. (E)

8	8.2	ETDOFC080260	A1	System Design Manual - 2 hour 220 V DC Uninterruptible Power Supply System (LAV/W), Part 3 – System sizing (Stage 2). ETDOFC080260 Revision A1. EDF. November 2009. (E)
8	8.2	ETDOFC080264	A1	System Design Manual - 2 hour 220 V DC Uninterruptible Power Supply System (LAV/W), Part 4 – Flow Diagrams. ETDOFC080264 Revision A1. EDF. September 2009. (E)
8	8.2	ETDOFC070283	C1	System Design Manual - 2 hour 220 V DC Uninterruptible Power Supply System (LAV/W), Part 5 - Instrumentation and Control. ETDOFC070283 Revision C1. EDF. November 2009. (E)
8	8.2	ETDOFC070286	C1	System Design Manual – 400 V Uninterrupted Power Supply (Conventional Island) (LVL/LVM), Part 5 - Instrumentation and Control. ETDOFC070286 Revision C1. EDF. November 2009. (E)
8	8.2	ETDOFC080199	A1	System Design Manual. LK – 400 V Normal Distribution System (Conventional Island), Part 2 – System operation. ETDOFC080199 Revision A1. EDF. October 2009. (E)
8	8.2	ETDOFC080258	A1	System Design Manual. LK – 400 V Normal Distribution System (Conventional Island), Part 3 – System sizing (Stage 2). ETDOFC080258 Revision A1. EDF. October 2009. (E)
8	8.2	ETDOFC080265	A1	System Design Manual. LK – 400 V Normal Distribution System (Conventional Island), Part 4 – Flow Diagrams. ETDOFC080265 Revision A1. EDF. September 2009. (E)
8	8.2	ETDOFC070282	C1	System Design Manual. LK – 400 V Normal Distribution System (Conventional Island), Part 5 - Instrumentation and Control. ETDOFC070282 Revision C1. EDF. November 2009. (E)
8	8.2	ETDOFC080063	A1	System Design Manual – 690 V Normal Distribution System (Conventional Island) (LIA/B/C/D), Part 2 – System operation. ETDOFC080063 Revision A1. EDF. September 2009. (E)
8	8.2	ETDOFC080259	A1	System Design Manual – 690 V Normal Distribution System (Conventional Island) (LIA/B/C/D), Part 3 – System sizing. ETDOFC080259 Revision A1. EDF. September 2009. (E)
8	8.2	ETDOFC080266	A1	System Design Manual – 690 V Normal Distribution System (Conventional Island) (LIA/B/C/D), Part 4 – Flow diagrams. ETDOFC080266 Revision A1. EDF. September 2009. (E)
8	8.2	ETDOFC070274	C1	System Design Manual – 690 V Normal Distribution System (Conventional Island) (LIA/B/C/D), Part 5 - Instrumentation and Control. ETDOFC070274 Revision C1. EDF. November 2009. (E)
SUB-CHAPTER 8.3 - REFERENCES				
8	8.3	RCCEDEC2005		Design and Construction Rules for Electrical Components of Nuclear Islands. RCC-E. AFCEN Edition. December 2005. (E)
8	8.3	17074709000RPT0002	5	UKEPR GDA Electrical System CAE Document. 17074-709-000-RPT-0002 Issue 05. October 2012. (E)
8	8.3	ETDOFC080180	A1	System Design Manual – 10 kV Normal Power Supply System (Conventional Island) (LGi) Part 2 – System operation. ETDOFC080180 Revision A1. EDF. September 2009. (E)

8	8.3	EZE2007en0036	D	System Design Manual – 10 kV Emergency Power Supply System (LHi), Part 2 – System Operation (Stage 2). EZE/2007/en/0036 Revision D. EDF. October 2008. (E)
8	8.3	ETDOFC080199	A1	System Design Manual. LK – 400 V Normal Distribution System (Conventional Island), Part 2 – System operation. ETDOFC080199 Revision A1. EDF. October 2009. (E)
8	8.3	ENSEMD050222	C	Book of Project Data completing the RCC-E December 2005 requirements for EPR. ENSEMD050222 Issue C. May 2007. (E)
8	8.3	EYTS2007fr0002	B1	System Design Manual 10 kV Diesel Generator Set Division 1/2/3/4 (LHP/Q/R/S), Part 2 – System operation (Stage 2). EYTS/2007/fr/0002 Revision B1. Sofinel. October 2009. (E)
8	8.3	EYTS2008fr0041	A1	System Design Manual 10 kV Diesel Generator Set Division 1/2/3/4 (LHP/Q/R/S), Part 3 – System Sizing (Stage 2). EYTS/2008/fr/0041 Revision A1. Sofinel. October 2009. (E)
8	8.3	EYTS2007fr0164	B1	System Design Manual 10 kV Diesel Generator Set Division 1/2/3/4 (LHP/Q/R/S), Part 5 - Instrumentation and Control (Stage 2). EYTS/2007/fr/0164 Revision B1. Sofinel. October 2009. (E)
8	8.3	ECEF060691	B1	Main diesel generator design basis – Functional requirements justification. ECEF060691 Revision B1. EDF. March 2010. (E) ECEF060691 Revision B1 is the English translation of ECEF060691 Revision B.
8	8.3	ECEF060964	B1	Functional justification of SBO diesel generator design basis. ECEF060964 Revision B1. EDF. April 2010. (E)
8	8.3	EYTS2008fr0068	B1	System Design Manual 690 V Emergency Diesel Generators (LJP/S), Part 3 – System sizing (Stage 2). EYTS/2008/fr/0068 Revision B1. Sofinel. October 2009. (E)
8	8.3	EYTS2008fr0034	B1	System Design Manual 690 V Emergency Diesel Generators (LJP/S), Part 5 – Instrumentation and Control (Stage 2). EYTS/2008/fr/0034 Revision B1. Sofinel. October 2009. (E)
8	8.3	ECEMA091072	A	Independence of the EDG/UDG on EPR FA3. ECEMA091072 Revision A. EDF. July 2009 (E)
8	8.3	EZE2007en0045	D	System Design Manual - 10 kV Emergency Power Supply System (LHi), Part 5 - Instrumentation and Control (Stage 2). EZE/2007/en/0045 Revision D. EDF. October 2008. (E)
8	8.3	EZE2007en0010	C	System Design Manual – 690 V AC 10 kV Emergency Power Supply System (LJi), Part 2 – System operation (Stage 2). EZE/2007/en/0010 Revision C. EDF. December 2008. (E)
8	8.3	EZE2007en0019	D	System Design Manual – 690 V AC 10 kV Emergency Power Supply System (LJi), Part 5 - Instrumentation and Control (Stage 2). EZE/2007/en/0019 Revision D. EDF. October 2008. (E)
8	8.3	EZE2007en0006	C	System Design Manual – 400 V Regulated Emergency Power Supply System in the Nuclear Island (LOi), Part 2 – System operation (Stage 2).EZE/2007/en/0006 Revision C. EDF. December 2008. (E)
8	8.3	EZE2007en0021	D	System Design Manual – 400 V Regulated Emergency Power Supply System in the Nuclear Island (LOi), Part 5 - Instrumentation and Control (Stage 2). (P5) EZE/2007/en/0021 Revision D. EDF. October 2008. (E)

8	8.3	EZE2007en0049	D	System Design Manual – 400 V Regulated Emergency Power Supply System in the Nuclear Island (LOi), Part 5 - Instrumentation and Control (Stage 2). (P5.6) EZE/2007/en/0049 Revision D. EDF. October 2008. (E)
8	8.3	EZE2007en0007	C	System Design Manual – 400 V Emergency Power Supply System (LLi), Part 2 – System operation (Stage 2). EZE/2007/en/0007 Revision C. EDF. December 2008. (E)
8	8.3	EZE2007en0020	D	System Design Manual – 400 V Emergency Power Supply System (LLi), Part 5 - Instrumentation and Control (Stage 2). EZE/2007/en/0020 Revision D. EDF. October 2008. (E)
8	8.3	EZE2007en0035	B	System Design Manual - 2 hour 220 VDC Uninterruptible Power Supply System in the Nuclear Island (LAA/B/C/D), Part 2 – System operation (Stage 2). EZE/2007/en/0035 Revision B. EDF. October 2008. (E)
8	8.3	EZE2007en0026	C	System Design Manual - 2 hour and 12 hour Uninterruptible Power Supply System (LVi), Part 2 – System operation (Stage 2). EZE/2007/en/0026 Revision C. EDF. October 2008. (E)
8	8.3	ECEF090881	A1	Functional scenarios considered in the design of the EPR Nuclear Island Batteries. ECEF090881 Revision A1. EDF. January 2010. (E)
8	8.3	EZE2007en0029	B	System Design Manual - 2 hour and 12 hour Uninterruptible Power Supply System (LVi), Part 5 - Instrumentation and Control (Stage 2). EZE/2007/en/0029 Revision B. EDF. October 2008. (E)
8	8.3	EZE2007en0039	B	System Design Manual - 2 hour 220 VDC Uninterruptible Power Supply System in the Nuclear Island (LAA/B/C/D), Part 5 - Instrumentation and Control (Stage 2). EZE/2007/en/0039 Revision B. EDF. October 2008. (E)
8	8.3	ETDOFC080062	A1	System Design Manual - 2 hour 220 V DC Uninterruptible Power Supply System (LAV/W), Part 2 – System operation. ETDOFC080062 Revision A1. EDF. November 2009. (E)
8	8.3	ETDOFC080260	A1	System Design Manual - 2 hour 220 V DC Uninterruptible Power Supply System (LAV/W), Part 3 – System sizing (Stage 2). ETDOFC080260 Revision A1. EDF. November 2009. (E)
8	8.3	ETDOFC080264	A1	System Design Manual - 2 hour 220 V DC Uninterruptible Power Supply System (LAV/W), Part 4 – Flow Diagrams. ETDOFC080264 Revision A1. EDF. September 2009. (E)
8	8.3	ETDOFC070283	C1	System Design Manual - 2 hour 220 V DC Uninterruptible Power Supply System (LAV/W), Part 5 - Instrumentation and Control. ETDOFC070283 Revision C1. EDF. November 2009. (E)
8	8.3	INPOAP913	1	The Equipment Reliability Process Description. INPO AP-913 Revision 1. Institute of Nuclear Power Operators. November 2001 (E).
8	8.3	EZE2008en0003	D	System Design Manual – 10 kV Normal Power Supply System (Nuclear Island) (LGF/G/H/I), Part 2 – System operation (Stage 2). EZE/2008/en/0003 Revision D. EDF. October 2008. (E)
8	8.3	EZE2007en0042	D	System Design Manual - 10 kV Normal Power Supply System (Nuclear Island) (LGF/G/H/I), Part 5 - Instrumentation and Control (Stage 2). EZE/2007/en/0042 Revision D. EDF. October 2008. (E)

8	8.3	EZE2007en0055	B	System Design Manual – 690 V Normal Distribution System (Nuclear Island) (LIF/LII), Part 2 – System operation (Stage 2). EZE/2007/en/0055 Revision B. EDF. December 2008. (E)
8	8.3	EZE2007en0053	D	System Design Manual – 690 V Normal Distribution System (Nuclear Island) (LIF/LII), Part 5 - Instrumentation and Control (Stage 2). EZE/2007/en/0053 Revision D. EDF. October 2008
8	8.3	EZE2007en0038	B	System Design Manual – 400 V Normal Distribution System (Nuclear Island) (LKi), Part 2 – System operation (Stage 2). EZE/2007/en/0038 Revision B. EDF. December 2008. (E)
8	8.3	EZE2007en0052	D	System Design Manual – 400 V Normal Distribution System (Nuclear Island) (LKi), Part 5 - Instrumentation and Control (Stage 2). EZE/2007/en/0052 Revision D. EDF. October 2008. (E)
8	8.3	74C02204		Book of Technical Specifications (BTS). MV Switchboards for Nuclear Power Plants. 74.C.022.04. EDF. April 2004. (E)
8	8.3	74C02302		Technical Specifications - Low Voltage Switchboards (380 V, 400 V and 690 V) for Nuclear Power Stations General Clauses. 74.C.023.02. EDF. March 2004. (E)
8	8.3	74C03202		Technical Specifications (TS) - Control Source Switchboards for Nuclear Power Stations. 74.C.032.02. EDF. March 2004. (E)
8	8.3	74C00103		Book of Technical Specifications (BTS) - Batteries Chargers / Rectifiers for Nuclear Power Plants - General Clauses. 76.C.001.03. EDF. October 2004. (E)
8	8.3	76C01702		Book of Technical Specifications (BTS) - Stationary Accumulator Batteries for Nuclear Power Plants. 76.C.017.02. EDF. November 2004. (E)
8	8.3	ETDOFC080299	B	UK Project Single Line Diagram Nuclear Island Conventional Island. ETDOFC080299 Revision B. EDF. June 2009. (E)
SUB-CHAPTER 8.4 - REFERENCES				
8	8.4	RCCEDEC2005D4000		Design and Construction Rules for Electrical components of nuclear islands. RCC-E. AFCEN Edition. December 2005. (E)
8	8.4	ENSECC090082	A	D. Soare. EMC (immunity) standards and requirements to be specified for electrical equipment. ENSECC090082 Revision A. EDF. June 2009. (E). ENSECC090082 Revision A is the English translation of ENSECC060193 Revision A.
8	8.4	CST72C04202		Book of Technical Specifications (BTS) - Main Single Phase Transformers for Three-Phase Assemblies, Main Three-Phase Transformers And Three-Phase Ancillary Power Supply Transformers of Nuclear Power Stations - General Clauses. 72.C.042.02 Final. EDF. April 2004. (E)
8	8.4	ENSEMD090183	A	Lightning safety reference base applicable to the EPR. ENSEMD090183 Revision A. EDF. August 2009. (E) ENSEMD090183 Revision A is the English version of ENSEMD060324 Revision A
8	8.4	ECEIG081012	A1	System Design Manual - Grounding System (LTR), Part 1. ECEIG081012 Revision A1. EDF. November 2009. (E)

8	8.4	ECEIG081018	A1	System Design Manual - Grounding System (LTR), Part 2 – System operation (Stage 2). ECEIG081018 Revision A1. EDF. November 2009. (E)
8	8.4	ECEIG081035	A1	System Design Manual - Grounding System (LTR), Part 3 – System sizing (Stage 2). ECEIG081035 Revision A1. EDF. November 2009. (E)
8	8.4	IECTS61000652001		Electromagnetic compatibility (EMC) – Part 6-5: Generic standards – Immunity for power station and substation environments. IEC TS 61000-6-5-2001 (E)
8	8.4	RCCEFM73		RCC-E Modification Sheet 73. Protection rules to increase the immunity of receptors to disturbance. Volume D Chapter D5500 (and A1300). AFCEN. December 2005 (E)
8	8.4	EZE2008en0003	D	System Design Manual – 10 kV Normal Power Supply System (Nuclear Island) (LGF/G/H/I), Part 2 – System operation (Stage 2). EZE/2008/en/0003 Revision D. EDF. October 2008. (E)
8	8.4	BSEN60439	1	Low-voltage switchgear and control gear assemblies. Type-tested and partially type-tested assemblies. BS EN standard 60439-1:1999. (E)
8	8.4	CST81C00403		Book of Technical Specifications (BTS). Converters (Inverters and Uninterruptible Power Supplies) for Nuclear Power Plants. General Clauses. 81.C.004.03 Final. EDF. June 2004. (E)
8	8.4	ENSEMD090015	A	Principle of selectivity and coordination of HVA and LV electrical protection systems for the EPR. ENSEMD090015 Revision A. EDF. February 2009. (E) ENSEMD090015 Revision A is the English translation of document ENSEMD080064.
8	8.4	ETDOFC080364	B1	System Design Manual. GPA: Electrical Protections for the Generator and Transformers. Part 2: System Operation. ETDOFC080364 Issue B1. EDF. January 2010. (E)
8	8.4	ENSEMD090233	A	Detailed design for the conventional island – Calculation of three-phase short-circuit currents for the HVA and LV switchboards. ENSEMD090233 Revision A (appendix 6.2). EDF. September 2009. (E)
8	8.4	IEC62305		International Standard. International Electro technical Commission – Protection against lightning. IEC 62305. 2010. (E)
SUB-CHAPTER 8.6 - REFERENCES				
8	8.6	RCCEDEC2005		Design and Construction Rules for Electrical Components of Nuclear Islands. RCC-E. AFCEN Edition. December 2005. (E)
8	8.6	ECEMA080959	A1	Key interlock principles . ECEMA080959 Issue A1. EDF. August 2011. (E)
8	8.6	IEC609090		International Standard. International Electro technical Commission – Short-Circuit currents in three-phase A.C System. IEC 60909-0. 2001. (E)
8	8.6	ENSEMD090233	A	Detailed design for the conventional island. Calculation of three-phase short-circuit currents for the HVA and LV switchboards. ENSEMD090233 Revision A. EDF. September 2009. (E) ENSEMD090233 Revision A is the English translation of ENSEMD070136 Revision A

8	8.6	ENSEMD090015	A	Principle of selectivity and coordination of HVA and LV electrical protection systems for the EPR. ENSEMD090015 Revision A. EDF. February 2009. (E) ENSEMD090015 Revision A is the English translation of document ENSEMD080064.
8	8.6	ENSEMD090232	A	Detailed design of the conventional island. Study of electrical transients. ENSEMD090232 Revision A. September 2009. (E)
8	8.6	ETDOFC080364	B1	System Design Manual. GPA: Electrical Protections for the Generator and Transformers. Part 2: System Operation. ETDOFC080364 Issue B1. EDF. January 2010. (E)
8	8.6	ETDOFC070285	B1	System Design Manual – 10 kV Normal Power Supply System (Conventional Island) (LGi), Part 5 - Instrumentation and Control. ETDOFC070285 Revision B1. EDF. November 2009. (E)
8	8.6	EZE2007en0036	D	System Design Manual – 10 kV Emergency Power Supply System (LHi), Part 2 – System Operation (Stage 2). EZE/2007/en/0036 Revision D. EDF. October 2008. (E)
8	8.6	ECEMA091072	A	Independence of the EDG/UDG on EPR FA3. ECEMA091072 Revision A. EDF. July 2009 (E)
8	8.6	ETDOFC080299	B	UK Project Single Line Diagram Nuclear Island Conventional Island. ETDOFC080299 Revision B. EDF. June 2009. (E)
8	8.6	ECEMA040711	C1	List of the Small Electrical and Instrumentation Equipment for the EPR Plant. ECEMA040711 Revision C1. EDF. August 2010. (E)
8	8.6	IEC60812		International Standard. International Electro technical Commission – Technical Analysis for system reliability – Procedure for failure mode and effects analysis (FMEA). IEC 60812. Second Edition. 2006 (E)
8	8.6	IEC61025		International Standard – International Electro technical Commission – Fault Tree Analysis. IEC 61025. 2006. (E)
8	8.6	ECEMA070291	E1	The list of programmed electrical components (PECs) and dedicated Programmable Logic Controller (PLC) integrated into various types of safety-class F1A/B or F2 electrical and electro-mechanical equipment supplied under EPR project contracts for the Flamanville 3 (FA3) unit ECEMA 070291 Revision E1. EDF. June 2010. (E)
CHAPTER 9				
SUB-CHAPTER 9.1 – REFERENCES				
9	9.1	NEERFDC102	D	N Leriquier. Equipment specification for new fuel dry storage rack NEER-F DC 102 Revision D. AREVA NP. March 2009. (E)
9	9.1	NEPCFDC364	A	EPR - FA3 Criticality study of the new fuel dry storage rack. NEPC-F DC 364 Revision A. AREVA. January 2010. (E) NEPC-F DC 364 Revision A is the English translation of NEPC-F DC 21 Revision B.
9	9.1	NEERFDC112	C	Ph Gauthier. Mechanical analyses of new fuel dry storage rack. NEER-F DC 112 Revision C. AREVA. September 2009. (E)

9	9.1	NEERFDB1286	E	Priol. New fuel dry storage rack, Equipment drawing NEER-F DB 1286 Revision E. AREVA NP. August 2009. (E)
9	9.1	NEERFDC100	D	N. Leriquier. Equipment specification for underwater fuel storage rack NEER-F DC 100 Revision D. AREVA NP. February 2009. (E)
9	9.1	NEPCFDC33	B	C Van Frank. EPR – FA3 Underwater fuel storage racks neutronic calculations. NEPC-F DC 33 Revision B. AREVA. July 2007. (E)
9	9.1	NEPCFDC164	B	Residual Decay Heat Curves for Major Components Design Purposes Heat Load inside Fuel Pool. NEPC-F DC 164 Revision B. AREVA NP. November 2008. (E)
9	9.1	DGSNRSD20729		Technical Guidelines for the design of the next generation of nuclear power plants with pressurised water reactors (index G1). DGSNR/SD2/0729. September 2004 (E)
9	9.1	NEERFDC113	D	L Mercier. Underwater fuel storage rack, Mechanical analyses NEER-F DC 113 Revision D. AREVA NP. March 2009. (E)
9	9.1	NEERFDB1285	E	Priol. Underwater fuel storage rack, Layout drawing NEER-F DB 1285 Revision E. AREVA NP. March 200.9 (E)
9	9.1	NEERFDB1287	E	Priol. Underwater fuel storage rack, Equipment drawing NEER-F DB 1287 Revision E. AREVA NP. March 2009. (E)
9	9.1	SFLEFMF2006712	G1	System Design Manual - PTR Fuel Pool Cooling System - Part 2 System Operation, SFL–EF-MF 2006-712 Revision G1. Sofinel. August 2009. (E)
9	9.1	EYTF2007fr0055	E1	System Design Manual - Spent Fuel Cooling and Purification System (PTR [FPPS/FPCS]), P3 – System and component design. EYTF/2007/fr/0055 Revision E1. Sofinel. September 2009. (E)
9	9.1	SFLEFMF2006759	F	System Design Manual - Spent Fuel Cooling and Purification System (PTR [FPPS/FPCS]) P4 – Flow diagrams SFL-EF-MF 2006-759 Revision F. Sofinel. September 2008. (E)
9	9.1	EYRC2006fr0004	B	Dossier de Système Élémentaire PTR, P4.1 – Schémas Mécaniques Fonctionnels. [System Design Manual Spent Fuel Cooling and Purification System (PTR [FPPS/FPCS]), P4.1 – System flow diagrams]. EYRC/2006/fr/0004 Revision B. SOFINEL. February 2007.
9	9.1	SFLEFMF20061559	E	Dossier de Système Élémentaire PTR, P4.2 – Schéma Mécanique Detaille. [System Design Manual Spent Fuel Cooling and Purification System (PTR [FPPS/FPCS]), P4.2 – Detailed flow diagrams]. SFL-EFMF 2006.1559 Revision E. SOFINEL. August 2008.
9	9.1	SFLEFMF2006751	F1	System Design Manual - Spent Fuel Cooling and Purification System (PTR [FPPS/FPCS]) P5 – Instrumentation and Control SFL EF MF 2006.751 Revision F1. Sofinel. September 2009. (E)
9	9.1	ECEF060501	A1	Operating situation with diversification of the heat sink by the outfall structure. ECEF060501 Revision A1. EDF. October 2012. (E) ECEF060501 Revision A1 is the English translation of ECEF060501 Revision A.
9	9.1	ECEF080499	B1	Functional study on the treatment of PCCs and RRC-As involving spent fuel pool cooling loss and draining. ECEF080499 Revision B1. EDF. November 2012. (E)

9	9.1	ECEF072078	B	Dossier de Système Élémentaire PTR, P4.1 – Schéma mécanique fonctionnel [System Design Manual Spent Fuel Cooling and Purification System (PTR [FPPS/FPCS]), P4.1 – System flow diagrams] ECEF072078 Revision B. EDF. June 2010.
9	9.1	ECEMA061033	A1	F.Darcel. System Design Manual. PMC (Fuel Handling) System Specifications. ECEMA061033 Revision A1. EDF. July 2009. (E)
9	9.1	ECEMA060983	A1	F Darcel. System Design Manual. DMK (Spent Fuel Casks Transfer Facility) System Specifications. ECEMA060983 Revision A1. EDF. March 2009. (E)
9	9.1	CST60C00703		Revaud. Book of Technical Specifications (TS) – "High safety" lifting and handling machines. CST 60.C.007.03. EDF (SEPTEN). 2004. (E)
9	9.1	ECEMA070399	A1	F Darcel. System Design Manual. PMO System Specifications. ECEMA070399 Revision A1. EDF. April 2009. (E)
9	9.1	ECEMA101802	B	N Dutheil. UK EPR GDA - Management of Nuclear Safety Significant Lifting. ECEMA101802 Revision B. EDF. December 2010. (E)
9	9.1	NEPCFDC18	E	Jamet. Functional Requirements and Interfaces – Fuel Control Rods. NEPC-F DC 18 Revision E. AREVA. 2008. (E)
9	9.1	EYTM2009FR0074	B1	J Belair, P Boursat. Reactor Building Handling Equipment - DMR - HR Plant System File P2 System Operation. EYTM /2009/FR/0074 Revision B1.Sofinel. 2009. (E) EYTM/2009/FR/0074 Revision B1 is the English translation of EYTM/2009/FR/0074 Revision B
9	9.1	EYTM2009FR0075	B1	J Belair, P Boursat. Reactor Building Handling Equipment - DMR - HR System Description - P3 Design of system and components. EYTM /2009/FR/0075 Revision B1. Sofinel. 2009. (E) EYTM/2009/FR/0075 Revision B1 is the English translation of EYTM/2009/FR/0075 Revision B
9	9.1	ENGSGC110243	B	Pools and Tanks Liner Design Requirements and Methodology. ENGSGC110243 Revision B. EDF. February 2012. (E)
9	9.1	ETCC2010		EPR Technical Code for Civil works. ETC-C 2010 Edition. AFCEN. 2010. (E)
9	9.1	ENGSGS110015	E	UK Companion Document to AFCEN ETC-C - Part 1. ENGSGC110015 Revision E. EDF. August 2012. (E) *Supersedes Revision D April 2012.*
9	9.1	ECEIG060543	C1	Design, Construction and Testing of EPR Pool Liners. ECEIG060543 Revision C1. EDF. June 2009. (E) ECEIG060543 Revision C1 is the English translation of document ECEIG060543 Revision C.
SUB-CHAPTER 9.2 – REFERENCES				
9	9.2	ETDOFC080069	A1	System Design Manual – Essential Service Water System (SEC [ESWS]), Part 2 – System operation. ETDOFC080069 Revision A1. EDF. September 2009. (E)
9	9.2	ETDOFC080276	A1	System Design Manual – Essential Service Water System (SEC [ESWS]), Part 4 – Flow Diagrams. ETDOFC080276 Revision A1. EDF. September 2009. (E)

9	9.2	ETDOIG070270	C	Dossier de Système Elémentaire - SEC, P4.2 Circuit d'eau brute secourue [System Design Manual – Essential Service Water System (SEC [ESWS]), Part 4.2 – Flow Diagrams] ETDOIG070270 Revision C. EDF. May 2008
9	9.2	ETDOFC070361	A1	System Design Manual – Essential Service Water System (SEC [ESWS]), Part 5 – Instrumentation and Control. ETDOFC070361 Revision A1. EDF. September 2009. (E)
9	9.2	ETDOFC050062	B1	Dossier de Système Elémentaire - CRF. [System Design Manual – Circulating Water System (CRF)]. ETDOFC050062 Revision B1. EDF. July 2011. (E)
9	9.2	ETDOIG060274	D	Dossier de Système Elémentaire - CRF, P4.2 Schémas Mécaniques Detaille. [System Design Manual – Circulating Water System (CRF), Part 4.2 - Detailed flow diagrams] ETDOIG060274 Revision D. EDF. September 2008.
9	9.2	ETDOFC080077	C1	System Design Manual – Circulating Water System (CRF), Part 5 – Instrumentation and Control. ETDOFC080077 Revision C1. EDF. September 2009. (E)
9	9.2	EYTS2007fr0155	B1	System Design Manual - Component Cooling Water System (RRI [CCWS]) P1 - History. EYTS/2007/fr/0155 Revision B1. Sofinel. November 2009. (E)
9	9.2	SFLEFMF2006446	F1	System Design Manual – Component Cooling Water System (RRI [CCWS]), P2 – System Operation (Stage 2 EC1). SFL-EF MF 2006.446 Revision F1. Sofinel. August 2009. (E)
9	9.2	SFLEFMF2006416	D1	System Design Manual – Component Cooling Water System (RRI [CCWS]), P3 – System and component design (Stage 2 EC1).SFL-EF MF 2006.416 Revision D1. Sofinel. August 2009. (E)
9	9.2	23952SIR00010DSE	D1	System Design Manual - Chemical reagents injection system (SIR) - Plant System File (stage 1). 23952SIR00010DSE Revision D1. January 2010. EDF. (E)
9	9.2	EZS2008EN0055	B	System Design Manual Chemical reagents injection system (SIR), Part 1, History of the System Manual. EZS/2008/en/0055 Revision B. December 2008. (E).
9	9.2	EZS2008EN0044	A	System Design Manual Chemical reagents injection system (SIR6), Part 2, System Operation. EZS/2008/en/0044 Revision A. November 2008. (E)
9	9.2	EZS2008EN0045	A	System Design Manual Chemical reagents injection system (SIR6), Part 3, System Design. EZS/2008/en/0045 Revision A. November 2008. (E)
9	9.2	EZS2008EN0046	A	System Design Manual Chemical reagents injection system (SIR6), Part 4, Flow Diagrams. EZS/2008/en/0046 Revision A. November 2008. (E)
9	9.2	EZS2008EN0050	A	System Design Manual Chemical reagents injection system (SIR6), Part 5, Instrumentation and Control. EZS/2008/en/0050 Revision A. November 2008. (E)
9	9.2	SFLEFMF2006447	E1	System Design Manual - Component Cooling Water System (RRI [CCWS]) P4 - Mechanical Diagrams (SMF and SMD). SFL- EF MF 2006.447 Revision E1. Sofinel. August 2009. (E)
9	9.2	ECEF071334	A	Dossier de Système Elémentaire - RRI, P4.1 – Schéma Mécaniques Fonctionnel. [System Design Manual – Component Cooling Water System (RRI [CCWS]), P4.1 – System flow diagrams.] ECEF071334 Revision A. April 2008.

9	9.2	EYTS2007fr0156 (EYTS2007fr0157 EYTS2007fr0158)	F	Dossier de Système Elémentaire - RRI, P4.2 – Schéma Mécanique Detaille (Stage 2 EC1). [System Design Manual – Component Cooling Water System (RRI [CCWS]), P4.2 – Detailed flow diagrams (Stage 2 EC1).] EYTS2007/fr/0156, EYTS2007/fr/0157, EYTS2007/fr/0158 Revision F. October 2008.
9	9.2	SFLEFMF2006448	F1	System Design Manual – Component Cooling Water System (RRI [CCWS]), P5 – Instrumentation and control. SFL-EF MF 2006.448 Revision F1. Sofinel. September 2009. (E)
9	9.2	EZS2007en0090	C	System Design Manual – Demineralised water distribution system (within nuclear island) (SED) Part 2 – System operation. EZS/2007/en/0090 Revision C. EDF. November 2008. (E)
9	9.2	EZS2007EN0128	C	System Design Manual - Demineralised Water Distribution System (within nuclear island) (SED) Part 1 - History of the System Manual. EZS/2007/en/0128 Revision C. EDF. December 2008 (E).
9	9.2	EZS2007en0091	C	System Design Manual – Demineralised water distribution system (within nuclear island) (SED) Part 3 – System sizing. EZS/2007/en/0091 Revision C. EDF. November 2008. (E)
9	9.2	EZS2007en0092	C	System Design Manual – Demineralised water distribution system (within nuclear island) (SED) Part 4 – Flow Diagrams. EZS/2007/en/0092 Revision C. EDF. October 2008. (E)
9	9.2	EZS2008en0010	C	System Design Manual – Demineralised water distribution system (within nuclear island) (SED) Part 5 – Instrumentation and Control. EZS/2008/en/0010 Revision C. EDF. October 2008. (E)
9	9.2	EZS2007en0111	C	System Design Manual – Demineralised pH9 water distribution system (within nuclear island) (SER) Part 2 – System operation. EZS/2007/en/0111 Revision C. EDF. January 2007. (E)
9	9.2	EZS2007en0112	D	System Design Manual – Demineralised pH9 water distribution system (within nuclear island) (SER) Part 3 – System sizing] EZS/2007/en/0112 Revision D. EDF. January 2008. (E)
9	9.2	EZS2007en0113	B	System Design Manual – Demineralised pH9 water distribution system (within nuclear island) (SER) Part 4 – Flow Diagrams. EZS/2007/en/0113 Revision B. EDF. December 2007. (E)
9	9.2	EZS2008en0059	A	System Design Manual – Demineralised pH9 water distribution system (within nuclear island) (SER) Part 5 – Instrumentation and Control. EZS/2008/en/0059 Revision A. EDF. October 2008. (E)
9	9.2	ETDOFC050283	A1	System Design Manual – Auxiliary (raw water) cooling system (SEN). ETDOFC050283 Revision A1. EDF. December 2009. (E)
9	9.2	ETDOFC070015	A1	System Design Manual – Water Intake Screening and Filtering System (SEF). ETDOFC070015 Revision A1. EDF. December 2009. (E)
9	9.2	ETDOFC080143	A1	System Design Manual – Circulating Water Filtration System (CFI [CWFS]), Part 2 – System operation. ETDOFC080143 Revision A1. EDF. August 2009. (E)
9	9.2	ETDOMA080153	A1	System Design Manual – Circulating Water Filtration System (CFI [CWFS]), Part 3 – System sizing. ETDOMA080153 Revision A1. EDF. August 2009. (E)

9	9.2	ETDOIG070273	B	Dossier de Système Elémentaire - CFI, P4.2 Schémas Mécaniques Detaille. [System Design Manual – Circulating Water Filtration System (CFI [CWFS]), Part 4.2 – Detailed Flow Diagrams]. ETDOIG070273 Revision B. EDF. March 2008.
9	9.2	ETDOFC070338	B1	System Design Manual – Circulating Water Filtration System (CFI [CWFS]), Part 5 – Instrumentation and Control. ETDOFC070338 Revision B1. EDF. August 2009. (E)
9	9.2	ETDOFC080149	A1	System Design Manual – Ultimate Cooling Water System (SRU [UCWS]), Part 2 – System operation. ETDOFC080149 Revision A1. EDF. September 2009. (E)
9	9.2	ETDOMA080206	A1	System Design Manual – Ultimate Cooling Water System (SRU [UCWS]), Part 3 – System sizing. ETDOMA080206 Revision A1. EDF. August 2009. (E)
9	9.2	ETDOIG070266	C	Dossier de Système Elémentaire - SRU, P4.2 Schémas Mécaniques Detaille [System Design Manual – Ultimate Cooling Water System (SRU [UCWS]), Part 4.2 – Detailed Flow Diagrams]. ETDOIG070266 Revision C. EDF. September 2008.
9	9.2	ETDOFC080070	C1	System Design Manual – Ultimate Cooling Water System (SRU [UCWS]), Part 5 – Instrumentation and Control. ETDOFC080070 Revision C1. EDF. September 2009. (E)
SUB-CHAPTER 9.3 – REFERENCES				
9	9.3	SFLEZS030035	E	System Design Manual – Nuclear Sampling System - Primary Side (REN [NSS]), Part 2 – System Operation (Stage 2). SFL-EZS-030035 Revision E. December 2008. (E)
9	9.3	NESSFDC625	A	System Design Manual – Nuclear Sampling System (Inside RB) (REN [NSS]), Part 2 – System Operation (Stage 2). NESS-F DC 625 Revision A. AREVA. November 2009. (E)
9	9.3	SFLEZS030036	E	System Design Manual – Nuclear Sampling System - Primary Side (REN [NSS]), Part 3 – Sizing of the System and its Components (Stage 2). SFL-EZS-030036 Revision E. December 2008. (E)
9	9.3	NESSFDC626	A	System Design Manual – Nuclear Sampling System (Inside RB) (REN [NSS]), Part 3 – System Design (Stage 2). NESS-F DC 626 Revision A. AREVA. November 2009. (E)
9	9.3	SFLEZS030037	F	System Design Manual – Nuclear Sampling System - Primary Side (REN [NSS]), Part 4 – Flow diagrams (Stage 2). SFL-EZS-030037 Revision F. EDF. December 2008. (E)
9	9.3	EZS2006en0016	B	System Design Manual – Nuclear Sampling System – Primary Side (REN [NSS]), Part 4.1 – Functional flow diagram. EZS/2006/en/0016 Revision B. EDF. October 2008. (E)
9	9.3	EZS2006en0018	A	System Design Manual – Nuclear Sampling System – Primary Side (REN [NSS]), Part 4.2 – Detailed flow diagram (Stage 2). EZS/2006/en/0018 Revision A. EDF. February 2007. (E)
9	9.3	NESSFDC627	A	System Design Manual – Nuclear Sampling System (Inside RB) (REN [NSS]), Part 4 – Flow diagrams (Stage 2). NESS-F DC 627 Revision A. AREVA. November 2009. (E)

9	9.3	EZS2006en0003	F	System Design Manual – Nuclear Sampling System - Primary Side (REN [NSS]), Part 5 – I&C (Stage 2). EZS/2006/en/0003 Revision F. EDF. October 2008. (E)
9	9.3	SFLEZS030029	C	System Design Manual – Nuclear Sampling System - Secondary Side (RES [NSS]), Part 1 - History of the System Manual. SFL-EZS-030029 Revision C. Sofinel (EDF). April 2007. (E).
9	9.3	SFLEZS030030	C	System Design Manual – Nuclear Sampling System - Secondary Side (RES [NSS]), Part 2 – System Operation (Stage 2). SFL-EZS-030030 Revision C. EDF. April 2007. (E)
9	9.3	SFLEZS030031	C	System Design Manual – Nuclear Sampling System - Secondary Side (RES NSS]), Part 3 – Sizing of the System and its components (Stage 2). SFL-EZS-030031 Revision C. EDF. April 2007. (E)
9	9.3	SFLEZS030032	C	System Design Manual – Nuclear Sampling System - Secondary Side (RES [NSS]), Part 4 – Flow diagrams (Stage 2). SFL-EZS-030032 Revision C. EDF. May 2007. (E)
9	9.3	EZS2006en0015	B	System Design Manual – Nuclear Sampling System – Secondary Side (RES [NSS]), Part 4.1 – Functional flow diagram. EZS/2006/en/0015 Revision B. EDF. July 2008. (E)
9	9.3	EZS2006en0017	A	System Design Manual – Nuclear Sampling System – Secondary Side (RES [NSS]), Part 4.2 – Detailed flow diagram. EZS/2006/en/0017 Revision A. EDF. February 2007. (E)
9	9.3	EZS2006en0004	C	System Design Manual – Nuclear Sampling System - Secondary Side (RES [NSS]), Part 5 – I&C (Stage 2). EZS/2006/en/0004 Revision C. EDF. June 2007. (E)
9	9.3	NGPS62006en1008	G	D Gschrey. Chemical and Volume Control System (RCV [CVCS]), P2 – Operation of the System. NGPS6/2006/en/1008. Revision G. AREVA. December 2008. (E)
9	9.3	NGPS62006en1018	G	Bewer M. Chemical and Volume Control System (RCV [CVCS]) P3 – System Sizing. NGPS6/2006/en/1018, Revision G. AREVA. December 2008 (E)
9	9.3	NGPS62006en1020	H	M Bewer. System Design Manual – Chemical and Volume Control System (RCV [CVCS]), P4 – Mechanical Diagrams (SMF and SMD). NGPS6/2006/en/1020 Revision H. AREVA. November 2008. (E)
9	9.3	NESSG2006en1014	F	Gaudiot M. Chemical and Volume Control System (RCV [CVCS]) P5 – Instrumentation and Control. NESS-G/2006/en/1014 Revision F. AREVA. August 2008 (E)
9	9.3	SFLEZS030008	G	G. Aranda. System Design Manual – Coolant Purification, Degasification, Storage and Treatment System (TEP [CSTS]), P2 –System Operation, SFL-EZS-030008 Revision G. Sofinel. October 2008. (E)
9	9.3	SFLEZS030009	F	R. Hechame. System Design Manual – Coolant Purification, Degasification, Storage and Treatment System (TEP [CSTS]), P3 – Sizing of the system and its components. SFL-EZS030009 Revision F. Sofinel. October 2008. (E)
9	9.3	SFLEZS030010	E	R Preiss. System Design Manual – Coolant Purification, Degasification, Storage and Treatment System (TEP [CSTS]) P4 – Flow diagrams. SFL-EZS030010 Revision E. SOFINEL. September 2008. (E)
9	9.3	EZS2006en0008	A	B Herve. System Design Manual – Coolant Purification, Degasification, Storage and Treatment System (TEP [CSTS]) P4.1 – Functional flow diagrams. EZS2006/en/0008 Revision A. SOFINEL. October 2006. (E)

9	9.3	EZS2006en0009	F	G. Aranda. System Design Manual – Coolant Purification, Degasification, Storage and Treatment System (TEP [CSTS]) P4.2 – Detailed flow diagrams. EZS2006/en/0009 Revision F. Sofinel. October 2008. (E)
9	9.3	SFLEZS030077	F	R. Hechame. System Design Manual – Coolant Purification, Degasification, Storage and Treatment System (TEP [CSTS]) P5 – Instrumentation and control. SFL-EZS030077 Revision F. Sofinel. November 2008. (E)
9	9.3	SFLEFMF2006401	E1	Muscetti R, Giraud A. Reactor Boron and Water Make-Up System (REA), P2 – System Operation. SFL-EF MF 2006.401 Revision E1. Sofinel. September 2009. (E)
9	9.3	EYTF2007fr0041	C1	Muscetti R., Giraud A.. Reactor Boron and Water Make-up System (REA) P3 – System sizing. EYTF/2007/fr/0041 Revision C1. EDF. September 2009. (E)
9	9.3	SFLEFMF2006225	E1	Klinger J., Giraud A.. Reactor Boron and Water Make-up System (REA) P4 – Mechanical Diagrams. SFL EF-MF-2006.225 Revision E1. Sofinel. September 2009. (E)
9	9.3	ECEF071383	A	Reactor Boron and Dossier de Système Elémentaire - Système d'appoint en eau et en bore (REA), P4.1 – Schémas Mécaniques Fonctionnel. [System Design Manual – Reactor Boron and Water Make-up System (REA), P4.1 – Functional flow diagram.] ECEF071383 Revision A. SOFINEL. March 2008.
9	9.3	EYTF2006FR0002	F	Dossier de Système Elémentaire - Système d'appoint en eau et en bore (REA), P4.2 – Schémas Mécaniques Détaillés. [System Design Manual – Reactor Boron and Water Make-Up System (REA [RBWMS]), P4.2 – Detailed Flow Diagrams.] EYTF-2006-FR-0002 Revision F. Sofinel. December 2008.
9	9.3	SFLEFMF2006593	F1	Klinger J., Giraud A.. Reactor Boron and Water Make-up System (REA) P5 – Instrumentation and Control. SFL-EFMF 2006.593 Revision F1. Sofinel. October 2008. (E)
SUB-CHAPTER 9.4 – REFERENCES				
9	9.4	NFISO17873		Criteria for the design and operation of ventilation systems for nuclear installations other than nuclear reactor. NF ISO 17873. April 2006. (E)
9	9.4	NFC15100		Installations électriques à basse tension – Règles. [Low voltage electrical installations – Rules.] NF C 15-100. December 2005
9	9.4	NFX44011		NF X 44011 Air Cleaning Devices, Method of measuring filter efficiency using uramine (fluorescent) aerosol. May 1972. (E)
9	9.4	EZH2008EN0031	C	System Design Manual – Nuclear Auxiliary building ventilation system (DWN [NABVS]), P1 - History of the System Manual. EZH/2008/EN/0031 Revision C. EDF. November 2008. (E)
9	9.4	EZL2006en0093	E	System Design Manual – Nuclear Auxiliary building ventilation system (DWN [NABVS]), P2 – System operation (Stage 2). EZL2006/en/0093 Revision E. EDF. November 2008. (E)
9	9.4	EZL2007en0069	F	System Design Manual – Nuclear Auxiliary building ventilation system (DWN [NABVS]), P4 – Flow diagrams. EZL2007/en/0069 Revision F. EDF. November 2008. (E)

9	9.4	ECEF081212	A	Dossier de Système Elémentaire – Ventilation du Bâtiment des Auxiliaires Nucléaires (DWN [NABVS]), P4.1 – Schéma mécanique fonctionnel. [System Design Manual – Nuclear auxiliary building ventilation system (DWN [NABVS]), P4.1 – Functional flow diagrams. ECEF081212 Revision A. EDF. July 2008.
9	9.4	EZL2006en0056	E	System Design Manual – Nuclear Auxiliary building ventilation system (DWN [NABVS]), P4.2 – Detailed flow diagrams. EZH2007/en/0056 Revision E. EDF. November 2008. (E)
9	9.4	EZL2007en0149	E	System Design Manual – Nuclear Auxiliary building ventilation system (DWN [NABVS]), P5 – Instrumentation and Control. EZL2007/en/0149 Revision E. EDF. November 2008. (E)
9	9.4	EYTS2008FR0032	B1	System Design Manual – Fuel Building Ventilation System (DWK [FBVS]) P1 - Plant System File History. EYTS/2008/fr/0032 Revision B1. EDF. October 2009. (E)
9	9.4	SFLEFMF2006164	D1	System Design Manual – Fuel Building Ventilation System (DWK [FBVS]) P2 - System Operation.SFL-EF MF-2006.164 Revision D1. Sofinel. October 2009. (E)
9	9.4	EYTS2007fr0043	B1	System Design Manual – Fuel Building Ventilation System (DWK [FBVS]) P3 – System and Component Design Basis. EYTS 2007/fr/0043 Revision B1. EDF. September 2009. (E)
9	9.4	EYTS2007fr0173	B1	System Design Manual – Fuel Building Ventilation System (DWK [FBVS]) P4 – Flow diagrams. EYTS 2007/fr/0173 Revision B1. Sofinel. October 2009. (E)
9	9.4	EYTS2007fr0172	B	Dossier de Système Elémentaire – Système de Ventilation du Bâtiment Combustible (DWK [FBVS]), P4.2 – Schéma mécanique détaille. [System Design Manual – Fuel Building Ventilation System (DWK [FBVS]), P4.2 – Detailed flow diagram.] EYTS 2007/fr/0172 Revision B. SOFINEL. April 2008.
9	9.4	EYTS2007fr0191	C1	System Design Manual – Fuel Building Ventilation System (DWK [FBVS]) P5 - Instrumentation and Control. EYTS 2007/fr/0191 Revision C1. Sofinel. October 2009. (E)
9	9.4	EZL2006en0092	F	System Design Manual – Nuclear Auxiliary building ventilation system (DWN [NABVS]) P3 Sizing Report (Stage 2). EZL2006/en/0092. Revision F. November 2008. (E)
9	9.4	EYTS2007FR0129	B1	System Design Manual – Containment Cooling Ventilation System (EVR [CCVS]) P1 - System Description Records. EYTS/2007/fr/0129 Revision B1. Sofinel. August 2009. (E)
9	9.4	EYTS2007fr0132	B1	System Design Manual – Containment Cooling Ventilation System (EVR [CCVS]) P2 - System Operation (Stage 1). EYTS/2007/Fr/0132 Revision B1. Sofinel. August 2009. (E)
9	9.4	SFLEFMF2006451	D1	System Design Manual – Containment Cooling Ventilation System (EVR [CCVS]) P3 - System Sizing]. SFL-EFMF2006.451 Revision D1. Sofinel. August 2009. (E)
9	9.4	SFLEFMF20061189	B1	System Design Manual - Containment Cooling Ventilation System EVR [CCVS] - P4 Mechanical diagrams. SFL-EFMF 2006.1189 Revision B1. August 2009. Sofinel. (E)

9	9.4	SFLEFMF20061186	A	Dossier de Système Elémentaire – EVR [CCVS], P4.1 – Schémas mécaniques fonctionnel (Stage 1). [System Design Manual – Containment Cooling Ventilation System (EVR [CCVS]), P4.1 – Functional flow Diagrams (Stage 1)]. SFL-EF-MF-2006.1186 Revision A. SOFINEL. July 2006.
9	9.4	SFLEFMF20061187	B	Dossier de Système Elémentaire – Ventilation Continue Enceinte System (EVR [CCVS]), P4.2 – Schéma mécanique détaille. [System Design Manual – Containment Cooling Ventilation System (EVR [CCVS]), P4.2 – Detailed flow diagram.] SFL-EFMF 2006.1187 Revision B. SOFINEL. June 2007
9	9.4	EYTS2006fr0017	D1	System Design Manual – Containment Cooling Ventilation System EVR [CCVS], P5 - Instrumentation and Control. EYTS 2006/fr/0017 Revision D1. EDF. August 2010. (E)
9	9.4	EYTS2007FR0141	B1	System Design Manual – Reactor Building Internal Filtration System (EVF) P1 - Plant System History File. EYTS/2007/fr/0141 Revision B1. Sofinel. October 2009. (E)
9	9.4	SFLEFMF2006166	C1	System Design Manual – Reactor Building Internal Filtration System (EVF) P2 - System Operation (Stage 1). SFL-EFMF2006.166 Revision C1 PREL. Sofinel. October 2009. (E)
9	9.4	SFLEFMF2006165	C1	System Design Manual – Reactor Building Internal Filtration System (EVF) P3 - System Sizing. SFL-EFMF2006.165 Revision C1. Sofinel. October 2009. (E)
9	9.4	EYTS2007fr0118	B1	System Design Manual – Reactor Building Internal Filtration System (EVF) P4 - Flow diagrams. EYTS 2007/fr/0118 Revision B1. Sofinel. October 2009. (E)
9	9.4	SFLEFMF2006516	A	Dossier de Système Elémentaire – EVF, P4.1 – Schéma mécanique fonctionnel (Stage 1 PREL). [System Design Manual – Reactor Building Internal Filtration System (EVF), P4.1 – Functional flow diagrams (Stage 1)].SFL-EFMF2006.516 Revision A. SOFINEL. July 2006.
9	9.4	SFLEFMF20061141	D	Dossier de Système Elémentaire – Filtration Interne (EVF), P4.2 – Schéma mécanique détaille. [System Design Manual – Reactor Building Internal Filtration System (EVF), P4.2 – Detailed flow diagrams]. SFL-EFMF2006.1141 Revision D. SOFINEL. December 2008.
9	9.4	EYTS2006fr0012	D1	System Design Manual – Reactor Building Internal Filtration System (EVF) P5 - Instrumentation and Control. EYTS 2006/fr/0012 Revision D1. Sofinel. November 2009. (E)
9	9.4	EYTS2007fr0128	B1	System Design Manual – Containment Sweep Ventilation System (EBA [CSVS]) P1 - System Description Records. EYTS/2007/fr/0128 Revision B1. Sofinel. November 2009. (E)
9	9.4	EYTS2007fr0131	B1	System Design Manual – Containment Sweep Ventilation System (EBA [CSVS]) P2 - System Operation (Stage 1). EYTS/2007/fr/0131 Revision B1. Sofinel. November 2009. (E)
9	9.4	SFLEFMF2006502	B1	System Design Manual – Containment Sweep Ventilation System (EBA [CSVS]) P3 - System Sizing. SFL-EFMF2006.502 Revision B1. Sofinel. March 2009. (E)
9	9.4	SFLEFMF20061188	C1	System Design Manual – Containment Sweep Ventilation System (EBA [CSVS]) P4 – Flow diagrams. SFL-EFMF2006.1188 Revision C1. Sofinel. November 2009. (E)

9	9.4	SFLEFMF20061184	A	Dossier de Système Elémentaire – EBA, P4.1 – Schéma mécanique fonctionnel (Stage 1 PREL). [System Design Manual – Containment Sweep Ventilation System (EBA [CSVS]), P4.1 – Functional flow diagrams (Stage 1)]. SFL-EF MF-2006.1184 Revision A. SOFINEL. July 2006.
9	9.4	SFLEFMF20061185	C	Dossier de Système Elémentaire – Ventilation Balayage Enceinte Système (EBA [CSVS]), P4.2 –Schéma mécanique détaille. [System Design Manual – Containment Sweep Ventilation System (EBA [CSVS]), P4.2 – Detailed flow diagrams]. SFL-EF MF-2006.1185 Revision C. SOFINEL. September 2007.
9	9.4	EYTS2006fr0016	D1	System Design Manual – Containment Sweep Ventilation System (EBA [CSVS]) P5 - Instrumentation and Control. EYTS/2006/fr/0016 Revision D1. Sofinel. November 2009. (E)
9	9.4	EZH2008EN0044	C	System Design Manual – Safeguard Building Controlled Area Ventilation System (DWL [CSBVS)], P1 - History of the System. EZH/2008/EN/0044 Revision C. EDF. November 2008. (E).
9	9.4	SFLEZL030008	G	System Design Manual – Safeguard Building Controlled Area Ventilation System (DWL [CSBVS)], P2 – System operation (Stage 1)SFL-EZL-030008 Revision G. EDF. November 2008. (E)
9	9.4	EZH2007en0033	E	System Design Manual – Safeguard Building Controlled Area Ventilation System (DWL [CSBVS)], P3 – Design Report (Stage 1) EZH/2007/en/0033 Revision E. EDF. November 2008. (E)
9	9.4	EZL2007en0122	F	System Design Manual – Safeguard Building Controlled Area Ventilation System (DWL [CSBVS]), P4 - Flow diagrams. EZL/2007/en/0122 Revision F. EDF. November 2008. (E)
9	9.4	ECEF081213	A	Dossier de Système Elémentaire – Ventilation de la Zone Contrôlée des Bâtiments des Auxiliaires de Sauvegarde (DWL), P4.1 – Schéma mécanique fonctionnel. [System Design Manual – Ventilation System of the Controlled Area of the Safeguard Buildings (DWL [CSBVS]), P4.1 – Functional flow diagrams.] ECEF081213 Revision A. EDF. July 2008.
9	9.4	EZH2007EN0048	E	System Design Manual – Safeguard Building Controlled Area Ventilation System (DWL [CSBVS]), P4.2 – Detailed flow diagrams. EZH/2007/en/0048 Revision E. EDF. November 2008. (E)
9	9.4	EZH2007en0107	D	System Design Manual – Safeguard Building Controlled Area Ventilation System (DWL [CSBVS]), P5 – Instrumentation and Control. EZH/2007/en/0107 Revision D. EDF. November 2008. (E)
9	9.4	ECES121567	A	GDA - DEL / DVL - Conceptual Design Note. ECES121567 Revision A. EDF. November 2012. (E)
9	9.4	EZH2008EN0045	C	System Design Manual – Safeguard Building Non-Controlled Area Ventilation System (DVL [SBVSE], P1 - History of the System Manual. EZH/2008/en/0045 Revision C. EDF. November 2008. (E)
9	9.4	EZL2007en0077	F	System Design Manual – Safeguard Building Non-Controlled Area Ventilation System (DVL [SBVSE], P2 – System operation (Stage 2) EZL/2007/en/0077 Revision F. EDF. November 2008. (E)
9	9.4	EZH2007en0035	F	System Design Manual – Safeguard Building Non-Controlled Area Ventilation System (DVL [SBVSE]), P3 – Design Report (Stage 2). EZH/2007/en/0035 Revision F. EDF. November 2008. (E)
9	9.4	EZL2007en0072	F	System Design Manual – Safeguard Building Non-Controlled Area Ventilation System (DVL [SBVSE]), P4 - Flow diagrams (Stage 2) EZL/2007/en/0072 Revision F. EDF. November 2008. (E)

9	9.4	ECEF081214	A	Dossier de Système Elémentaire – Ventilation de la Zone Non-contrôlée des Bâtiments des Auxiliaires de Sauvegarde (DVL), P4.1 – Schéma mécanique fonctionnel. [System Design Manual – Safeguard Building Non-Controlled Area Ventilation System (DVL [SBVSE]), P4.1 – Functional flow diagrams.] ECEF081214 Revision A. EDF. July 2008.
9	9.4	EZH2007EN0049	E	System Design Manual – Safeguard Building Non-Controlled Area Ventilation System (DVL [SBVSE]), P4.2 – Detailed flow diagrams. EZH/2007/en/0049 Revision E. EDF. November 2008. (E)
9	9.4	EZH2007en0025	D	System Design Manual – Safeguard Building Non-Controlled Area Ventilation System (DVL [SBVSE]), P5 - Instrumentation and Control (Stage 2) EZH/2007/en/0025 Revision D. EDF. November 2008. (E)
9	9.4	EZH2008EN0042	C	System Design Manual – Main Control Room Air Conditioning System (DCL [CRACS], P1 - History of the System Manual. EZH/2008/en/0042 Revision C. EDF. November 2008. (E)
9	9.4	EZL2006en0077	F	System Design Manual – Main Control Room Air Conditioning System (DCL [CRACS], P2 – System operation (Stage 1) EZL/2006/en/0077 Revision F. EDF. November 2008. (E)
9	9.4	EZH2007en0036	F	System Design Manual – Main Control Room Air Conditioning System (DCL [CRACS]), P3 - System Sizing. EZH/2007/en/0036 Revision F. EDF. November 2008. (E)
9	9.4	EZH2007en0051	F	System Design Manual – Main Control Room Air Conditioning System (DCL [CRACS]), P4 - Flow diagrams. EZH/2007/en/0051 Revision F. EDF. November 2008. (E)
9	9.4	ECEF081239	A	Dossier de Système Elémentaire – Ventilation de la Salle de Commande (DCL), P4.1 – Schéma mécanique fonctionnel. [System Design Manual – Main Control Room Air Conditioning System (DCL [CRACS]), P4.1 – Functional flow diagrams.] ECEF081239 Revision A. EDF. April 2008.
9	9.4	EZH2007EN0052	F	System Design Manual – Main Control Room Air Conditioning System (DCL [CRACS]), P4.2 – Detailed flow diagrams. EZH/2007/en/0052 Revision F. EDF. November 2008. (E)
9	9.4	EZH2007en0011	E	System Design Manual – Main Control Room Air Conditioning System (DCL [CRACS]), P5 - Instrumentation and Control. EZH/2007/en/0011 Revision E. EDF. October 2008. (E)
9	9.4	SFLEFMF2006430	E1	System Design Manual – Main Diesel and SBO Diesel Building Ventilation System (DVD), P2 - System Operation (Stage 2). SFL-EF MF 2006.430 Revision E1 PREL. SOFINEL. October 2009. (E)
9	9.4	SFLEFMF2006431	D1	System Design Manual – Main Diesel and SBO Diesel Building Ventilation System (DVD) P3 - System Sizing (Stage 2). SFL-EF MF 2006.431 Revision D1. Sofinel. October 2009. (E)
9	9.4	EYTS2007fr0135	C1	System Design Manual – Main Diesel and SBO Diesel Building Ventilation System (DVD), P4 - Flow diagrams (Stage 2). EYTS2007/fr/0135 Revision C1 PREL. SOFINEL. July 2009. (E)
9	9.4	ECEF071412	A	Dossier de Système Elémentaire – Système de Ventilation des Bâtiments Diesel (DVD), P4.1 – Schéma mécanique fonctionnel. [System Design Manual – Main Diesel and SBO Diesel Building Ventilation System (DVD), P4.1 – Functional flow diagrams.] ECEF071412 Revision A. SOFINEL. April 2008.

9	9.4	SFLEFMF20061650	F1	System Design Manual – Main Diesel and SBO Diesel Building Ventilation System (DVD) P5 - Instrumentation and Control (Stage 2). SFL-EF MF 2006.1650 Revision F1. Sofinel. October 2009. (E)
9	9.4	EZH2007EN0142	D	System Design Manual – Safety Chilled Water System (DEL), P1 - History of System Manual. EZH2007EN0142, Revision D. EDF. November 2008. (E)
9	9.4	SFLEZL030004	H	System Design Manual – Safety Chilled Water System (DEL), P2 - System Operation. SFL-EZL-030004 Revision H. EDF. November 2008. (E)
9	9.4	EZH2007en0001	F	System Design Manual – Safety Chilled Water System (DEL), P3 - System and component design. EZH2007/en/0001 Revision F. EDF. November 2008. (E)
9	9.4	EZL2007en0067	F	System Design Manual – Safety Chilled Water System (DEL), P4 - Flow diagrams. EZL2007/en/0067 Revision F. EDF. November 2008. (E)
9	9.4	ECEF072079	A	Dossier de Système Elémentaire – Eau Réfrigérée de Sureté (DEL), P4.1 - Schéma mécanique fonctionnel (Stage 1). [System Design Manual – Safety Chilled Water System (DEL), P4.1 – Functional flow diagrams (Stage 1)].ECEF072079 Revision A. EDF. December 2007.
9	9.4	SFLEZL030015	A	Dossier de Système Elémentaire – Système d'Eau Réfrigérée de Sureté (DEL), P4.2 - Schéma mécanique détaillé. [System Design Manual – Safety Chilled Water System (DEL), P4.2 – Detailed flow diagrams]. SFL-EZL-030015 Revision A. EDF. September 2006.
9	9.4	SFLEZL030023	H	System Design Manual – Safety Chilled Water System (DEL), P5 - Instrumentation and Control. SFL-EZL-030023 Revision H. EDF. October 2008. (E)
9	9.4	EZH2008EN0043	C	System Design Manual – Operational Chilled Water System (DER), P1 - History of System Manual. EZH2008EN0043, Revision C. EDF. November 2008. (E)
9	9.4	EZL2007en0078	F	System Design Manual – Operational Chilled Water System (DER), P2 - System Operation. EZL2007/en/0078 Revision F. EDF. November 2008. (E)
9	9.4	EZH2007EN0002	F	System Design Manual – Operational Chilled Water System (DER), P3 - System and component design. EZH2007/en/0002 Revision F. EDF. November 2008. (E)
9	9.4	EZL2007en0080	F	System Design Manual – Operational Chilled Water System (DER), P4 - Flow diagrams. EZL2007/en/0080 Revision F. EDF. November 2008. (E)
9	9.4	ECEF080980	A	Dossier de Système Elémentaire – DER, P4.1 - Schéma mécanique fonctionnel (Stage 1). [System Design Manual – Operational Chilled Water System (DER), P4.1 – Functional flow diagrams (Stage 1)]. ECEF080980. Revision A. EDF. May 2008.
9	9.4	SFLEZL030013	F	System Design Manual – Operational Chilled Water System (DER), P4.2 – Detailed flow diagrams. SFL-EZL-030013 Revision F. EDF. November 2008. (E)
9	9.4	EZH2007en0044	F	System Design Manual – Operational Chilled Water System (DER), P5 - Instrumentation and Control. EZH2007/en/0044 Revision F. EDF. October 2008. (E)

9	9.4	ETDOFC060066	A1	System Design Manual – Circulating Water Pumping Station Ventilation System (DVP). ETDOFC060066 Revision A1. EDF. December 2009. (E)
9	9.4	EYTS2007fr0138	A1	System Design Manual – Operating Building Contaminable Room Ventilation System (8DWB) P1 -DSE History. EYTS/2007/fr/0138 Revision A1. Sofinel. August 2009. (E)
9	9.4	SFLEFMF2006131	D1	System Design Manual – Operating Building Contaminable Room Ventilation System (8DWB) P2 - System Operation (Stage 2). SFL-EF MF 2006.131 Revision D1. Sofinel. August 2009. (E)
9	9.4	SFLEFMF2006132	E1	System Design Manual – Operating Building Contaminable Room Ventilation System (8DWB) P3 - System and component design (Stage 2). SFL-EF MF 2006.132 Revision E1. Sofinel. August 2009. (E)
9	9.4	EYTS2006fr0001	B1	System Design Manual – Operating Building Contaminable Room Ventilation system (8DWB) P4 - Flow diagrams (Stage 2). EYTS2006/fr/0001 Revision B1. Sofinel. August 2009. (E)
9	9.4	EYTS2006fr0013	A	Dossier de Système Elémentaire – 8DWB, P4.1 - Schémas mécaniques fonctionnels. [System Design Manual – Operating Building Contaminable Room Ventilation System (8DWB), P4.1 – Functional flow diagrams]. EYTS/2006/fr/0013 Revision A. SOFINEL. September 2006.
9	9.4	EYTS2006fr0003	C	Dossier de Système Elémentaire – Système de Ventilation de la Zone Contrôlée du POE (8DWB), P4.2 - Schéma mécanique détaillé. [System Design Manual – Operating Building Contaminable Room Ventilation System (8DWB), P4.2 – Detailed flow diagrams]. EYTS/2006/fr/0003 Revision C. SOFINEL. February 2008.
9	9.4	EYTS2006fr0006	B1	System Design Manual – Operating Building Contaminable Room Ventilation System (8DWB) P5 - Instrumentation and Control (Stage 2). EYTS2006/fr/0006 Revision B1. Sofinel. August 2009. (E)
9	9.4	EYTS2008FR0038	B1	System Design Manual – Effluent Treatment Building Ventilation System (8DWQ) P1 - History. EYTS/2008/fr/0038 Revision B1. Sofinel. November 2009. (E)
9	9.4	EYTS2007fr0108	B1	System Design Manual – Effluent Treatment Building Ventilation System (8DWQ) P2 - System Operation (Stage 1). EYTS/2007/fr/0108 Revision B1. Sofinel. November 2009. (E)
9	9.4	EYTS2007fr0109	B1	System Design Manual – Effluent Treatment Building Ventilation System (8DWQ), P3 - System and component design (Stage 1). EYTS/2007/fr/0109 Revision B1. Sofinel. November 2009. (E)
9	9.4	EYTS2007fr0189	B1	System Design Manual – Effluent Treatment Building Ventilation System (8DWQ) P4 - Flow diagrams. EYTS/2007/fr/0189 Revision B1. Sofinel. November 2009. (E)
9	9.4	EYTS2007fr0188	B	Dossier de Système Elémentaire – Système de Ventilation du BTE (8DWQ), P4.2 - Schéma mécanique détaillé. [System Design Manual – Effluent Treatment Building Ventilation System (8DWQ), P4.2 – Detailed flow diagrams]. EYTS2007/fr/0188 Revision B. SOFINEL. May 2008.
9	9.4	EYTS2007fr0184	C1	System Design Manual – Effluent Treatment Building Ventilation System (8DWQ) P5 - Instrumentation and Control. EYTS/2007/fr/0184 Revision C1. Sofinel. November 2009. (E)

9	9.4	EYTS2008FR0009	B1	System Design Manual – Ventilation of Contaminable Rooms in the Access Tower (DWW [ABVS]) P1 - History. EYTS/2008/fr/0009 Revision B1. Sofinel. November 2009. (E)
9	9.4	EYTS2007fr0187	D1	System Design Manual – Ventilation of Contaminable Rooms in the Access Tower (DWW [ABVS]) P2 - System Operation (Stage 2). EYTS2007/fr/0187 Revision D1 BPE. Sofinel. November 2009. (E)
9	9.4	EYTS2007fr0087	B1	System Design Manual – Ventilation of Contaminable Rooms in the Access Tower DWW [ABVS]) P3 - System Sizing (Stage 2). EYTS2007/fr/0087 Revision B1 BPE. Sofinel. November 2009. (E)
9	9.4	EYTS2008fr0023	B1	System Design Manual – Ventilation of Contaminable Rooms in the Access Tower (DWW [ABVS]) P4 - Flow diagrams (Stage 2). EYTS2008/fr/0023 Revision B1 BPE. Sofinel. November 2009. (E)
9	9.4	ECEF081184	A	Dossier de Système Elémentaire – Système de Ventilation de la Zone (DWW), P4.1 - Schéma mécanique fonctionnel. [System Design Manual – Access Building Controlled Area Ventilation System-(DWW [ABVS]), P4.1 – Functional flow diagrams]. ECEF081184 Revision A. SOFINEL. September 2008
9	9.4	EYTS2007fr0148	C	Dossier de Système Elémentaire – Ventilation de Zone Contrôlée de la Tour d'Accès (DWW), P4.2 - Schéma mécanique détaillé. [System Design Manual – Access Building Controlled Area Ventilation System (DWW [ABVS]), P4.2 – Detailed flow diagrams]. EYTS/2007/fr/0148 Revision C. SOFINEL. September 2008.
9	9.4	EYTS2007fr0165	C1	System Design Manual – Access Building Controlled Area Ventilation System (DWW [ABVS]) P5 - Instrumentation and Control (Stage 2). EYTS2007/fr/0165 Revision C1 BPE. Sofinel. November 2009. (E)
SUB-CHAPTER 9.5 – REFERENCES				
9	9.5	ECEMA061374	A1	EPR FA3 JDT [FDS] System Specifications. ECEMA061374 Revision A1. EDF. January 2011. (E)
9	9.5	EYTE2008fr0104	A1	System Design Manual Fire Detection System (JDT [FDS]) Part 2 – System operation. EYTE2008/fr/0104 Revision A1. Sofinel. September 2009. (E)
9	9.5	EYTE2008fr0107	A1	System Design Manual Fire Detection System (JDT [FDS]), Part 3 – system sizing. EYTE2008/fr/0107 Revision A1. Sofinel. September 2009. (E)
9	9.5	EYTE2008fr0106	A1	System Design Manual Fire Detection System (JDT [FDS]) Part 4 – Flow diagrams. EYTE2008/fr/0106 Revision A1. Sofinel. September 2009. (E)
9	9.5	EYTE2008fr0105	A1	System Design Manual Fire Detection System (JDT [FDS]), Part 5 – Instrumentation and Control. EYTE2008/fr/0105 Revision A1. Sofinel. September 2009. (E)
9	9.5	EZH2007en0005	C	System Design Manual - Fire Fighting Water System of the Nuclear Island (JPI), Part 1. EZH2007/en/0005 Revision C. EDF. September 2008 (E)
9	9.5	EZH2007en0006	D	System Design Manual - Fire Fighting Water System of the Nuclear Island (JPI), Part 2 – System operation. EZH2007/en/0006 Revision D. EDF. July 2008 (E)
9	9.5	EZH2007en0007	E	System Design Manual - Fire Fighting Water System of the Nuclear Island (JPI), Part 3 – System Sizing. EZH2007/en/0007 Revision E. EDF. July 2008 (E)

9	9.5	EZH2007en0009	D	System Design Manual - Fire Fighting Water System of the Nuclear Island (JPI), Part 4 – Flow diagrams. EZH2007/en/0009 Revision D. EDF. May 2008 (E)
9	9.5	EZH2007en0010	D	System Design Manual - Fire Fighting Water System of the Nuclear Island (JPI), Part 4.2 – Process and Instrumentation diagrams. EZH/2007/en/0010 Revision D. EDF. July 2008. (E)
9	9.5	SFLEZH2007en0021	B	System Design Manual - Fire Fighting Water System of the Nuclear Island (JPI), Part 5 Plant System File – Instrumentation and Control SFL-EZH/2007/en/0021 Revision B. EDF. January 2008. (E)
9	9.5	EYTF2007fr0036	D1	System Design Manual Effluent Treatment Building Protection and Fire Fighting Water Distribution System (8JPI), Part 2 – System operation. EYTF2007/fr/0036 Revision D1. Sofinel. September 2009. (E)
9	9.5	EYTF2007fr0045	C	System Design Manual Effluent Treatment Building Protection and Fire Fighting Water Distribution System (8JPI), Part 3 – System sizing. EYTF2007/fr/0045 Revision C. EDF. July 2008 (E)
9	9.5	EYTF2007fr0057	C1	System Design Manual Effluent Treatment Building Protection and Fire Fighting Water Distribution System (8JPI), Part 4 – Flow Diagrams. EYTF2007/fr/0057 Revision C1. Sofinel. September 2009. (E)
9	9.5	EYTF2007fr0056	C	Dossier de Système Elémentaire – Protection Incendie du BTE (8JPI), P4.2 – Schéma mécanique détaillé. [System Design Manual – Effluent Treatment Building Protection and Fire Fighting Water Distribution System (8JPI), Part 4.2 – Detailed flow diagram.] EYTF2007/fr/0056 Revision C. EDF. July 2008
9	9.5	EYTF2007fr0077	C1	System Design Manual Effluent Treatment Building Protection and Fire Fighting Water Distribution System (8JPI), Part 5 – Instrumentation and Control. EYTF2007/fr/0077 Revision C1. SOFINEL. September 2009. (E)
9	9.5	ETDOIG080014	A1	System Design Manual - JAC: Classified fire-fighting water supply system Part 1: Plant System File Evolution. ETDOIG080014 Revision A1. EDF. August 2009. (E)
9	9.5	ETDOIG080015	A1	System Design Manual - JAC: Flamanville 3 classified fire-fighting water supply system Part 2 : System operation. ETDOIG080015 Revision A1. EDF. August 2009. (E)
9	9.5	ETDOIG080016	A1	System Design Manual - JAC: Flamanville 3 classified fire-fighting water supply system Part 3: System and component design. ETDOIG080016 Revision A1. EDF. August 2009. (E)
9	9.5	ETDOIG080017	A1	System Design Manual - JAC: Flamanville 3 Classified Fire-Fighting Water Supply System Part 4: Flow diagrams. ETDOIG080017 Revision A1. EDF. August 2009. (E)
9	9.5	ETDOFC080059	A1	System Design Manual - JAC: Flamanville 3 classified fire-fighting water supply system Part 5: Instrumentation and Control. ETDOFC080059 Revision A1. EDF. August 2009. (E)
9	9.5	EYTF2008fr0078	B1	System Design Manual - Diesel Generator Protection and Fire-Fighting Water Distribution System (JPV) Part 1. EYTF2008/fr/0078 Revision B1. Sofinel. September 2009. (E)
9	9.5	EYTF2007fr0028	C1	System Design Manual - Diesel Generator Protection and Fire-Fighting Water Distribution System (JPV), Part 2 – System operation. EYTF2007/fr/0028 Revision C1. September 2009. Sofinel. (E)

9	9.5	EYTF2007fr0035	C1	System Design Manual - Diesel Generator Protection and Fire-Fighting Water Distribution System (JPV) Part 4 – Flow Diagrams. EYTF2007/fr/0035 Revision C1. Sofinel. September 2009. (E)
9	9.5	EYTF2007fr0033	C	Dossier de Système Elémentaire – Protection et Distribution Eau Incendie Diesels (JPV), P4.2 – Schéma mécanique détaillé. [System Design Manual – Diesel Generator Protection and Fire-Fighting Water Distribution System (JPV), Part 4.2 – Detailed flow diagram.] EYTF/2007/fr/0033 Revision C. EDF. July 2008.
9	9.5	EYTF2007fr0054	E1	System Design Manual - Diesel Generator Protection and Fire-Fighting Water Distribution System (JPV) Part 5 – Instrumentation and Control. EYTF2007/fr/0054 Revision E1. Sofinel. September 2009. (E)
9	9.5	EZH2008en0041	C	System Design Manual – Electrical Building Smoke Extraction System (DFL) Part 1. EZH2008/en/0041 Revision C. EDF. November 2008. (E)
9	9.5	EZH2007en0012	F	System Design Manual – Electrical Building Smoke Extraction System (DFL), Part 2 – System operation. EZH2007/en/0012 Revision F. EDF. October 2008 (E)
9	9.5	EZH2007en0153	E	System Design Manual – Electrical Building Smoke Extraction System (DFL), Part 3 – System sizing. EZH2007/en/0153 Revision E. EDF. October 2008 (E)
9	9.5	EZH2007en0140	F	System Design Manual – Electrical Building Smoke Extraction System (DFL), Part 4 – Flow diagrams. EZH2007/en/0140 Revision F. EDF. October 2008 (E)
9	9.5	ECEF081218	A	Dossier de Système Elémentaire – Système de Contrôle des Fumées (DFL), P4.1 – Schéma mécanique fonctionnel. [System Design Manual —Smoke Control System (DFL), Part 4.1 – Detailed flow diagram.] ECEF081218 Revision A. EDF. July 2008.
9	9.5	EZH2007en0150	F1	System Design Manual – Electrical Building Smoke Extraction System (DFL), P4.2 - SMD. EZH/2007/en/0150 Revision F1. EDF. October 2008. (E)
9	9.5	EZH2007en0111	D	System Design Manual – Electrical Building Smoke Extraction System (DFL), Part 5 Instrumentation and Control. EZH2007/en/0111 Revision D. EDF. October 2008 (E)
9	9.5	ECUK100261	A	UK EPR – Specification of the Door Monitoring System. ECUK100261 Revision A. EDF. June 2010. (E)
9	9.5	EYTS2007fr0002	B1	System Design Manual - 10 kV Diesel Generator Set Division 1/2/3/4 (LHP/Q/R/S), Part 2 – System operation (Stage 2). EYTS/2007/fr/0002 Revision B1. Sofinel. October 2009. (E)
9	9.5	EYTS2008fr0041	A1	System Design Manual - 10 kV Diesel Generator Set Division 1/2/3/4 (LHP/Q/R/S), Part 3 – System sizing (Stage 2). EYTS/2008/fr/0041 Revision A1. Sofinel. October 2009. (E)
9	9.5	EYTS2007fr0164	B1	System Design Manual - 10 kV Diesel Generator Set Division 1/2/3/4 (LHP/Q/R/S), Part 5 - Instrumentation and Control (Stage 2). EYTS/2007/fr/0164 Revision B1. Sofinel. October 2009. (E)
9	9.5	EYTS2008fr0068	B1	System Design Manual - 690V Emergency Diesel Generators (LJP/S), Part 3 – System sizing (Stage 2). EYTS/2008/fr/0068 Revision B1. Sofinel. October 2009. (E)
9	9.5	ECEF060964	B1	Functional justification of SBO diesel generator design basis. ECEF060964 Revision B1. EDF. April 2010. (E)

9	9.5	EYTS2008fr0034	B1	System Design Manual 690 V Emergency Diesel Generators (LJP/S), Part 5 – Instrumentation and Control (Stage 2). EYTS/2008/fr/0034 Revision B1. Sofinel. October 2009. (E)
9	9.5	EZS2007en0080	B	System Design Manual - Compressed Air System (SAR) Part 1. EZS/2007/en/0080 Revision B. Sofinel. August 2008. (E)
9	9.5	EZS2007en0024	C	System Design Manual - Compressed Air System (SAR) Part 2 – System operation. EZS/2007/en/0024 Revision C. Sofinel. August 2008. (E)
9	9.5	EZS2007en0049	C	System Design Manual - Compressed Air System (SAR) Part 3 – System design. EZS/2007/en/0049 Revision C. Sofinel. August 2009.(E)
9	9.5	EZS/2007en0050	B	System Design Manual - Compressed Air System (SAR), Part 4 – Flow Diagrams. EZS/2007/en/0050 Revision B. Sofinel. August 2008. (E)
9	9.5	EZS2007en0069	C	System Design Manual - Compressed Air System (SAR), Part 4.2 – Detailed Flow Diagrams. EZS/2007/en/0069 Revision C. Sofinel. August 2008. (E)
9	9.5	EZS2007en0115	B	System Design Manual - Compressed Air System (SAR), Part 5 – Instrumentation & Control EZS/2007/en/0115 Revision B. Sofinel. August 2008. (E)
9	9.5	ECEIG061187	A1	System Design Manual - Specifications for DN./ DS. Systems: “Lighting of EPR rooms”. ECEIG061187 Revision A1. EDF. June 2009. (E)
9	9.5	SFLEZS030063	D	Stoehr S., Kokemueller D. Nitrogen Gas Distribution System (SGN) P2 - System Operation. SFL-EZS-030063 Revision D PREL. Sofinel. November 2008. (E)
9	9.5	SFLEZS030066	D	Stoehr S., Kokemueller D. Nitrogen Gas Distribution System (SGN) P3 – Sizing of the system and its components. SFL-EZS030066 Revision D PREL. Sofinel. November 2008. (E)
9	9.5	SFLEZS030056	D	Stoehr S., Kokemueller D. Nitrogen Gas Distribution System (SGN) P4 - Flow diagrams. SFL-EZS-030056 Revision D PREL. Sofinel. October 2008. (E)
9	9.5	EZS2007en0010	C	Stoehr S., Kokemueller D. Nitrogen Gas Distribution System (SGN) P5 – Instrumentation and Control (I&C). EZS/2007/en/0010 Revision C PREL. Sofinel. November 2008. (E)
9	9.5	SFLEZS030069	E	Stoehr S., Kokemueller D. Nitrogen Gas Distribution System (SGN) P1. SFL-EZS-030069 Revision E PREL. Sofinel. December 2008. (E)
9	9.5	SFLEZS030068	E	Stoehr S., Kokemueller D. System Design Manual - Oxygen Gas Distribution System (SGO) P1 - History of the System Manual. SFL-EZS-030068 Revision E PREL. Sofinel. October 2008. (E)
9	9.5	SFLEZS030062	D	Stoehr S., Kokemueller D. Oxygen Gas Distribution System (SGO) P2 - System Operation. SFL-EZS-030062 Revision D BPE. Sofinel. December 2008. (E)
9	9.5	SFLEZS030065	D	Stoehr S., Kokemueller D. Oxygen Gas Distribution System (SGO) P3 – Sizing of the system and its components. SFL-EZS030065 Revision D BPE. Sofinel. December 2008. (E)

9	9.5	SFLEZS030057	E	Stoehr S., Kokemueller D. Oxygen Gas Distribution System SGO P4 - Flow diagrams. SFL-EZS 030057 Revision E BPE. Sofinel. December 2008. (E)
9	9.5	EZS2007en0011	C	Stoehr S., Kokemueller D. Oxygen Gas Distribution System (SGO) P5 - Instrumentation and Control. EZS/2007/en/0011 Revision C BPE. Sofinel. October 2008. (E)
9	9.5	SFLEZS030061	D	Stoehr S., Kokemueller D. Hydrogen Gas Distribution System (SGH) P2 - System Operation. SFL-EZS-030061 Revision D PREL. Sofinel. December 2008. (E)
9	9.5	SFLEZS030064	D	Stoehr S., Kokemueller D. Hydrogen Gas Distribution System (SGH) P3 – Sizing of the system and its components. SFL-EZS030064 Revision D PREL. Sofinel. December 2008. (E)
9	9.5	SFLEZS030058	D	Stoehr S., Kokemueller D. Hydrogen Gas Distribution System (SGH) P4 - Flow diagrams. SFL-EZS 030058 Revision D PREL. Sofinel. December 2008. (E)
9	9.5	EZS2007en0009	C	Stoehr S., Kokemueller D. Hydrogen Gas Distribution System (SGH) P5 – Instrumentation and Control. EZS/2007/en/0009 Revision C BPE. Sofinel. October 2008. (E)
9	9.5	SFLEZS030075	A	System Design Manual - Nuclear Island Hydrogen Detection System (KRH), Part 2 – System Operation, SFL-EZS-030075 Revision A. SOFINEL. September 2006. (E)
9	9.5	SFLEZS030076	A	System Design Manual - Nuclear Island Hydrogen Detection System (KRH), Part 3 – Sizing of the System and its Components, SFL-EZS-030076 Revision A. SOFINEL. September 2006. (E)
CHAPTER 10				
SUB-CHAPTER 10.3 – REFERENCES				
10	10.3	NESSFDC623	A	System Design Manual – Main Steam Supply System (VVP [MSSS]) P1]. NESS-F DC 623 Revision A. AREVA. December 2009. (E)
10	10.3	NESSFDC578	A	System Design Manual - Main Steam Supply System (VVP [MSSS]) - Part 2: System Operation. NESS-F DC 578 Revision A. AREVA. November 2009. (E)
10	10.3	NESSFDC579	A	System Design Manual - Main Steam Supply System (VVP [MSSS]) - Part 3: System Design. NESS-F DC 579 Revision A. AREVA. November 2009. (E)
10	10.3	NESSFDC594	A	System Design Manual - Main Steam Supply System (VVP [MSSS]) - Part 4: Flow diagrams. NESS-F DC 594 Revision A. AREVA. September 2009. (E)
10	10.3	NESSFDC629	A	System Design Manual – Main Steam Supply System (VVP [MSSS]) P5 - Instrumentation and Control. NESS-F DC 629 Revision A. AREVA. March 2010. (E)
10	10.3	ECEMA101028	A	F Chavigny. Demonstration of the accessibility and controllability for in-service inspection of the structural integrity components. ECEMA101028 Revision A. EDF. April 2010. (E)
10	10.3	PEEMF110959	B	Ultrasonic Examination of MSL Girth Welds. PEEM-F 11.0959 Revision B. AREVA. March 2012. (E)

10	10.3	NEERF102070	D	Demonstration of integrity of High Integrity Components against fast fracture. Fracture Mechanic Analyses – Non Destructive Testing – Fracture Toughness. NEER-F 10.2070 Revision D. AREVA. August 2012. (E)
10	10.3	PEEOF120359	B	UK EPR - Avoidance of fracture demonstration of the Main Steam Isolation Valve. PEEO-F 12.0359 Revision B. AREVA. July 2012. (E)
10	10.3	PEEMF101134	D	UK Technical report on the manufacturing non destructive testing to be qualified. PEEM-F.101134 Revision D. AREVA. December 2010. (E)
10	10.3	NEEMF100179	E	UK EPR™ Avoidance of Fracture- Fracture mechanics and minimum fracture toughness aspects: Materials Data. NEEM-F 10 0179 Revision E. AREVA. November 2010. (E)
SUB-CHAPTER 10.4 – REFERENCES				
10	10.4	ETDOFC080022	A1	System Design Manual – Main Steam Bypass System (to condenser) (GCT) Part 1 – System operation. ETDOFC080022 Revision A1. EDF. September 2009. (E)
10	10.4	ETDOFC080023	A1	System Design Manual – Main Steam Bypass System (to condenser) (GCT) Part 2 – System sizing. ETDOFC080023 Revision A1. EDF. September 2009. (E)
10	10.4	ETDOFC080024	A1	System Design Manual – Main Steam Bypass System (to condenser) (GCT) Part 3 – Flow diagrams. ETDOFC080024 Revision A1. EDF. December 2009. (E)
10	10.4	ETDOFC080025	A1	System Design Manual – Main Steam Bypass System (to condenser) (GCT) Part 4 - Instrumentation and Control. ETDOFC080025 Revision A1. EDF. December 2009. (E)
10	10.4	PESSFDC87	A	System Design Manual - Turbine By Pass System, Part 5: Instrumentation & Control. PESS-F DC 87 Revision A. AREVA. June 2012. (E). PESS-F DC 87 Revision A is the English translation of NESS-F DC 398 Revision B.
10	10.4	ETDOFC060117	A1	System Design Manual – Motor driven Feedwater Pump (APA [MFWPS]). ETDOFC60117 Revision A1. EDF. December 2009. (E)
10	10.4	ETDOFC060129	A1	System Design Manual - Start-Up and Shutdown Feedwater System (AAD [SSS]) - 1st Stage. ETDOFC060129 Revision A1. EDF. January 2010. (E)
10	10.4	ETDOFC050062	B1	CRF (Water Circulation) - EPR Operating Manual. ETDOFC050062 Revision B1. EDF. February 2011. Revision B1 is the English translation of ETDOFC050062 Revision B.
10	10.4	ETDOIG060274	D	Dossier de Système Elémentaire - CRF, P4.2 Schémas Mécaniques Detaille. [System Design Manual – Circulating Water System (CRF), Part 4.2 - Detailed flow diagrams] ETDOIG060274 Revision D. EDF. September 2008.
10	10.4	ETDOFC080077	C1	System Design Manual – Circulating Water System (CRF), Part 5 – Instrumentation and Control. ETDOFC080077 Revision C1. EDF. September 2009. (E)
10	10.4	SFLEZS030046	G	System Design Manual – Steam Generator Blowdown System (APG [SGBS]) P2 –System Operation. SFL-EZS-030046 Revision G. Sofinel. December 2008. (E)

10	10.4	SFLEZS030047	G	M Seiter and M Lenz. System Design Manual – Steam Generator Blowdown System (APG [SGBS]) P3 – Sizing of the system and its components. SFL-EZS-030047 Revision G. Sofinel. December 2008. (E)
10	10.4	SFLEZS030048	F	Steam Generator Blowdown System (APG [SGBS]) P4 - Flow Diagrams. SFL-EZS-030048 Revision F. SOFINEL. July 2008. (E)
10	10.4	ECEF081522	A	Dossier de Système Elémentaire – Système de Purge des Générateurs de Vapeur (APG), P4.1 – Schéma mécanique fonctionnel. [Steam Generator Blowdown System (APG [SGBS]) P4.1 - Functional Flow Diagrams. ECEF081522 Revision A. SOFINEL. July 2008
10	10.4	EZS2007en0013	E	M Seiter and M Lenz. System Design Manual – Steam Generator Blowdown System (APG [SGBS]) P4.2 - Detailed Flow Diagrams (SMD). EZS/2007/en/0013 Revision E. Sofinel. December 2008. (E)
10	10.4	SFLEZS030079	G	System Design Manual – Steam Generator Blowdown System (APG [SGBS]) P5 – Instrumentation and Control (I&C). SFL-EZS-030079 Revision G. Sofinel. November 2008. (E)
SUB-CHAPTER 10.5 – REFERENCES				
10	10.5	ECEMA040920	C1	Application of the break preclusion assumption in the main reactor coolant and steam lines of the EPR FA3. ECEMA040920 Revision C1. EDF. (E)
10	10.5	ENSNDR080245	A	Break Preclusion in reactor main coolant lines and main steam lines. Positioning of the concept and associated safety requirements . ENSNDR080245 Revision A. EDF. (E)
10	10.5	NESPFDC74	D	Lepeytre. Specification for procurement of seamless P355NH steel pipes of Q1 level – Main VVP (MSSS) and ARE (MFWS) lines. NESP-F DC 74 Revision D. AREVA. May 2008. (E)
10	10.5	NESPFDC75	D	Lepeytre. Specification for procurement of P355NH steel elbows and fittings (forged or die formed) of Q1 level – Main VVP (MSSS) and ARE (MFWS) lines. NESP-F DC 75 Revision D. AREVA. May 2008. (E)
SUB-CHAPTER 10.6 – REFERENCES				
10	10.6	NESSFDC576	A	System Design Manual - Main Feedwater System (ARE [MFWS]) - Part 2: System Operation. NESS-F DC 576 Revision A. AREVA. September 2009. (E)
10	10.6	ETDOFC060117	A1	System Design Manual - First stage system design manual for the motor-driven feedwater pump system (APA[MFWPS]). ETDOFC060117 Revision A1. EDF. December 2009. (E)
10	10.6	ETDOFC060129	A1	System Design Manual - Start-Up and Shutdown Feedwater System (AAD [SSS]) - 1st Stage. ETDOFC060129 Revision A1. EDF. January 2010. (E)
10	10.6	NESSFDC577	A	System Design Manual - Main Feedwater System - Part 3: System Design. NESS-F DC 577 Revision A. AREVA. September 2009. (E)

10	10.6	NESSFDC593	A	System Design Manual - Main Feedwater System - Part 4: Flow diagrams NESS-F DC 593 Revision A. AREVA. September 2008. (E)
10	10.6	NESSFDC611	A	System Design Manual – Main Steam Generator Feedwater System (ARE [MFWS]) P5 - Instrumentation and Control. NESS-F DC 611 Revision A. AREVA. December 2009. (E)
CHAPTER 11				
SUB-CHAPTER 11.0 – REFERENCES				
11	11.0	ISBN0105443905		Environmental Protection Act 1990. HM Stationery Office. November 1990, reprinted January 2004. ISBN 0105443905.
11	11.0	ISBN9780105457916		Water Resources Act 1991 . ISBN 978-010545791-6. The Stationery Office Ltd. (E)
11	11.0	ISBN0105424994		The Pollution Prevention and Control Act 1999 (as amended). HM Stationery Office. August 1999. ISBN 0105424994.
11	11.0	ISBN0105412937		The Radioactive Substances Act 1993. HM Stationery Office. ISBN 0-10: 0105412937.
11	11.0	ISBN0108502163		The Nuclear Installations Act 1965 (as amended). HM Stationery Office. ISBN 0108502163
11	11.0	ISBN9780105425953		Environment Act 1995. ISBN 978-010542595-3. The Stationery Office Ltd. (E)
11	11.0	ISBN 0105427918		Radioactive Material (Road Transport) Act 1991. HM Stationery Office. July 1991. ISBN 0105427918.
11	11.0	REF0006		Radioactive Substances (Basic Safety Standards) (England and Wales) Direction 2000. DEFRA.
11	11.0	ISBN0110856147		The Ionising Radiations Regulations 1999. Statutory Instrument 1999 No. 3232. HM Stationery Office. ISBN 0-11-085614-7.
11	11.0	ISBN0110774698		The Carriage of Dangerous Goods and Use of Transportable Pressure Equipment Regulations 2007. HM Stationery Office. July 2007. ISBN 0110774698.
11	11.0	1999829EURATOM		Treaty establishing the European Atomic Energy Community (1957). EUR-Lex. See also COMMISSION RECOMMENDATION of 6 December 1999 on the application of Article 37 of the Euratom Treaty (1999/829/Euratom). (E)
11	11.0	TAST024		UK Health and Safety Executive (HSE). Technical Assessment Guide, Management of Radioactive Materials and Radioactive Waste on Nuclear Licensed Sites. T/AST/024 Issue 003. March 2001. (E)
11	11.0	9723EC		Directive 97/23/EC of the European Parliament and of the Council of 29 May 1997 on the approximation of the laws of the Member States concerning pressure equipment. (E)
SUB-CHAPTER 11.1 – REFERENCES				
11	11.1	REF0003		Process and Information Document for Generic Assessment of Candidate Nuclear Power Plant Designs. The Environment Agency. January 2007. (E)

11	11.1	ENTERP090062	A	Primary Source Term of the EPR Reactor. ENTERP090062 Revision A. EDF. March 2009. (E) ENTERP090062 Revision A is the English translation of ENTERP070147 Rev A.
11	11.1	ECEIG101686	B	Use of source term in the different GDA areas. ECEIG101686 Revision B. EDF. November 2010. (E)
11	11.1	ECEF110139	A	Zinc Injection claims, arguments and evidences: overall balance for UK-EPR. ECEF110139 Revision A. EDF. March 2011 (E)
11	11.1	ECEF110448	A	Analysis of UK EPR™ Source Term: Identification, Quantification and Characterisation. ECEF110448 Revision A. EDF. July 2011. (E)
11	11.1	ECEF111022	A	Corrosion product characterization under PWR/EPR primary coolant conditions: thermodynamic assessments and power plant feedback. ECEF111022 Revision A. EDF. July 2011. (E)
SUB-CHAPTER 11.2 – REFERENCES				
11	11.2	REF0003		Process and Information Document for Generic Assessment of Candidate Nuclear Power Plant Designs. The Environment Agency. January 2007. (E)
11	11.2	SFLEZS030008	G	System Design Manual – Coolant Purification, Degasification, Storage and Treatment System (TEP [CSTS]), P2 – System Operation. SFL-EZS-030008 Revision G. Sofinel. October 2008. (E)
11	11.2	SFLEZS030009	F	System Design Manual – Coolant Purification, Degasification, Storage and Treatment System (TEP [CSTS]) P3 – Sizing of the system and its components. SFL-EZS030009 Revision F. Sofinel. October 2008. (E)
11	11.2	SFLEZS030010	E	System Design Manual – Coolant Purification, Degasification, Storage and Treatment System (TEP [CSTS]) P4 – Flow diagrams. SFL-EZS030010 Revision E. SOFINEL. September 2008. (E)
11	11.2	EZS2006en0008	A	System Design Manual – Coolant Purification, Degasification, Storage and Treatment System (TEP [CSTS]) P4.1 – Functional flow diagrams. EZS2006/en/0008 Revision A. SOFINEL. October 2006. (E)
11	11.2	EZS2006en0009	F	System Design Manual – Coolant Purification, Degasification, Storage and Treatment System (TEP [CSTS]) P4.2 – Detailed flow diagrams. EZS/2006/en/0009 Revision F. Sofinel. October 2008. (E)
11	11.2	SFLEZS030077	F	System Design Manual – Coolant Purification, Degasification, Storage and Treatment System (TEP [CSTS]) P5 – Instrumentation and Control. SFL-EZS-030077 Revision F. Sofinel. November 2008. (E)
11	11.2	DARPEFLABII3		DARPE FLA. Authorization request for water intake and liquid and gaseous releases of the Flamanville site. Part B. Chapter II.3 - Gaseous and liquid radioactive releases. (E)
11	11.2	SFLEZS030046	G	System Design Manual – Steam Generator Blowdown System (APG [SGBS]) P2 –System Operation. SFL-EZS-030046 Revision G. Sofinel. December 2008. (E)
11	11.2	SFLEZS030047	G	System Design Manual – Steam Generator Blowdown System (APG [SGBS]) P3 – Sizing of the system and its components. SFL-EZS-030047 Revision G. Sofinel. December 2008. (E)

11	11.2	ETDOIG060270	A1	System Design Manual – Specifications for describing the (SEK [CILWDS]) plant system in the turbine hall and site galleries in the Flamanville 3 EPR. ETDOIG060270 Revision A1. EDF. November 2009. (E)
11	11.2	23952SEK00014SH	C	Dossier de Système Elémentaire – SEK DSE P4.2: Schéma mécanique détaillé. [System Design Manual – Conventional Island liquid waste discharge system (SEK [CILWDS]), Part 4.2 – Detailed Flow Diagrams] 23952SEK00014SH Revision C. Assystem. September 2008.
11	11.2	23952SEK00025DSE	C1	System Design Manual – Conventional Island liquid waste discharge system (SEK [CILWDS]), Part 5 – Instrumentation and Control. 23952SEK00025DSE Revision C1. Assystem. December 2008. (E)
11	11.2	SFLEZS030050	D	System Design Manual - Gaseous Waste Processing System (TEG [GWPS]), P1 - History of the System Manual. SFL–EZS 030050 Revision D. Sofinel. October 2008. (E)
11	11.2	SFLEZS030051	E	System Design Manual - Gaseous Waste Processing System (TEG[GWPS]), P2 – System operation. SFLEZS030051 Revision E. Sofinel. September 2008. (E)
11	11.2	SFLEZS030052	E	System Design Manual - Gaseous Waste Processing System (TEG[GWPS]), P3 – Sizing of the System and its Components. SFLEZS030052 Revision E. Sofinel. September 2008. (E)
11	11.2	SFLEZS030053	D	System Design Manual - Gaseous Waste Processing System (TEG[GWPS]) P4 – Flow diagrams. SFL–EZS 030053 Revision D. Sofinel. September 2008. (E)
11	11.2	ECEF080592	A	System Design Manual - Gaseous Waste Processing System (TEG[GWPS]) P4.1 – Functional flow diagrams. ECEF080592 Revision A. EDF. April 2008. (E)
11	11.2	EZS2007en0007	D	System Design Manual - Gaseous Waste Processing System (TEG[GWPS]) P4.2 – Detailed flow diagrams. EZS/2007/en/0007 Revision D. Sofinel. September 2008. (E)
11	11.2	EZS2007en0015	C	System Design Manual - Gaseous Waste Processing System (TEG[GWPS]) P5 – Instrumentation and Control. EZS 2007/en/0015 Revision C. Sofinel. October 2008. (E)
11	11.2	EZL2006EN0093	E	System Design Manual – Nuclear Auxiliary Building Ventilation System (DWN [NABVS]), P2 – System operation. EZL2006/en/0093 Revision E. Sofinel. November 2008. (E)
11	11.2	EZL2006EN0092	F	System Design Manual – Nuclear Auxiliary Building Ventilation System (DWN [NABVS]), P3 - Sizing Report. EZL2006/en/0092 Revision F. Sofinel. November 2008. (E)
11	11.2	SFLEFMF2006164	D1	System Design Manual – Fuel Building Ventilation System (DWK [FBVS]), P2 - System Operation. SFL-EF MF 2006.164 Revision D1. Sofinel. October 2009. (E)
11	11.2	SFLEZL030008	G	System Design Manual – Safeguard Building Controlled Area Ventilation System (DWL [CSBVS)], P2 – System operation (Stage 1)SFL-EZL-030008 Revision G. SOFINEL. November 2008. (E)
11	11.2	EHZ2007EN0033	E	System Design Manual – Safeguard Building Controlled Area Ventilation System (DWL [CSBVS]) P3 – Sizing of the system and its components (Stage 1). EZH/2007/en/0033 Revision E. Sofinel. November 2008. (E)

11	11.2	EYTS2007FR0131	B1	System Design Manual – Containment Sweep Ventilation System (EBA [CSVS]) P2 - System Operation. EYTS2007/fr/0131 Revision B1. Sofinel. November 2009. (E)
11	11.2	SFLEFMF2006502	B1	System Design Manual – Containment Sweep Ventilation System (EBA [CSVS]), P3 – Design of System and its Components. SFL-EFMF-2006.502 Revision B1. Sofinel. March 2009. (E)
11	11.2	SFLEFMF2006131	D1	System Design Manual – Operating Building Contaminable Room Ventilation System (8DWB), P2 - System Operation (Stage 2). SFL-EF MF 2006.131 Revision D1. Sofinel. August 2009. (E)
11	11.2	SFLEFMF2006132	E1	System Design Manual – Operating Building Contaminable Room Ventilation System (8DWB), P3 - Design of System and Components (Stage 2). SFL-EF MF 2006.132 Revision E1. Sofinel. August 2009. (E)
11	11.2	EYTS2007FR0108	B1	System Design Manual – Effluent Treatment Building Ventilation System (8DWQ) P2 - System Operation. EYTS2007/fr/0108 Revision B1. Sofinel. November 2009. (E)
11	11.2	EYTS2007FR0109	B1	System Design Manual – Effluent Treatment Building Ventilation System (8DWQ) P3 - System and component design. EYTS2007/fr/0109 Revision B1. Sofinel. November 2009. (E)
11	11.2	EYTS2007FR0187	D1	System Design Manual – Peripheral Room Ventilation System (DWW [ABVS]) P2 - System Operation. EYTS2007/fr/0187 Revision D1. Sofinel. November 2009. (E)
11	11.2	DARPEFLABII41		DARPE FLA. Authorization request for water intake and liquid and gaseous releases of the Flamanville site. Part B. Chapter II.4.1 - Chemicals associated with liquid radioactive effluents and drainage water discharged into the sea. (E)
11	11.2	UKEPR0011001	6	GDA UK EPR - BAT Demonstration. UKEPR-0011-001 Issue 06. EDF/AREVA. August 2012. (E)
11	11.2	UKEPR0010001	3	GDA UK EPR – Integrated Waste Strategy Document. UKEPR-0010-001 Issue 03. EDF/AREVA. June 2012. (E)
11	11.2	UKEPR0012001	2	GDA UK EPR - Mapping Document for Radioactive Waste Management Case. UKEPR-0012-001 Issue 02. EDF/AREVA. March 2011. (E)
11	11.2	NESHG2008en0123	A	Solid Radioactive Waste Strategy Report (SRWSR). AREVA NP. NESH-G/2008/en/0123 Revision A. November 2008. (E)
11	11.2	GUIDANCE2010		Guidance from the Health and Safety Executive, the Environment Agency and the Scottish Environment Protection Agency to nuclear licensees . The management of higher activity radioactive waste on nuclear licensed sites, radioactive waste management cases. February 2010. (E)
11	11.2	REF0016		Policy for the Long Term Management of Solid Low Level Radioactive Waste in the United Kingdom. DEFRA, March 2007. (E)
11	11.2	ECUK110016App4	A	EPR - Radioactive waste conditioning - ECUK110016 Revision A. Appendix 4. EDF. January 2011. (E)
11	11.2	ECEF080440	A1	System Design Manual – Effluent Treatment Building Solid Waste Treatment System (8TES [SWTS]) P1. ECEF080440 Revision A1. EDF. January 2010. (E)
11	11.2	ECEF070007	B1	System Design Manual – Effluent Treatment Building Solid Waste Treatment System (8TES [SWTS]) P2 – System operation. ECEF070007 Revision B1. EDF. January 2010. (E)

11	11.2	ECEF070008	B1	System Design Manual – Effluent Treatment Building Solid Waste Treatment System (8TES [SWTS]), P3 – System dimensioning. ECEF070008 Revision B1. EDF. January 2010. (E)
11	11.2	ECEF080573	A1	System Design Manual – Effluent Treatment Building Solid Waste Treatment System (8TES [SWTS]), P4 – Flow diagrams. ECEF080573 Revision A1. EDF. January 2010. (E)
11	11.2	ECEF080624	A	Dossier de Système Elémentaire - Traitement des effluents solides du BTE 8TES [SWTS] – Part 4.1 – Schéma Mécanique Fonctionnel. [System Design Manual - Effluent Treatment Building Solid Waste Treatment System 8TES [SWTS], P4.2 – Functional Flow Diagrams]. ECEF080624 Revision A. EDF. April 2008.
11	11.2	ECEF071498	B	Dossier de Système Elémentaire - Traitement des effluents solides du BTE 8TES [SWTS]– Part 4.2 - Schéma Mécanique Detaille [System Design Manual - Effluent Treatment Building Solid Waste Treatment System 8TES [SWTS], P4.2 - Detailed Flow Diagrams]. ECEF071498 Revision B. EDF. January 2008.
11	11.2	ECEF080380	A1	System Design Manual – Effluent Treatment Building Solid Waste Treatment System (8TES [SWTS]), P5 – Instrumentation and Control. ECEF080380 Revision A1. EDF. January 2010. (E)
11	11.2	SFLEZM200118	B	System Design Manual – Solid Waste Treatment System (TES [SWTS]) – Filter changing equipment TES11 P2 – System operation. SFLEZM200118 Revision B. Sofinel. September 2006. (E)
11	11.2	SFLEZM200119	B	System Design Manual – Solid Waste Treatment System (TES [SWTS]) – Filter changing equipment TES11 P3 – Sizing of the system and its components. SFLEZM200119 Revision B. Sofinel. September 2006. (E)
11	11.2	SFLEZS030039	D	System Design Manual – Solid Waste Treatment System (TES [SWTS]) – Transfer of the spent resins (TES21, TES31) P1. SFLEZS030039 Revision D. Sofinel. March 2007. (E)
11	11.2	SFLEZS030040	D	System Design Manual – Solid Waste Treatment System (TES [SWTS])– Transfer of the spent resins (TES21, TES31) P2 – System operation. SFL-EZS030040 Revision D. Sofinel. March 2007. (E)
11	11.2	SFLEZS030041	D	System Design Manual – Solid Waste Treatment System (TES [SWTS])– Transfer of the spent resins (TES21, TES31) P3 – Sizing of the system and its components. SFL-EZS030041 Revision D. Sofinel. March 2007. (E)
11	11.2	SFLEZS030042	D	System Design Manual – Solid Waste Treatment System (TES [SWTS]) – Transfer of the spent resins (TES21, TES31) P4 – Flow diagrams. SFL-EZS030042 Revision D. Sofinel. March 2007. (E)
11	11.2	EZS2006en0019	A	System Design Manual – Solid Waste Treatment System (TES [SWTS]) – Transfer of the spent resins (TES21, TES31), P4.1 – Simplified flow diagrams. EZS/2006/en/0019 Revision A. SOFINEL. March 2007. (E)
11	11.2	EZS2006en0020	B	System Design Manual – Solid Waste Treatment System (TES [SWTS]) – Transfer of the spent resins (TES21, TES31), P4.2 – Detailed flow diagrams. EZS/2006/en/0020 Revision B. SOFINEL. March 2007. (E)
11	11.2	320L027		Form D1 Application: UK EPR Project, from LLW Repository Ltd. 320.L.027. December 2008. (E)
11	11.2	ECUK110016App1	A	EPR - Radioactive waste conditioning - ECUK110016 Revision A. Appendix 1. EDF. January 2011. (E)
11	11.2	CoRWM700		Managing our Radioactive Waste Safely - CoRWM's Recommendations to Government. CoRWM Document 700. July 2006. (E)

SUB-CHAPTER 11.3 – REFERENCES				
11	11.3	REF0003		Process and Information Document for Generic Assessment of Candidate Nuclear Power Plant Designs. The Environment Agency. January 2007. (E)
11	11.3	C20034832		Commission recommendation of 18 December 2003 on standardised information on radioactive airborne and liquid discharges into the environment from nuclear power reactors and reprocessing plants in normal operation. C(2003)4832, 2004/2/Euratom. Official Journal of the European Union. 6 January 2004. (E)
11	11.3	UKEPR0011001	6	GDA UK EPR - BAT Demonstration. UKEPR-0011-001 Issue 06. EDF/AREVA. August 2012. (E)
11	11.3	NESHG2008en0123	A	Solid Radioactive Waste Strategy Report (SRWSR). NESH-G/2008/en/0123 Rev A. AREVA NP. November 2008. (E)
11	11.3	ECMT050107	A1	Projected assessment of solid radioactive waste from the EPR. ECMT050107 Revision A1. EDF. November 2009. (E) ECMT050107 Revision A1 is the English translation of ECMT050107 Revision A.
11	11.3	D450711001253	0	B Lantes. EPR waste arising over the operating life. D450711001253 Version 0. EDF. March 2011 (E)
11	11.3	SFLEZM200118	B	System Design Manual – Solid Waste Treatment System (TES [SWTS]) – Filter changing equipment TES11 P2 – System operation. SFL-EZM200118 Revision B. Sofinel. September 2006. (E)
11	11.3	SFLEZM200119	B	System Design Manual – Solid Waste Treatment System (TES [SWTS]) – Filter changing equipment TES11 P3 – Sizing of the system and its components. SFL-EZM200119 Revision B. Sofinel. September 2006. (E)
11	11.3	SFLEZS030040	D	System Design Manual – Solid Waste Treatment System (TES [SWTS])– Transfer of the spent resins (TES21, TES31) P2 – System operation. SFL-EZS030040 Revision D. Sofinel. March 2007. (E)
11	11.3	SFLEZS030041	D	System Design Manual – Solid Waste Treatment System (TES [SWTS])– Transfer of the spent resins (TES21, TES31) P3 – Sizing of the system and its components. SFL-EZS030041 Revision D. Sofinel. March 2007. (E)
11	11.3	SFLEZS030039	D	System Design Manual – Solid Waste Treatment System (TES [SWTS]) – Transfer of the spent resins (TES21, TES31) P1. SFL-EZS030039 Revision D. Sofinel. March 2007. (E)
11	11.3	SFLEZS030042	D	System Design Manual – Solid Waste Treatment System (TES [SWTS]) – Transfer of the spent resins (TES21, TES31) P4 – Flow diagrams. SFL-EZS030042 Revision D. Sofinel. March 2007. (E)
11	11.3	EZS2006en0019	A	System Design Manual – Solid Waste Treatment System (TES [SWTS]) – Transfer of the spent resins (TES21, TES31), P4.1 – Simplified flow diagrams. EZS/2006/en/0019 Revision A. SOFINEL. March 2007. (E)
11	11.3	EZS2006en0020	B	System Design Manual – Solid Waste Treatment System (TES [SWTS]) – Transfer of the spent resins (TES21, TES31), P4.2 – Detailed flow diagrams. EZS/2006/en/0020 Revision B. SOFINEL. March 2007. (E)
11	11.3	UKEPR0017001	0	Decontamination Processes and Techniques for the UKEPR. UKEPR-0017-001 Issue 00. EDF/AREVA. March 2011. (E)
11	11.3	ISODIS21238		Nuclear Energy - Nuclear Fuel Technology - The Scaling Factor method to determine the radioactivity of low and intermediate level radioactive waste packages generated at nuclear power plant. ISO/DIS 21238. (E)

11	11.3	ENTERP090190		Methods for limiting the production and discharge of tritium for the EPR. ENTERP090190. EDF. August 2009. (E) ENTERP090190 is the English translation of ENTERP040216 Revision A
11	11.3	ECEP050315	A1	Analysis of environmental performance in the EPR France project. ECEP050315 Revision A1. EDF. March 2012. (E) ECEP050315 Revision A1 is the English translation of ECEP050315 Revision A.
11	11.3	ECEF110138	A	Zinc Injection Implementation at UK-EPR. ECEF110138. Revision A. EDF. March 2011 (E)
11	11.3	ECEF110139	A	Zinc Injection claims, arguments and evidences: overall balance for UK-EPR. ECEF110139 Revision A. EDF. March 2011 (E)
11	11.3	ECEF110448	A	Analysis of UK EPR™ Source Term: Identification, Quantification and Characterisation. ECEF110448 Revision A. EDF. June 2011. (E)
11	11.3	ECEF110449	A	Activity management at UK-EPR Auxiliary Systems : system performances and control actions. ECEF110449 Revision A. EDF. June 2012. (E)
11	11.3	ENTERP090183		EPR – Production and discharge of C14. ENTERP090183. EDF. August 2009. (E) ENTERP090183 is the English translation of ENTERP050168 Revision C.
11	11.3	DARPEFLACIV1TABLEC		DARPE FLA (Authorization request for water intake and liquid and gaseous releases of the Flamanville site) Part C. Chapter IV-1 Table C-IV.1.1-j - Radionuclides and activities discharged considered in the maximum ("requested limits") gaseous discharge calculations for the EPR unit. (E)
11	11.3	ECEF060592	A1	DARPE FLA (Authorization request for water intake and liquid and gaseous releases of the Flamanville site) Annexe B5e. Expected results, excluding hazards of discharges of liquid chemical substances associated with radioactive effluents and waste water from the machine rooms for the EPR nuclear reactor. ECEF060592 Revision A1. EDF. May 2009. (E) ECEF060592 Revision A1 is the English translation of ECEF060592 Revision A
11	11.3	ECEF050301	C1	DARPE FLA (Authorization request for water intake and liquid and gaseous releases of the Flamanville site) Annexe B5c. ECEF050301 Revision C1. EDF. May 2009. (E) ECEF050301 Revision C1 is the English translation of ECEF050301 Revision C.
11	11.3	DARPEFLABIII4		DARPE FLA (Authorization request for water intake and liquid and gaseous releases of the Flamanville site) Part B Chapter III.4 Table B-III.4.1.4.a - Expected performance excluding contingencies and maximum annual additional discharges for chemicals associated with radioactive effluents. (E)
11	11.3	EDEAPC050176	D1	DARPE FLA (Authorization request for water intake and liquid and gaseous releases of the Flamanville site) Annexe B5d. EDEAPC050176 Revision D1. EDF. November 2009. (E) EDEAPC050176 Revision D1 is the English translation of EDEAPC050176 Revision D.
11	11.3	EDEAPC050094	D1	DARPE FLA (Authorization request for water intake and liquid and gaseous releases of the Flamanville site) Annexe B5a. EDEAPC050094 Revision D1. EDF. November 2009. (E) EDEAPC050094 Revision D1 is the English translation of EDEAPC050094 Revision D.
11	11.3	DARPEFLABIII42		DARPE FLA (Authorization request for water intake and liquid and gaseous releases of the Flamanville site) Part B Chapter III.4.2~~-2~~ - Other chemicals discharged into the sea - and associated annexes. (E)

SUB-CHAPTER 11.4 – REFERENCES				
11	11.4	REF0003		Process and Information Document for Generic Assessment of Candidate Nuclear Power Plant Designs. The Environment Agency. January 2007. (E)
11	11.4	ECUK110016App4	A	EPR - Radioactive waste conditioning - ECUK110016 Revision A. Appendix 4. EDF. January 2011. (E)
11	11.4	ECUK110016App3	A	EPR - Radioactive waste conditioning - ECUK110016 Revision A. Appendix 3. EDF. January 2011. (E)
11	11.4	SFLEZS030018	F	System Design Manual – Nuclear Island Vent and Drain System (RPE [NVDS]) - P2 - System Operation. BNI - SFLEZS030018 Revision F. Sofinel. October 2008. (E)
11	11.4	NESSFDC608	A	System Design Manual – Nuclear Island Vent and Drain System (RPE [NVDS]) - P2 - System Operation. RB - NESS-F DC 608 Revision A. AREVA. November 2009. (E)
11	11.4	EYTF2008FR0010	A1	System Design Manual – Nuclear Island Vent and Drain System (8RPE [ETBVDS]) P2 - System Operation. ETB - EYTF2008/fr/0010 Revision A1. EDF. September 2009. (E)
11	11.4	SFLEZS030019	F	System Design Manual - Nuclear Island Vent and Drain System (RPE [NVDS]) - P3 – Sizing of the System and its Components. BNI - SFLEZS030019 Revision F. Sofinel. July 2008. (E)
11	11.4	NESSFDC609	A	System Design Manual - Nuclear Island Vent and Drain System (RPE [NVDS]) P3 – Sizing of the System and its Components. RB - NESS-F DC 609 Revision A. AREVA. November 2009. (E)
11	11.4	SFLEZS030020	D	System Design Manual - Nuclear Island Vent and Drain System (RPE [NVDS]) P4 – Flow Diagrams. BNI - SFLEZS030020 Revision D. SOFINEL. October 2007. (E)
11	11.4	EZS2007EN0021	A	System Design Manual - Nuclear Island Vent and Drain System (RPE [NVDS]) - P4.1 : Functional Flow Diagrams, BNI - EZS2007EN0021. Revision A. Sofinel. October 2007. (E)
11	11.4	EZS2007EN0022	C	System Design Manual - Nuclear Island Vent and Drain System (RPE [NVDS]) - P4.2 : Detailed Flow Diagrams : BNI - EZS2007EN0022 Revision C. Sofinel. November 2008. (E)
11	11.4	NESSFDC610	A	System Design Manual - Nuclear Island Vent and Drain System (RPE [NVDS]) P4 – Flow Diagrams. RB - NESS-F DC 610 Revision A. AREVA. November 2009. (E)
11	11.4	EYTF2007FR0044	C1	System Design Manual - Effluent Treatment Building Vent and Drain System (8RPE [ETBVDS]) P4 – Flow Diagrams. ETB - EYTF2007/fr/0044 Revision C1. EDF. October 2009. (E)
11	11.4	EYTF2007FR0043	C	System Design Manual - Effluent Treatment Building Vent and Drain System (8RPE [ETBVDS]) P4.2 – Detailed Flow Diagrams. ETB - EYTF2007/fr/0043 Revision C. EDF. October 2008.
11	11.4	SFLEZS030055	F	System Design Manual - Nuclear Island Vent and Drain System (RPE [NVDS]) - P5 – Instrumentation and Control. BNI - SFLEZS030055 Revision F. Sofinel. November 2008. (E)

11	11.4	NESSFDC616	A	System Design Manual - Nuclear Island Vent and Drain System (RPE [NVDS]) P5 – Instrumentation and Control. RB - NESS-F DC 616 Revision A. AREVA. November 2009. (E)
11	11.4	EYTF2008FR0047	B1	System Design Manual - Effluent Treatment Building Vent and Drain System (8RPE [ETBVDS]) P5 – Instrumentation and Control. ETB - EYTF2008/fr/0047 Revision B1. EDF. October 2009. (E)
11	11.4	ECEF070502	A1	System Design Manual – Liquid Waste Processing System (TEU [LWPS]) Part 2: Operation of the TEU [LWPS] EPR FA3 system ECEF070502 Revision A1. EDF. January 2010. (E)
11	11.4	ECEF070503	A1	System Design Manual – Liquid Waste Processing System (TEU [LWPS]) Part 3 : Design basis of the system and its components. ECEF070503 Revision A1. EDF. January 2010. (E)
11	11.4	ECEF070504	A1	System Design Manual – Liquid Waste Processing System (TEU [LWPS]) Part 4 : Mechanical Diagrams ECEF070504 Revision A1. EDF. January 2010. (E)
11	11.4	ECEF072208	A	Dossier Système Elémentaire - Traitement Des Effluents Uses, P4.1: Schéma Mécanique Fonctionnel. [System Design Manual – Liquid Waste Processing System (TEU [LWPS]) P4.1, Functional flow diagram]. ECEF072208 Revision A. EDF. November 2007.
11	11.4	ECEF071497	B	Dossier Système Elémentaire - Traitement Des Effluents Uses, P4.2: Schéma Mécanique Detaille. [System Design Manual – Liquid Waste Processing System (TEU [LWPS]) P4.2, Detailed flow diagram]. ECEF071497 Revision B. EDF. January 2008.
11	11.4	ECEF071379	A	Dossier de Système Elémentaire – TEU DSE P5: Contrôle-commande. [System Design Manual – Liquid Waste Processing System (TEU [LWPS]) P5 : Instrumentation and Control] ECEF071379 Revision A. EDF. December 2008.
11	11.4	EYTS2008fr0016	A1	System Design Manual – Effluent Treatment Building Sampling System (TEN) P1. EYTS2008/fr/0016 Revision A1. EDF. August 2009. (E)
11	11.4	EYTS2007fr0038	B1	System Design Manual – Effluent Treatment Building Sampling System (TEN) P2 - System Operation. EYTS2007/fr/0038 Revision B1. EDF. August 2009. (E)
11	11.4	EYTS2007fr0174	B1	System Design Manual - Effluent Treatment Building Sampling System (TEN) P3 – System and component design basis. EYTS2007/fr/0174 Revision B1. EDF. August 2009. (E)
11	11.4	EYTS2007fr0040	B1	System Design Manual - Effluent Treatment Building Sampling System (TEN) P4 – Mechanical Diagrams. EYTS2007/fr/0040 Revision B1. EDF. August 2009. (E)
11	11.4	EYTS2007fr0101	A	Dossier Système Elémentaire – Circuit d'échantillonnage des effluents du BTE (TEN), P4.1 - Schéma Mécanique Fonctionnel. [System Design Manual – Effluent Treatment Building Sampling System (TEN), P4.1 - Functional flow diagram]. EYTS2007/fr/0101 Revision A. EDF. November 2007.
11	11.4	EYTS2007fr0039	B	Dossier Système Elémentaire – Circuit d'échantillonnage des effluents du BTE (TEN), P4.2 - Schéma Mécanique Detaille. [System Design Manual – Effluent Treatment Building Sampling System (TEN), P4.2 - Detailed flow diagram.] EYTS2007/fr/0039 Revision B. EDF. November 2007.

11	11.4	DARPEFLABII3		DARPE FLA (Authorization request for water intake and liquid and gaseous releases of the Flamanville site) Part B Chapter II.3 - Gaseous and liquid radioactive releases. (E)
11	11.4	23952SEK00014SH	C	Dossier de Système Elémentaire – SEK DSE P4.2: Schéma mécanique détaillé [System Design Manual – Site Liquid Waste Discharge System (SEK [CILWDS]) Part 4.2 : Detailed Flow Diagrams] 23952 SEK 00014SH Revision C. Assystem. September 2008.
11	11.4	23952SEK00025DSE	C1	System Design Manual – Conventional Island Liquid Waste Discharge System (SEK [SiteLWDS]) P5 : Instrumentation and Control. 23952SEK00025DSE Revision C1. Assystem. December 2008. (E)
11	11.4	EYTS2007FR0228	B1	System Design Manual - Waste Building Component Cooling System (TRI) - Part 1 History. EYTS/2007/fr/0228 Revision B1. Sofinel. November 2009. (E)
11	11.4	EYTS2007FR0036	B1	System Design Manual - Waste Building Component Cooling System (TRI) - Part 2 System Operation. EYTS/2007/fr/0036 Revision B1. EDF. November 2009. (E)
11	11.4	EYTS2007FR0037	C1	System Design Manual - Waste Building Component Cooling System (TRI) - Part 3 System and component design basis. EYTS/2007/fr/0037 Revision C1. EDF. November 2009. (E)
11	11.4	EYTS2007FR0070	C1	System Design Manual - Waste Building Component Cooling System (TRI) - Part 4 Mechanical diagrams. EYTS/2007/fr/0070 Revision C1. EDF. November 2009. (E)
11	11.4	EYTS2007FR0069	C	Dossier Système Elémentaire – (TRI), P4.2 - Schéma Mécanique Detaille. [System Design Manual – Waste Building Component Cooling System (TRI), P4.2 - Detailed flow diagram.] EYTS2007/fr/0069 Revision C. EDF. August 2008.
11	11.4	EYTS2007FR0115	D1	System Design Manual - Waste Building Component Cooling System (TRI) - Part 5 Instrumentation and Control. EYTS/2007/fr/0115 Revision D1. EDF. November 2009. (E)
11	11.4	SFLEZS030050	D	System Design Manual - Gaseous Waste Processing System (TEG [GWPS]), P1 - History of the System Manual. SFL–EZS 030050 Revision D. Sofinel. October 2008. (E)
11	11.4	SFLEZS030051	E	System Design Manual - Gaseous Waste Processing System (TEG[GWPS]), P2 – System operation. SFLEZS030051 Revision E. Sofinel. September 2008. (E)
11	11.4	SFLEZS030052	E	System Design Manual - Gaseous Waste Processing System (TEG [GWPS]), P3 – System design. SFL–EZS 030052 Revision E. Sofinel. September 2008. (E)
11	11.4	SFLEZS030053	D	System Design Manual - Gaseous Waste Processing System (TEG [GWPS]) P4 – Flow diagrams. SFL–EZS 030053 Revision D. Sofinel. September 2008. (E)
11	11.4	ECEF080592	A	System Design Manual - Gaseous Waste Processing System (TEG [GWPS]) P4.1 – Functional flow diagrams. ECEF080592 Revision A. EDF. April 2008. (E)
11	11.4	EZS2007en0007	D	System Design Manual - Gaseous Waste Processing System (TEG [GWPS]) P4.2 – Detailed flow diagrams. EZS/2007/en/0007 Revision D. Sofinel. September 2008. (E)

11	11.4	EZS2007en0015	C	System Design Manual - Gaseous Waste Processing System (TEG [GWPS]), P5 – Instrumentation and Control EZS 2007/en/0015 Revision C. Sofinel. October 2008. (E)
11	11.4	ENTERP070291	A	EPR: activity concentrations in the TEP, REA, TEG, RPE, TEU and TES auxiliary systems. ENTERP070291 Revision A. EDF. November 2007. (E). ENTERP070291 Revision A is the English translation of ENTERP070070 Revision A.
11	11.4	SFLEZM200118	B	System Design Manual – Solid Waste Treatment System (TES [SWTS]) – Filter changing equipment TES11, P2 – System operation. SFLEZM200118 Revision B. Sofinel. September 2006. (E)
11	11.4	SFLEZM200119	B	System Design Manual – Solid Waste Treatment System (TES [SWTS]) – Filter changing equipment TES11, P3 – Sizing of the system and its components. SFLEZM200119 Revision B. Sofinel. September 2006. (E)
11	11.4	SFLEZS030039	D	System Design Manual – Solid Waste Treatment System (TES [SWTS]) – Transfer of the spent resins (TES21, TES31), P1. SFLEZS030039 Revision D. Sofinel. March 2007. (E)
11	11.4	SFLEZS030040	D	System Design Manual – Solid Waste Treatment System (TES [SWTS])– Transfer of the spent resins (TES21, TES31) P2 – System operation. SFL-EZS030040 Revision D. Sofinel. March 2007. (E)
11	11.4	SFLEZS030041	D	System Design Manual – Solid Waste Treatment System (TES [SWTS])– Transfer of the spent resins (TES21, TES31) P3 – Sizing of the system and its components. SFL-EZS030041 Revision D. Sofinel. March 2007. (E)
11	11.4	SFLEZS030042	D	System Design Manual – Solid Waste Treatment System (TES [SWTS]) – Transfer of the spent resins (TES21, TES31) P4 – Flow diagrams. SFL-EZS030042 Revision D. Sofinel. March 2007. (E)
11	11.4	EZS2006en0019	A	System Design Manual – Solid Waste Treatment System (TES [SWTS]) – Transfer of the spent resins (TES21, TES31), P4.1 – Simplified flow diagrams. EZS/2006/en/0019 Revision A. SOFINEL. March 2007. (E)
11	11.4	EZS2006en0020	B	System Design Manual – Solid Waste Treatment System (TES [SWTS]) – Transfer of the spent resins (TES21, TES31), P4.2 – Detailed flow diagrams. EZS/2006/en/0020 Revision B. SOFINEL. March 2007. (E)
11	11.4	NESHG2008en0123	A	Solid Radioactive Waste Strategy Report (SRWSR). AREVA NP. NESH-G/2008/en/0123 Rev A. November 2008. (E)
SUB-CHAPTER 11.5 – REFERENCES				
11	11.5	REF0003		Process and Information Document for Generic Assessment of Candidate Nuclear Power Plant Designs. The Environment Agency. January 2007. (E)
11	11.5	WASTEPLANTS		The required level of design of waste plants for new build reactors in the Generic Design Assessment. May 2009. (http://www.hse.gov.uk/newreactors/wasteplants.pdf) (E)
11	11.5	WASTEMAN2010		The management of higher activity radioactive waste on nuclear licensed sites, radioactive waste management cases, Guidance from the Health and Safety Executive, the Environment Agency and the Scottish Environment Protection Agency to nuclear licensees, February 2010. (http://www.hse.gov.uk/nuclear/wastemanage/RWM-part2.pdf) (E)

11	11.5	UKEPR0012001	2	GDA UK EPR - Mapping Document for Radioactive Waste Management Case. UKEPR-0012-001 Issue 02. EDF/AREVA. March 2011. (E)
11	11.5	ISBN0108502163		The Nuclear Installations Act 1965 (as amended). HM Stationery Office. ISBN 0108502163. (E)
11	11.5	ISBN0110856147		The Ionising Radiations Regulations 1999. Statutory Instrument 1999 No. 3232. HM Stationery Office. ISBN 0-11-085614-7. (E)
11	11.5	ISBN0110299086		The Radiation (Emergency Preparedness and Public Information) Regulations 2001 (REPPIR). Statutory Instrument 2001 No. 2975. ISBN 0-11-02990- 6.HM Stationery Office. The Stationery Office Ltd. (E)
11	11.5	ISBN0110856252		Management of Health and Safety at Work Regulations 1999. ISBN 0-11-085625-2. The Stationery Office Ltd. (E)
11	11.5	ISBN9780110742427		The Nuclear Reactors (Environmental Impact Assessment for Decommissioning) (Amendment) Regulations 2006 (EIADR) (SI 2006 No. 657). ISBN 978-0110742427. The Stationery Office Ltd. (E)
11	11.5	ISBN0105412937		The Radioactive Substances Act 1993. HM Stationery Office. ISBN 0-10: 0105412937. (E)
11	11.5	REF0017		Management of Radioactive Materials And Radioactive Waste On Nuclear Licensed Sites. (http://www.hse.gov.uk/nuclear/waste1.pdf). (E)
11	11.5	ISBN0110795997		Provision and Use of Work equipment Regulations 1998 (PUWER). ISBN 0-11--9599--7.The Stationery Office Ltd. (E)
11	11.5	ISBN0110795989		Lifting Operations and Lifting Equipment Regulations 1998. ISBN 0-11-079598-9. The Stationery Office Ltd. (E)
11	11.5	ISBN0110258320		The Personal Protective Equipment at Work Regulations 1992.ISBN 0-11-025832--. The Stationery Office Ltd. (E)
11	11.5	ISBN0110858360		The Pressure Systems Safety Regulations 2000 (PSSR). Statutory Instruments 2000 No. 1281, ISBN 0-11-085836-0. The Stationery Office Ltd. (E)
11	11.5	ISBN0110727665		Control of Major Accident Hazards (Amendment) Regulations 2005 (COMAH). ISBN 0--110-72766-5. The Stationery Office Ltd. (E)
11	11.5	ISBN0110429575		Dangerous Substances and Explosive Atmospheres Regulations 2002 (DSEAR). ISBN 0-11-042957-5. The Stationery Office Ltd. (E)
11	11.5	UKEPR0008001	1	ILW Interim Storage Facility. UKEPR-0008-001 Issue 01. EDF/AREVA. November 2009. (E)
11	11.5	R10006A	2	Human Factors in Long Term Waste Management. R10-006 (A) Issue 2. AREVA. February 2010. (E)
11	11.5	ELIDC0902019	B	Plan for the Development of Waste Management Facilities over the EPR Lifetime. ELIDC0902019 Revision B. EDF. March 2010. (E)
11	11.5	ELI0800226	A	Dry Interim Storage facility for ILW. ELI0800226 Rev A. November 2008. (E)
11	11.5	NESHG2008en0123	A	Solid Radioactive Waste Strategy Report (SRWSR). AREVA NP. NESH-G/2008/en/0123 Rev A. November 2008. (E)
11	11.5	UKEPR0016001	1	GDA UK EPR Decommissioning. UKEPR-0016-001 Issue 01. EDF/AREVA. March 2011. (E)
11	11.5	UKEPR0009001	1	Spent Fuel Interim Storage Facility. UKEPR-0009-001 Issue 01. EDF/AREVA. November 2009. (E)
11	11.5	SGNNT100330200004	A	Maintenance of Interim Storage Facilities. SGN NT 100330 20 0004 Revision A. AREVA. February 2010. (E)

11	11.5	ELI0800224	A	Interim storage facility for spent fuel assemblies coming from an EPR plant. ELI0800224 Revision A. EDF. November 2008. (E)
11	11.5	ELIDC1000094	A	The safety of long-term spent fuel storage in ponds. ELIDC1000094 Revision A. EDF. April 2010. (E)
11	11.5	NT10033020005	A	Current French and International Research and Development Programmes for Interim Storage of Spent Fuel. SGN NT 100330 20 005 Revision A. AREVA. February 2010. (E)
11	11.5	320L027		Form D1 Application: UK EPR Project, from LLW Repository Ltd. 320.L.027. December 2008. (E)
11	11.5	REF0018		Disposability Assessment of Solid Waste Arisings from New Build. NDA, Radioactive Waste Management Directorate. April 2008. (E)
11	11.5	TN11261814		Generic Design Assessment: Summary of Disposability Assessment for Wastes and Spent Fuel arising from Operation of the UK EPR. NDA. Radioactive Waste Management Directorate. Technical Note no. 11261814 . (https://www.nda.gov.uk/documents/upload/TN-17548-Generic-Design-Assessment-Summary-of-Disposability-Assessment-for-Wastes-and-Spent-Fuel-arising-from-Operation-of-the-EPWR.pdf.). October 2009. (E)
11	11.5	NXA10747397	2	Generic Design Assessment: Disposability Assessment for Wastes and Spent Fuel arising from Operation of the UK EPR. Part 1: Main Report. NXA/10747397 Issue 2. January 2010. (E)
11	11.5	NXA10777960	2	Generic Design Assessment: Disposability Assessment for Wastes and Spent Fuel arising from Operation of the UK EPR. Part 2: Data Sheets and Inventory Tables. NXA/10777960 Issue 2. January 2010. (E)
11	11.5	R10017	1	The case for disposability of spent fuel and ILW. R10-017 (A) Issue 1. AREVA. February 2010. (E)
11	11.5	REGEPR00637R		Letter dated 14th January 2011 from EDF/AREVA to HSE/EA “Consolidation of the Plan for the Disposability of Waste and Spent Fuel”, REG EPR00637R
11	11.5	NT100330000003	B	Technical note. Encapsulation Facilities for Spent Fuel. NT 100330 00 0003 Revision B. AREVA/EDF. September 2010. (E)
11	11.5	R010002A	3	Management of Records for Long Term Management of Spent Fuel and ILW. R010 002 (A) Issue 3. AREVA. February 2010. (E)
CHAPTER 12				
SUB-CHAPTER 12.0 – REFERENCES				
12	12.0	ICRP60		1990 Recommendations of the International Commission on Radiological Protection. ICRP Publication 60. Ann. ICRP 21 (1-3). 1991. (E)
12	12.0	9629EURATOM	Vol39	Council Directive 96/29/EURATOM of 13 May 1996 laying down basic safety standards for the protection of the health of workers and the general public against the dangers arising from ionizing radiation. Official Journal of the European Communities, L159, Vol. 39. June 1996. (E)

12	12.0	ISBN0110856147		The Ionising Radiations Regulations 1999. Statutory Instrument 1999 No. 3232. HM Stationery Office. ISBN 0-11-085614-7. (E)
12	12.0	TINS037	1	Nuclear Safety Directorate. Ionising Radiations Regulations 1999 on Sites Regulated by NSD. T/INS/037 Issue 1. UK Health and Safety Executive (HSE). (E)
12	12.0	REF0006		Radioactive Substances (Basic Safety Standards) (England and Wales) Direction 2000. (E)
12	12.0	ISBN0105412937		The Radioactive Substances Act 1993. HM Stationery Office. ISBN 0-10: 0105412937.
12	12.0	ISBN0110299086		The Radiation (Emergency Preparedness and Public Information) Regulations 2001 (REPPIR). Statutory Instrument 2001 No. 2975. HM Stationery Office. ISBN 0-11-029908-6. (E)
SUB-CHAPTER 12.1 – REFERENCES				
12	12.1	ICRP60		1990 Recommendations of the International Commission on Radiological Protection. ICRP Publication 60. Ann. ICRP 21 (1-3). 1991. (E)
12	12.1	9629EURATOM	Vol39	Council Directive 96/29/EURATOM of 13 May 1996 laying down basic safety standards for the protection of the health of workers and the general public against the dangers arising from ionizing radiation. Official Journal of the European Communities, L159, Vol. 39. June 1996. (E)
12	12.1	ECEP081158		EPR – Source term. Letter ECEP081158. EDF. October 2009. (E)
12	12.1	ECEIG040828	A1	EPR - Establishment of the reference zone, taking account of the statistics of the best French nuclear plants ECEIG040828 Revision A1. EDF. April 2009. (E) ECEIG040828 Revision A1 is the English translation of ECEIG040828 Revision A.
SUB-CHAPTER 12.2 – REFERENCES				
12	12.2	ENTERP090062	A	Primary Source Term for the EPR Reactor ENTERP090062 Revision A. EDF. March 2009. (E) ENTERP090062 Revision A is the English translation of ENTERP070147 Revision A.
12	12.2	NEEMFDC30	F	EPR FA3 SPECIFIC ACTIVITY CONCENTRATIONS OF NUCLIDES IN REACTOR BUILDING SYSTEMS. NEEM-F DC 30 Revision F. AREVA. December 2007. (E)
12	12.2	ENTERP070291	A	EPR: activity concentrations in the TEP, REA, TEG, RPE, TEU and TES auxiliary systems. ENTERP070291 Revision A. EDF. November 2007. (E). ENTERP070291 Revision A is the English translation of ENTERP070070 Revision A.
SUB-CHAPTER 12.3 – REFERENCES				
12	12.3	ECEIG111267	B	UK EPR - Overview of the radiological zoning of the nuclear island. ECEIG111267 Revision B. EDF. March 2012. (E) *Supersedes Revison A July 2011.*

12	12.3	ISBN0110856147		The Ionising Radiations Regulations 1999. Statutory Instrument 1999 No. 3232. HM Stationery Office. ISBN 0-11-085614-7.
12	12.3	9629EURATOM	Vol39	Council Directive 96/29/EURATOM of 13 May 1996 laying down basic safety standards for the protection of the health of workers and the general public against the dangers arising from ionizing radiation. Official Journal of the European Communities, L159, Vol. 39. June 1996.
12	12.3	ECEIG081619	A1	EPR - Optimised dose assessment for unit at power activities planned in the FA3 EPR Reactor Building. ECEIG081619 Revision A1. EDF. November 2009. (E)
12	12.3	EYTS2007fr0042 / EYTS/2007/fr/0224	B1 / A1	System Design Manual – Radiation Monitoring System (KRT [PRMS]) P2 - System Operation. (KRT) EYTS/2007/fr/0042 Revision B1. Sofinel. November 2009. (E) (8KRT) EYTS/2007/fr/0224 Revision A1. SOFINEL. August 2009. (E)
12	12.3	EYTS2007fr0186	A1	System Design Manual – Radiation Monitoring System (KRT [PRMS]) P3 – System sizing. (KRT) EYTS/2007/fr/0186 Revision A1. Sofinel. November 2009. (E)
12	12.3	EYTS2008fr0011 / EYTS/2008/fr/0052	B1 / B1	System Design Manual – Radiation Monitoring System (KRT [PRMS]) P5 – Instrumentation and Control. (KRT) EYTS/2008/fr/0011 Revision B1. Sofinel. November 2009. (E) (8KRT) EYTS/2008/fr/0052 Revision B1. SOFINEL. August 2009. (E)
12	12.3	PEPRFDC38	B	Steam Generator Tube Rupture – Mitigation Strategy. PEPR-F DC 38 Revision B. AREVA. February 2012. (E). Reference has been superseded by Revision D. October 2012.
SUB-CHAPTER 12.4 – REFERENCES				
12	12.4	ECEIG040828	A1	EPR - Taking into account the statistics from the best French plants. – Establishment of the reference dose. ECEIG040828 Revision A1. EDF. April 2009. (E) ECEIG040828 Revision A1 is the English translation of ECEIG040828 Revision A
12	12.4	ECEIG040681	B1	EPR – FA3 – Methodology of the optimisation studies for Activities with High Level regarding Radiological Protection. ECEIG040681 Revision B1. EDF. January 2006. (E) ECEIG040681 Revision B1 is the English translation of ECEIG040681 Revision B
12	12.4	ECEIG040601	C1	EPR - List of priority activities concerned by optimization. ECEIG040601 Revision C1. EDF. July 2009. (E) ECEIG040601 Revision C1 is the English translation of ECEIG040601 Revision C
12	12.4	NFPMRDC0004	B	Report on the suppression or reduction of stellite hardfacing in RPV internals. NFPMR DC 0004. Revision B. AREVA. October 2004. (E)
12	12.4	ECEIG081619	A1	EPR - Optimised dose assessment for unit at power activities planned in the FA3 EPR Reactor Building. ECEIG081619 Revision A1. EDF. November 2009. (E) ECEIG081619 Revision A1 is the English translation of ECEIG081619 Revision A.

12	12.4	EYRL2008fr0003	D1	Optimization of activities with radiation protection impact – chapter 2 – “Removing and fitting heat insulation and lagging”. EYRL2008FR0003 Revision D1. Sofinel. June 2010. (E) EYRL2008FR0003 Revision D1 is the English translation of EYRL2008FR0003 Revision D.
12	12.4	EYRL2008fr0048	C1	Optimization of activities with radiation protection impact – chapter 2 – Worksite logistics. EYRL2008FR0048 Revision C1. Sofinel. February 2010. (E) EYRL2008FR0048 Revision C1 is the English translation of EYRL2008FR0048 Revision C
12	12.4	ECEMA071469	C1	EPR - optimisation of activities that impact radiation protection - RCP, RCV and RIS/RRA valves - volume 2. ECEMA071469 Revision C1. EDF. May 2010. (E) ECEMA071469 Revision C1 is the English translation of ECEMA071469 Revision C
12	12.4	ECEMA070805	B1	EPR – Optimisation of radiological protection activities – “SG preparation and tests” – Section 2. ECEMA070805 Revision B1. EDF. April 2010 (E) ECEMA070805 Revision B1 is the English translation of ECEMA070805 Revision B
12	12.4	ECEMA070986	B1	EPR – Optimizing radiation protection-related activities – “opening/closing of reactor vessel head” activity – Chapter 2. ECEMA070986 Revision B1. EDF. April 2010. (E) ECEMA070986 Revision B1 is the English translation of ECEMA070986 Revision B
12	12.4	EYTM2007fr0030	C1	EPR – Optimization of activities with radiation protection impact fuel shipment – chapter 2 – YR2621. EYTM/2007/fr/0030 Revision C1. EDF. June 2010. (E) EYTM2007FR0030 Revision C1 is the English translation of EYTM2007FR0030 Revision C.
12	12.4	D40029206123	0.1	EPR Optimisation of Radioprotection activities. Waste treatment Phases 1 and 2. D4002.92-06/123 Revision 0.1. EDF CNEN. March 2009. (E)
SUB-CHAPTER 12.5 – REFERENCES				
12	12.5	ISBN0110299086		The Radiation (Emergency Preparedness and Public Information) Regulations 2001 (REPPIR). Statutory Instrument 2001 No. 2975. HM Stationery Office. ISBN 0-11-029908-6.
12	12.5	ISBN0110856147		The Ionising Radiations Regulations 1999. Statutory Instrument 1999 No. 3232. ISBN 0-11-085614-7. HM Stationery Office. (E)
12	12.5	ISBN0108502163		The Nuclear Installations Act 1965 (as amended). ISBN 0108502163. HM Stationery Office. (E)
12	12.5	REF0019		The Licensing of Nuclear Installations. April 2007. http://www.hse.gov.uk/nuclear/notesforapplicants.pdf. UK Health and Safety Executive (HSE).
12	12.5	ECEIG101497	A1	Access to the Safeguards Buildings and the Fuel Building in a long term post accident situation (PCC4 and RRC-B). ECEIG101497 Revision A1. EDF. February 2011. (E) ECEIG101497 Revision A1 is the English translation of ECEIG101497 Revision A.

CHAPTER 13				
SUB-CHAPTER 13.1 – REFERENCES				
13	13.1	ENSNEA080058	A	EPR - External hazards – inventory of combined events with internal faults and/or other (internal and external) hazards taken into account in design. ENSNEA080058 Revision A. EDF. November 2008. (E) ENSNEA080058 Revision A is the English translation of ENSNEA050062 Revision A.
13	13.1	ENGSDS050211	A1	E Champain. Design basis floor response spectra: EPR plant series. ENG SDS 05 0211 Revision A1. EDF. June 2009. (E) ENG SDS 05 0211 Revision A1 is the English translation of ENG SDS 05 0211 Revision A.
13	13.1	ENSN090020	A	J Barbaud. Verifying nuclear facilities for earthquakes verification guide according to CCE nos 1005 and 1032. ENSN090020 Revision A. EDF. February 2009. (E) ENSN090020 Revision A is the English translation of ENSN8779 Revision E.
13	13.1	ECEIG040652	C	Earthquake Event. Report of layout analysis methodology in buildings. Conditions for preparation of design meeting. ECEIG040652 Revision C. EDF. 2007. (E)
13	13.1	ENSNDR080217	A	EPR - Multiple non-seismic equipment failures: presentation of the verification approach. ENSNDR080217 Revision A. EDF. December 2008. (E) ENSNDR080217 Revision A is the English translation of ENSNDR030011 Revision B.
13	13.1	ECEIG040650	C1	Internal flooding - Layout Analysis Methodology for buildings in the nuclear island. ECEIG040650 Revision C1. EDF. September 2008. (E)
13	13.1	ECEIG021405	H1	EPR - General hypotheses note for the civil engineering design of the nuclear island buildings.. ECEIG021405 Revision H1 (TR 07/312). EDF. October 2008. (E)
13	13.1	ENSN930077	A	Hazards of external origin aircraft crashes. ENSN930077 Revision A (TR 93-04). EDF. (E)
13	13.1	IAEANSG15		Standard NS-G-1.5 (External Events Excluding Earthquakes in the Design of Nuclear Power Plants). IAEA. 2003. (E)
13	13.1	334002		Unified Facilities Criteria UFC 3-340-02 ‘Structures to resist the effects of accidental explosions’ (E)
13	13.1	ELIMR0600394	A1	Hazards related to the roads and utility networks and the industrial environment of the Nuclear Power Station – Methodological Guide. ELIMR0600394 Revision A1. EDF. June 2009. (E) ELIMR0600394 Revision A1 is the English translation of ELIMR0600394 Revision A.
13	13.1	ENSNEA090003	A	Protection of nuclear units against risk of external flooding. Methodological procedure for checking protection provisions. ENSNEA090003 Revision A. EDF. February 2009. (E) . ENSNEA090003 Revision A is the English translation of document ENSN0000850 D
13	13.1	BSEN199113		Eurocode 1: Actions on structures – Part 1-3: General actions – Snow loads. BS EN 1991-1-3. 24 July 2003. (E)
13	13.1	BSEN199114		Eurocode 1: Actions on structures – Part 1-4: General actions – Wind actions. BS EN 1991-1-4. 25 April 2005. (E)
13	13.1	ENSNEA090019	A	Reference base for projectiles generated by extreme wind. ENSNEA090019 Revision A. EDF. March 2009. (E) ENSNEA090019 Revision A is the English translation of ENSNEA050020 Revision A

13	13.1	ENSNEA090034	A	A D Chesne. Safety requirements reference baseline: general design rules for protection from extreme cold. ENSNEA090034 Revision A. EDF. April 2009. (E) ENSNEA090034 Revision A is the English translation of ENSN870087 design report.
13	13.1	ENSNEA090013	A	A D Chesne. "Heatwave" reference base. ENSNEA090013 Revision A. EDF. April 2009. (E) ENSNEA090013 Revision A is the English translation of ENSNEA040103 design report.
13	13.1	ECEIG061084	A1	Methodology for studies to be carried out in the processing of "Extreme cold" for the EPR. ECEIG061084 Revision A1. EDF. 2006. (E) ECEIG061084 Revision A1 is the English translation of ECEIG061084 Revision A.
13	13.1	ENITSF090003	A	Protection of nuclear power plants against the risks of low heat sink water level – Methodology for determining the Lowest Safety Water Level. ENITSF09003 Revision A. EDF-SA. January 2009. (E)
13	13.1	ENSEMD090183	A	Lightning safety reference base applicable to the EPR. ENSEMD090183 Revision A. EDF. August 2009. (E) ENSEMD090183 Revision A is the English translation of ENSEMD060324 Revision A
13	13.1	ECESN120418	A	UK EPR Hazard Fault Schedule – Representative Cases Based on Application of Hazard Fault Schedule Methodology. ECESN120418 Revision A. EDF. July 2012. (E)
SUB-CHAPTER 13.2 – REFERENCES				
13	13.2	ECEF041142	B1	Procedure ENG 2-98 : High energy pipes leaks and breaks analysis methodology. ECEF041142 Revision B1. EDF. June 2009. (E) ECEF041142 Revision B1 is the English translation of ECEF041142 Revision B.
13	13.2	ENSNDR090183	D	Identification of High Integrity Components – components whose gross failure is discounted. ENSNDR090183 Revision D. EDF. October 2012. (E) *Supersedes Revison B December 2010.*
13	13.2	ENSNDR080245	A	Break Preclusion in reactor main coolant lines and main steam lines. Positioning of the concept and associated safety requirements . ENSNDR080245 Revision A. EDF. (E) ENSNDR080245 Revision A is the English translation of ENSN040134 Revision C.
13	13.2	NUREG0800362	2	Determination of rupture locations and dynamic effects associated with the postulated rupture of piping. NUREG-0800 Standard Review Plan - 3.6.2 Revision 2. March 2007. (E)
13	13.2	NUREG080033	3	Protection Against Postulated Piping Failures in Fluid Systems Outside Containment. NUREG-0800 Standard Review Plan - Branch Technical Position 3-3 Revision 3. March 2007. (E)
13	13.2	NUREG080034	2	Postulated Rupture Locations in Fluid System Piping Inside and Outside Containment. NUREG-0800 Standard Review Plan - Branch Technical Position 3-4 Revision 2. March 2007. (E)
13	13.2	IAEANSG111		Protection against internal hazards other than fires and explosions in the design of nuclear power plants. IAEA Safety Guide NS-G-1.11. 2004. (E)

13	13.2	RCCM2007		Design and Construction Rules for Mechanical Components of PWR Nuclear Islands. AFCEN. 2007 Edition. (E)
13	13.2	ECEF092040	A1	1st stage analysis report on the consequences of a high-energy line break - Reactor Building. ECEF092040 Revision A1. EDF. April 2010. (E)
13	13.2	ECEF092041	A1	Stage 1 analysis of consequences of high-energy line breaks in fuel building. ECEF092041 Revision A1. EDF. November 2009. (E)
13	13.2	ECEF092042	A1	1st stage analysis: consequences of a high-energy line break - safeguard auxiliary and electrical buildings. ECEF092042 Revision A1. EDF. February 2010. (E)
13	13.2	ECEF101595	A1	Additional information for the stage 1 analysis report on the consequences of high energy line breaks – fuel building. ECEF101595 Revision A1. EDF. May 2011. (E)
13	13.2	ECEF101596	A1	Additional information for the stage 1 analysis report on the consequences of high energy line breaks – safeguard auxiliary buildings. ECEF101596 Revision A1. EDF. May 2011. (E)
13	13.2	EZLT2010en0007	B	High Energy Pipe Break: Propagation of degraded ambient conditions in the Nuclear Island. EZLT/2010/en/0007 Revision B. EDF. August 2010. (E)
13	13.2	ECEIG091634	B1	EPR – Internal Missiles – Risk assessment report on building structure and layout. ECEIG091634 Revision B1. EDF. April 2011. (E)
13	13.2	ECEIG112173	A	Internal Missiles - Detailed analysis of the selected safety classified components gross failure. ECEIG112173 Revision A. EDF. January 2012. (E)
13	13.2	6CS20099	B	Reactor coolant pump of EPR - Mechanical behaviour of the flywheel – dimensioning analysis. 6 CS 20099 Revision B. AREVA. November 2006. (E)
13	13.2	ECEIG070282	A1	EPR – Internal missiles – methodology description for analysis of layout in the buildings – Conditions for preparing design reviews. ECEIG070282 Revision A1. EDF. April 2010. (E)
13	13.2	ECEMA101802	B	UK EPR GDA - Management of Nuclear Safety Significant Lifting. ECEMA101802 Revision B. EDF. December 2010. (E)
13	13.2	ECEIG070272	A1	EPR - Load drops - Methodology for risks analysis in civil engineering and building installations - Design review preparation conditions. ECEIG070272 A1. EDF-SA. March 2009. (E)
13	13.2	ENGSGC100483	B	Methods with regard to the risk of "dropped loads" for EPR UK for civil works structures. ENGSGC100483 Revision B. EDF. February 2012. (E)
13	13.2	ECEIG120198	A	Safety case of four representative load drops from RS2 cranes. ECEIG120198 Revision A. EDF. February 2012. (E)
13	13.2	PEPSFDC135	B	Identification of Dropped loads and Fuel Handling Human Based Safety Claims- Refueling Machine. PEPS-F DC 135 Revision B. AREVA. July 2012. (E)
13	13.2	PEPSG2011en1076	C	ALARP Justification of representative Drop Load Cases from SC1 polar crane. PEPS-G/2011/en/1076 Revision C. AREVA. February 2012. (E)

13	13.2	PEPSFDC134	B	Identification of Dropped loads and Fuel Handling Human Based Safety Claims- Polar Crane. PEPS-F DC 134 Revision B. AREVA. June 2012. (E)
13	13.2	ECEIG060350	A1	Internal explosion methodology report ECEIG060350 Revision A1. EDF. February 2009. (E) ECEIG060350 Revision A1 is the English translation of ECEIG060350 Revision A.
13	13.2	ECEIG070648	A1	Analysis of operational experience feedback concerning hydrogen-related explosions in safety-classified buildings at nuclear power plants in France and abroad, as applied to the EPR. ECEIG070648 Revision A1. EDF. October 2008. (E)
13	13.2	EYRL2007fr0036	H1	Identification of systems and rooms in the EPR nuclear island (BNI) with risk of internal explosion. EYRL/2007/fr/0036 Revision H1. EDF. April 2011. (E)
13	13.2	ENGSIN080356	A	B. Gautier. Reference Base for safety requirements concerning protection against the risk of internal explosion for the EPR. ENGSIN080356. Revision A. March 2009. EDF. (E) ENGSIN080356 Revision A is the English translation of document ENGSIN060237 Revision A
13	13.2	ECEIG0100658	C1	Fire methodology report. ECEIG0100658 Revision C1. EDF. February 2009. (E)
13	13.2	ENGSIN050312	B	M Mosse. EPR Technical Code for Fire Protection. ETC-F Version G. ENGSIN050312 Revision B. EDF. August 2007. (E)
13	13.2	EN135012		Fire classification of construction products and building elements – EN 13501-2:2003. (E)
13	13.2	NESPG2009EN1322	C	Fire Safety Assessment of Safety Fire Barriers- Evaluation of fire resistance using experience of comparable EPR layouts. NESP-G/2009/en/1322 Revision C. AREVA. March 2011. (E)
13	13.2	ECEIG091608	A	UK EPR – MAGIC simulation for one representative safety compartment (SCO) and comparison with standard resistance curves without taking credit for active fire protection systems. ECEIG091608 Revision A. EDF. September 2009. (E)
13	13.2	ENSNEA090055	A	Principle of common mode fire risk analysis. ENSNEA090055 Revision A. EDF. July 2009. (E) ENSNEA090055 Revision A is the English translation of ENSNEA070029 Revision A.
13	13.2	ENGSIN080155	A	Principles of the EPRESSI method. ENGSIN080155 Revision A. EDF-SA. May 2008. (E)
13	13.2	HI8204024	B	MAGIC software version 4.1.1: Mathematical model. HI-82/04/024/B. EDF-SA. 2005. (E) This document contains EDF-SA proprietary information and can be accessed by direct contact with EDF SA
13	13.2	HI8204022	B	Qualification file of Fire Code Magic version 4.1.1. HI-82/04/022 Revision B. EDF. December 2005. (E)
13	13.2	EYRL2007FR0026	D1	Justification of fire zoning in the emergency diesel buildings at Flamanville 3. EYRL/2007/FR0026 Revision D1. SOFINEL. August 2009. (E) EYRL/2007/FR/0026 Revision D1 is the English translation of EYRL/2007/FR/0026 Revision D
13	13.2	EYRL2007FR0020	E1	Justification of fire zoning within the Flamanville 3 Fuel Building. EYRL/2007/FR/0020 Revision E1. SOFINEL. September 2009. (E) EYRL/2007/FR/0020 Revision E1 is the English translation of EYRL/2007/FR/0020 Revision E

13	13.2	EYRL2007FR0022	E1	Justification of fire zoning within the Flamanville 3 Reactor Building. EYRL/2007/FR/0022 Revision E1. SOFINEL. August 2009. (E) EYRL/2007/FR/0022 Revision E1 is the English translation of EYRL/2007/FR/0022 Revision E
13	13.2	EZL2006en0082	F	Transverse Technical Studies Fire (Internal Hazard) – Studies by safeguard buildings 1/2/3/4 justification of the fire zoning. EZL/2006/en/0082 Revision F. EDF. April 2009. (E)
13	13.2	EYRT2009FR0027	B	Verification of zoning fire resistance in Waste Building Flamanville 3. EYRT/2009/FR/0027 Revision B. SOFINEL. November 2009. (E)
13	13.2	EYRT2009FR0028	B	Verification of zoning fire resistance in Diesel buildings Flamanville 3. EYRT/2009/FR/0028 Revision B. SOFINEL. November 2009. (E)
13	13.2	EYRL2008FR0105	B	Preliminary analysis for verification of zoning fire resistance in fuel building. EYRL/2008/FR/0105 Revision B. SOFINEL. April 2009. (E)
13	13.2	EYRT2010FR0042	B1	Location of cable trays in EPR FA3 Nuclear Island. EYRT/2010/FR/0042 Revision B1. EDF. November 2011. (E)
13	13.2	ENSNEA090003	A	Protection of nuclear units against risk of external flooding. Methodological procedure for checking protection provisions. ENSNEA090003 Revision A. EDF. February 2009. (E) . ENSNEA090003 Revision A is the English translation of document ENSN0000850 Revision D.
13	13.2	ECEIG110718	A	Identification of bounding cases - leak volumes and retention volumes. ECEIG110718 Revision A. EDF. June 2011. (E)
13	13.2	ECEIG111647	B	Bounding Cases: Mitigation Measures Report. ECEIG111647 Revision B. EDF. January 2012. (E)
13	13.2	16895707000RPT0013	E	EDF/AREVA GDA Human Factors Internal Flooding. 16895-707-000-RPT-0013 Revision E. AMEC. September 2012. (E)
13	13.2	ECEIG121115	B	UK EPR - Internal flooding - Multi-legged safety case and ALARP consequence assessment analyses. ECEIG121115 Revision B. EDF. September 2012. (E)
13	13.2	ECEIG040650	C1	Internal flooding – Layout Analysis Methodology for buildings in the nuclear island. ECEIG040650 Revision C1. EDF. 2008. (E)
13	13.2	PF2009EN0001	D	Flooding Safety Analysis for Nuclear Auxiliary Building HNX. PF2009EN0001 Revision D. SOFINEL. February 2011. (E)
13	13.2	EZT2009EN0005	E	Flooding event : Analysis of the Safeguard Buildings. EZT/2009/EN/0005 Revision E. SOFINEL March 2011. (E)
13	13.2	EYRT2009FR0008	C1	EPR FA3 Fuel Building Internal Flooding Assessment. EYRT/2009/FR/0008 Revision C1. SOFINEL. November 2011. (E)
13	13.2	EYRT2009FR0076	D1	Analysis of Internal Flooding in the FA3 EPR Reactor Building. EYRT/2009/FR/0076 Revision D1. SOFINEL. November 2011. (E)
13	13.2	EYRT2009FR0032	E1	Study of Internal Flooding events in the Diesel Generator Building of the Flamanville FA3 EPR. EYRT/2009/FR/0032 Revision E1. EDF. April 2012. (E)

13	13.2	ECESN120418	A	UK EPR Hazard Fault Schedule – Representative Cases Based on Application of Hazard Fault Schedule Methodology. ECESN120418 Revision A. EDF. July 2012. (E)
CHAPTER 14				
SUB-CHAPTER 14.0 – REFERENCES				
14	14.0	NEPRFDC584	A	V. Verreman. Consistency Between PSA List and PCC List. NEPR-F DC 584 Revision A. AREVA. July 2010 (E)
14	14.0	EPRRDC1685	C	R Gagner. EPR sizing at 4500MWth. EPRR DC 1685 Revision C. AREVA. February 2004. (E)
14	14.0	FS10000607	3.0	Nuclear Fuel - Specification and Justification of Design and Safety Criteria. FS1-0000607 Revision 3.0. AREVA. October 2010. (E)
14	14.0	ENCNTC100060	B	EPR-UK : Fuel rod PCMI failure criterion for RCCA ejection fault studies. ENCNTC100060 Revision B. EDF. September 2010. (E)
14	14.0	JNMVol95		M Ishikawa, S Shiozawa. A study of fuel behaviour under reactivity initiated accident conditions – review. Journal of Nuclear Materials, Volume 95. November 1980. (E)
14	14.0	FFDC05182	B	EPR HARMONI® RCCA Description, Functional Requirements and Materials Properties. FF DC 05182 Revision B. AREVA. July 2009. (E)
SUB-CHAPTER 14.1 – REFERENCES				
14	14.1	EPRRDC1685	C	R Gagner. EPR sizing at 4500MWth. EPRR DC 1685 Revision C. AREVA. February 2004. (E)
14	14.1	EPRRDC1651	D	N. Goreaud. EPR – Lower Plenum Hydraulics Design of the Flow Distribution Device. EPRR DC 1651 Revision D. AREVA. December 2001. (E)
14	14.1	EPDSDC150	C	S. Ebalard. ORIGEN-S: Qualification manual. EPDS DC 150 Revision C.AREVA. November 2005. (E)
14	14.1	NFPSCDC283	C	S. Laurent. Residual Decay Heat Curves for System Design and Accident Analysis (Update 4500 MWth). NFPSC DC 283 Revision C. AREVA. November 2005. (E)
14	14.1	NFPSCDC1042	C	S. Laurent. EPR – FA3 NSSS operating parameters. NFPSC DC 1042 Revision C. AREVA. December 2006 (E)
14	14.1	NFPSCDC286	B	S. Laurent. Neutronic data for transient analyses (update 4500 MWth). NFPSC DC 286 Revision B. AREVA. February 2006. (E)
14	14.1	NFPSCDC284	B	C. Hove. Core Reactivity Control (update 4500 MWth). NFPSC DC 284 Revision B. AREVA. January 2006. (E)
14	14.1	EPRRDC1690	C	R. Gagner. RPV Thermal-hydraulic Design: Rod Drop Time. EPRR DC 1690 Revision C. AREVA. July 2005. (E)
14	14.1	NEEGFDC5	F	A. Nicoli. EPR™ 79/19 TE – Steam generator operating parameters. NEEG-F DC 5 Revision F. AREVA. January 2009. (E)

SUB-CHAPTER 14.2 – REFERENCES				
14	14.2	NEPSFDC557	D	Classification of structure systems and components. NEPS-F DC 557 Revision D. AREVA. October 2012. (E)
14	14.2	NESSFDC539	A	System Design Manual – Safety injection system and Residual Heat Removal System – Part 2: System Operation. NESS-F DC 539 Revision A. AREVA. May 2009. (E)
14	14.2	NESSFDC580	A	System Design Manual – Main Steam Relief Train – Part 2: System Operation. NESS-F DC 580 Revision A. AREVA. November 2009. (E)
14	14.2	SFLEZS030046	G	System Design Manual – Steam Generator Blowdown System APG P2 - System Operation. SFL-EZS-030046 Revision G. Sofinel. December 2008. (E)
14	14.2	NESSFDC535	A	System Design Manual. Extra Boration System - Part 2: System Operation. NESS-F DC 535 Revision A. AREVA. April 2009. (E)
14	14.2	NESSFDC538	A	System Design Manual – Reactor Coolant System – Part 2: System Operation. NESS-F DC 538 Revision A. AREVA. May 2009. (E)
14	14.2	NESSFDC578	A	System Design Manual – Main Steam Supply System – Part 2: System Operation. NESS-F DC 578 Revision A. AREVA. November 2009. (E)
14	14.2	SFLEFMF2006829	E1	System Design Manual – Emergency Feedwater System – Part 2: System Operation. SFL-EFMF 2006.829 Revision E1. Sofinel. September 2009. (E)
14	14.2	SFLEFMF2006712	G1	System Design Manual – Fuel Pool Cooling System – Part 2: System Operation. SFL-EFMF 2006.712 Revision G1. Sofinel. August 2009. (E)
14	14.2	ETDOIG080015	A1	System Design Manual - Classified fire-fighting water production system – Part 2: System Operation. ETDOIG080015 Revision A1. EDF. August 2009. (E)
14	14.2	EZH2007en0006	D	System Design Manual - Fire Fighting Water System of the Nuclear Island (JPI) – Part 2: System Operation. EZH/2007/en/0006 Revision D. EDF. July 2008. (E)
14	14.2	SFLEFMF2006446	F1	System Design Manual – Component Cooling Water System – Part 2: System Operation. SFL-EFMF 2006.446 Revision F1. Sofinel. August 2009. (E)
14	14.2	ETDOFC080069	A1	System Design Manual – Essential Service Water System – Part 2: System Operation. ETDOFC/080069 Revision A1. EDF. September 2009. (E)
14	14.2	SFLEZL030004	H	System Design Manual – Safety Chilled Water System – Part 2: System Operation. SFL-EZL-030004 Revision H. EDF. November 2008. (E)
14	14.2	IAEANSR1		Safety of Nuclear Power Plants: Design (Requirements for Design). ISSN 1020-525X IAEA Safety Standards Series N° NS-R-1. IAEA. 2000. (E)
14	14.2	PEPRFDC28	A	ALARP demonstration for the design of the pressurizer safety valves regarding the passive single failure. PEPR-F DC 28 Revision A. AREVA. November 2010 (E)
14	14.2	PESSFDC27	A	UK EPR - Main Steam Isolation Valves ALARP Assessment regarding functional diversity and Single Failure Criterion. PESS-F DC 27 Revision A. AREVA. November 2010. (E)

SUB-CHAPTER 14.3 – REFERENCES				
14	14.3	NFPSCDC285	A	S. Laurent. Fuel management – Neutronic design report (SCIENCE calculations) (Update 4500 MWth). NFPSC DC 285 Revision A. AREVA. September 2004. (E)
14	14.3	ITSRDC162	A	T. Godefroid. EPR – Basic Design – Extra boration system – System manual – D2.4 Physical phenomena determining the operating conditions. ITSR DC 162 Revision A. AREVA. November 1998. (E)
14	14.3	NFPSCDC286	B	S. Laurent. Neutronic data for transient analysis (Update 4500 MWth). NFPSC DC 286 Revision B. AREVA. February 2006. (E)
14	14.3	PSRRDC6	B	N. Nicaise. PSAR Sections 15.2.2D, 15.2.3D, 15.2.4D. PSRR DC 6 Revision B. AREVA. December 2003. (E)
14	14.3	EPRRDC1693	C	J. Feingold. PSAR section 15.1 Plant characteristics assumed in the accident analyses. EPRR DC 1693 Revision C. December 2003 (E)
14	14.3	NFEPCDC15	C	C. Panefresco. Fuel management – Neutronic design report (SCIENCE calculations). NFEPC DC 15 Revision C. AREVA. September 2003. (E)
14	14.3	KWUST1487e2168		Dr Gerth, Dr Ro. Verification of the Korea specific Version of NLOOP by recalculation of the “Station Blackout Accident” at KNU 1. KWU Work-Report ST14-87-e2168 . AREVA. (E) This document contains proprietary information and can be accessed only within AREVA offices
14	14.3	KWUR15842137		Seitz, Hofmann . Verifikation des Rechenprogramms NLOOP für die Anlage Biblis-B am Verlauf der Störung TUSA ohne FDU vom 08.02.84 . [Verification of the NLOOP code for the Biblis-B Nuclear Power Station using a turbine trip incident without main steam by pass which occurred on Feb. 8. 1984.] KWU Arbeitsbericht R15-84-2137 from 26.10.84. AREVA . This document contains proprietary information and can be accessed only within AREVA offices
14	14.3	KWUE411912066		Richter . Vergleichsrechnung zu dem PKL III versuch “A5.2, totaler Ausfall des Speisewassers bei einem intakten DE” mit dem midifizierten Rechenprogramm NLOOP . [Comparison of results from the modified NLOOP code with the test outcome of the PKL III experiment“ A5.2 Complete Loss of main feedwater in an intact steam generator”]. Siemens Arbeitsbericht KWU E411/91/2066 . AREVA . This document contains proprietary information and can be accessed only within AREVA offices
14	14.3	A1C13058600		R. Muller. Analysis of Grafenrheinfeld Pump Shaft Break Event with PANBOX 2 including COBRA 3-CP. Work Report A1C-1 305860-0, dated April 26, 1999. AREVA. (E) This document contains proprietary information and can be accessed only within AREVA offices
14	14.3	REF0025		Finnemann . Proceedings of a Specialists Meeting on Calculation of 3-Dimensional Rating Distributions in Operating Reactors, Paris, 26th-28th November 1979 (Nodal Expansion Method For The Analysis Of Space-Time Effects In LWRs). AREVA . (E) . This document contains proprietary information and can be accessed only within AREVA offices
14	14.3	AIC1307321		Dr. Goll . Mikrostrukturelle Bestimmung der Brennstoffzentraltemperatur im Normalbetrieb . [Determination of the fuel temperature in normal operation]. Work Report A1C-1 307321, dated January 17, 2000 . AREVA .This document contains proprietary information and can be accessed only within AREVA offices

14	14.3	REF0026		EPR Preliminary Safety Analysis Report (PSAR 4250) - section 15.2.2M. “Uncontrolled RCCA Withdrawal (State A)”. Edition 2003. AREVA. (E)
14	14.3	REF0027		EPR Preliminary Safety Analysis Report, Section 15.2.2P “RCCA misalignment up to rod drop without limitation” (PSAR 4250). Edition 2003. AREVA. (E)
14	14.3	NEPRFDC469	A	L. CARFANTAN. Definition of P/S I&C functions. NEPR-F DC 469 Revision A. AREVA. April 2009.(E)
14	14.3	DNM03451		Experience Feedback from test facilities and plants on RCS loop level measurements. AREVA. DNM03451. August 1998. (E)
14	14.3	EPRRDC1685	C	R Gagner. EPR sizing at 4500MWth. EPRR DC 1685 Revision C. AREVA. February 2004. (E)
14	14.3	DNM01808		Safety Injection System Manual, Chapter D2.3, Description, equipment characteristics. DNM01808. (E)
14	14.3	NESSFDC538	A	System Design Manual - Reactor Coolant System (RCP [RCS]) - Part 2: System Operation. NESS-F DC 538 Revision A. AREVA. May 2009. (E) NESS-F DC 538 Revision A is the English translation of NFPMS DC 1132 Revision F.
14	14.3	NFPSCDC283	C	S. Laurent. Residual Decay Heat Curves for System Design and Accident Analysis (Update 4500 MWth). NFPSC DC 283 Revision C. AREVA. November 2005. (E)
14	14.3	EPRRDC1701	B	R. Gagner. EPR Operation at 4250MWth. EPRR DC 1701 Révision B. AREVA. February 2004. (E)
14	14.3	NEPCFDC164	B	Residual Decay Heat Curves for Major Components Design Purposes Heat Load inside Fuel Pool. NEPC-F DC 164 Revision B. AREVA NP. November 2008. (E)
14	14.3	SFLEFMF2006751	F1	System Design Manual - Fuel Pool Purification and Cooling Systems (PTR [FPPS/FPCS]) Part 5 – Instrumentation and Control. SFL EF MF 2006.751 Revision F1. Sofinel. September 2009. (E) SFL EF MF 2006.751 Revision F1 is the English translation of SFL EF MF 2006.751 Revision F.
14	14.3	SFLEFMF2006712	G1	System Design Manual - Fuel Pool Cooling System (PTR [FPPS/FPCS]) - Part 2 System Operation SFL-EFMF 2006.712 Revision G1. Sofinel. August 2009. (E) SFL–EFMF 2006.712 Revision G1 is the English translation of SFL–EFMF 2006.712 Revision G.
14	14.3	ECEF080499	B1	Functional study on the treatment of PCCs and RRC-As involving spent fuel pool cooling loss and draining. ECEF080499 Revision B1. EDF. November 2012. (E)
14	14.3	REF0028		EPR FA3 Preliminary Safety Analysis Report, Section 15.2.2x “PCC 2 Loss of a PTR cooling train or a PTR support system (State A)”. Edition 2006. EDF. (E)
SUB-CHAPTER 14.4 – REFERENCES				
14	14.4	EMLRDC0022	B	Y Benoit. EPR RF02 MSRIV description (Main steam relief isolation valve). EMLR DC 0022 Revision B. AREVA. October 2003. (E)
14	14.4	ITSRDC0178	E	J L Riviere. MSSS / VDA System specification. ITSR DC 0178 Revision E. AREVA. January 2004. (E).
14	14.4	PEPRFDC24	A	N Jarrin. UK EPR – Safety Case for the Inherent Boron Dilution Following LOCA. PEPR-F DC 24 Revision A. AREVA. July 2011 (E)

14	14.4	EPRRDC1685	C	R Gagner. EPR sizing at 4500MWth. EPRR DC 1685 Revision C. AREVA. February 2004. (E)
14	14.4	NFPSRDC1044	B	L Foucart. Safety Report Chapter 15.2.3e Small Break LOCA < DN 50 (State A), notably break occurring on the Extra Boration System Injection Line (4500 MWth update). NFPSR DC 1044 Revision B. AREVA. July 2005. (E)
14	14.4	PEPRFDC38	D	T Bruyères. Steam Generator Tube Rupture Mitigation Strategy. PEPR-F DC 38 Revision D. AREVA. October 2012. (E).
14	14.4	EPDDC490	C	A Barbier. CATHARE 2 – Code synthetic qualification assessment. EPD DC 490 Revision C. AREVA. September 2007. (E)
14	14.4	EMF2100	6	S-RELAP5 Models and correlation. Code Manual EMF-2100 Revision 6. (E)
14	14.4	KWUNDS1971002		Validation of the S-RELAP5 program. KWU NDS1/97/1002. (E)
14	14.4	REF0029		EPR Preliminary Safety Analysis Report (PSAR 4250) – Section 15.2.3P. “Uncontrolled Single Control Rod Withdrawal (State A)”. Edition 2003. AREVA. (E)
14	14.4	REF0028a		EPR Preliminary Safety Analysis Report, Section 15.2.3r “PCC-3: Loss of electrical supplies (> 2 hours), fuel pool cooling aspects (State A)”. EDF. Edition 2006. (E)
14	14.4	NEPCFDC164	B	Residual Decay Heat Curves for Major Components Design Purposes Heat Load inside Fuel Pool. NEPC-F DC 164 Revision B. AREVA NP. November 2008. (E)
14	14.4	ECEF080499	B1	Functional study on the treatment of PCCs and RRC-As involving spent fuel pool cooling loss and draining. ECEF080499 Revision B1. EDF. November 2012. (E)
14	14.4	SFLEFMF2006751	F1	System Design Manual - Fuel Pool Purification and Cooling Systems (PTR [FPPS/FPCS]) Part 5 – Instrumentation and Control. SFL EF MF 2006.751 Revision F1. Sofinel. September 2009. (E) SFL EF MF 2006.751 Revision F1 is the English translation of SFL EF MF 2006.751 Revision F.
14	14.4	SFLEFMF2006712	G1	System Design Manual - Fuel Pool Cooling System (PTR [FPPS/FPCS]) - Part 2 System Operation SFL–EFMF 2006.712 Revision G1. Sofinel. August 2009. (E) SFL–EFMF 2006.712 Revision G1 is the English translation of SFL–EFMF 2006.712 Revision G.
SUB-CHAPTER 14.5 – REFERENCES				
14	14.5	NESSFDC538	A	System Design Manual - Reactor Coolant System (RCP [RCS]) - Part 2: System Operation. NESS-F DC 538 Revision A. AREVA. May 2009. (E)
14	14.5	REF0030		EPR FA3 Preliminary Safety Analysis Report, Section 15.2.4a. “Loss of external power supplies (> 2 hours) (State C)”. Edition 2006. EDF. (E)
14	14.5	PEPRFDC3	A	J-B Decaudin. Main Steam Line Break. PEPR-F DC 3 Revision A. AREVA. May 2010. (E)
14	14.5	EPRRDC1685	C	R Gagner. EPR sizing at 4500MWth. EPRR DC 1685 Revision C. AREVA. February 2004. (E)
14	14.5	NEEGFDC1210	F	D Venturi. 79/19TE Steam generator secondary side pressure vessel details right and left SGs. NEEG-F DC 1210 Revision F. AREVA. April 2008. (E)

14	14.5	PSRRDC13	B	M Petiard – H. Lafitte. EPR Preliminary Safety Analysis Report, Section 15.2.4b. 1- Steam system piping break (State A). Edition 2003. PSRR DC 13 Revision B. AREVA. November 2003. (E)
14	14.5	EPRRDC1651	D	N. Goreaud. EPR – Lower Plenum Hydraulics Design of the Flow Distribution Device. EPRR DC 1651 Revision D. AREVA. December 2001. (E)
14	14.5	EPDDC490	C	A Barbier. CATHARE 2 – Code synthetic qualification assessment. EPD DC 490 Revision C. AREVA. September 2007. (E)
14	14.5	EPRRDC1706		L Foucart. EPR Preliminary Safety Analysis Report, Section 15.2.4c.2 Feedwater System Piping Failure (State B) Edition 2003. EPRRDC 1706. AREVA. December 2003. (E)
14	14.5	EPRRDC1701	B	R Gagner. EPR™ operation at 4250MWth. EPRR DC 1701. Revision B. AREVA. February 2004. (E)
14	14.5	EPRRDC1693	C	J Feingold. EPR Preliminary Safety Analysis Report, Sub-chapter 15.1 “Plant characteristics assumed in the accident analyses”. Edition 2003. EPRR DC 1693 Revision C. AREVA. December 2003. (E)
14	14.5	NFPSCDC1042	C	S Laurent. EPR – FA3 NSSS Operating Parameters. NFPSC DC 1042 Revision C. AREVA. December 2006. (E)
14	14.5	NFPSCDC283	C	S. Laurent. Residual Decay Heat Curves for System Design and Accident Analysis (Update 4500 MWth). NFPSC DC 283 Revision C. AREVA. November 2005. (E)
14	14.5	NFPSCDC284	B	C Hove. Core Reactivity Control (update 4500 MWth). NFPSC DC 284 Revision B. AREVA. January 2006. (E)
14	14.5	REF0031		P Cohen. Water Coolant Technology of Power Reactors. 1980. (E)
14	14.5	NFPMRDB1207	G	EPR FA3 – Primary Loops – Assembly. AREVA-NP drawing. NFPMR DB 1207 Revision G. AREVA. March 2009 (E)
14	14.5	REF0033		B Cournarie. Rapport de sûreté Chapitre 15.2.4f2 APRP brèches intermédiaires et grosses brèches (jusqu’à la rupture de la ligne d’expansion dans l’état B) Update 4500 MWth). [Safety report of Chapter 15.2.4 f2 LOCA large and intermediate breaks (until the rupture of expansion lines in state B]. Edition 2005. AREVA. March 2005.
14	14.5	REF0034		EPR FA3 Preliminary Safety Analysis Report, Section 15.2.4g. Small break LOCA (≤ DN 50) (states C, D). Edition 2003. AREVA. 2003. (E)
14	14.5	A1C13058600		R Muller. Analysis of Grafenrheinfeld Pump Shaft Break Event with PANBOX 2 including COBRA 3-CP. Work Report A1C-1 305860-0. AREVA . April 1999. (E)
14	14.5	AIC1307321		Dr Goll. Mikrostrukturelle Bestimmung der Brennstoffzentraltemperatur im Normalbetrieb. [Determination of the fuel temperature in normal operation]. Work Report A1C-1 307321. AREVA. January 2000.
14	14.5	STRLMLEM94217		Rapport d’évaluation du code CATHARE 2 (version 1.3, revision 5) [Calculation code CATHARE 2 (version 1.3 revision 5) assessment report] STR/LML/EM/94-217.
14	14.5	SETHLES90101		BETHSY – Test 4.3b – Multiple SGTR – test report. SETH/LES/90-101. (E)
14	14.5	REF0035		EPR FA3 – Preliminary safety analysis report. Section 15.2.4k. “Steam Generator Tube Rupture 2 Tubes in 1 SG (State A)”. Edition 2003. AREVA. 2003. (E).

14	14.5	REF0036		EPR FA3 Preliminary Safety Analysis Report, Section 15.2.4q. Isolatable break in the RIS system in RRA mode (to DN 250) outside or inside of the enclosure (states C, D). Edition 2006. AREVA. 2006. (E)
14	14.5	NFPSRDC1080	A	L Foucart. Justification of the 2A-RHR break as a "specific study" – Functional aspect. NFPSR DC 1080 Revision A. AREVA. April 2005. (E).
14	14.5	ECEF080499	B1	Functional study on the treatment of PCCs and RRC-As involving spent fuel pool cooling loss and draining. ECEF080499 Revision B1. EDF. November 2012. (E)
14	14.5	SFLEFMF2006712	G1	System Design Manual - Fuel Pool Cooling System (PTR [FPPS/FPCS]) - Part 2 System Operation. SFL–EFMF 2006.712 Revision G1. Sofinel. August 2009. (E)
14	14.5	NEPCFDC164	B	Residual Decay Heat Curves for Major Components Design Purposes Heat Load inside Fuel Pool. NEPC-F DC 164 Revision B. AREVA NP. November 2008. (E)
14	14.5	NESSFDC546	A	System Design Manual - Safety Injection and Residual Heat Removal System - Part 4: Flow diagrams. NESS-F DC 546 Revision A. AREVA. June 2009. (E)
14	14.5	SFLEFMF2006751	F1	System Design Manual - Fuel Pool Purification and Cooling Systems (PTR [FPPS/FPCS]) Part 5 – Instrumentation and Control SFL EF MF 2006.751 Revision F1. Sofinel. September 2009. (E)
SUB-CHAPTER 14.6 – REFERENCES				
14	14.6	REF0008		Technical Guidelines for the design and construction of the next generation of nuclear power plants with pressurised water reactors – Adopted during the GPR/German experts plenary meetings held on October 19th and 26th 2000 (E)
14	14.6	ICRPPub63		ICRP Publication 63. Principles for Intervention for Protection of the Public in a Radiological Emergency. A report by a task group of Committee 4 of ICRP, adopted by the ICRP in November 1992, Oxford, New York, Seoul, Tokyo, 1993 (E)
14	14.6	L2111		Council Decree dated December 22, 1987, Specifying Maximum Values for Radioactivity in Foodstuffs and Animal Feeds in the Event of a Nuclear Accident or Other Radiological Emergency, Official Journal of the European Community No. L371/11, amended on July 18, 1989. Official Journal of the European Community No. L211/1 (E)
14	14.6	ORNLTM200539	6	C. Gauld, O. W. Hermann, R. M. Westfall, ORIGEN-S: Scale System Module to calculate fuel depletion, actinide transmutation, fission product buildup and decay, and associated radiation source terms. NUREG/CR-200, ORNL/TM-2005/39 Version 6 Vol. II, Sect. F7. Oak Ridge National Laboratory, 2002 (E)
14	14.6	REF0037		Incident Calculation Bases for the Guidelines Issued by the Federal Minister of the Interior (BMI) for the assessment of the design of PWR Nuclear Power Plants pursuant to Sec. 28, para. (3) of the Radiological Protection Ordinance (StrlSchV). 1989 (E)
14	14.6	NUREGCR2139		Sutter, S.L.: Johnston, J.W., Mishima, J.: Aerosols generated by free fall spill of powders and solutions in static air. NUREG/CR 2139. 1981 (E)

14	14.6	15006855		Forschungsvorhaben Reaktorsicherheit Abschlussbericht Förderungsvorhaben BMFT [Research project nuclear reactor safety; Final report of the financially funded project BMFT] 1500 685/5
14	14.6	R91484008		K. H. Neeb; W. Morell; F. Richter; E.Haas; W. Kastner; R. Rippel; G. Roebig; R. Ahrens-Botzong Experimentelle Untersuchungen zur Abgabe von paltprodukten (insbesondere von Jod und Cäsium) bei Auslegungsstörfällen-Teilvorhaben I [Experimental investigation on the release of fission products (in particular of iodine and caesium) during design basis accidents – Sub project I] R/914/84/008 - Kraftwerk Union AG - April 1984
14	14.6	BMFT15007701		Forschungsvorhaben Reaktorsicherheit Abschlussbericht Förderungsvorhaben BMFT [Research project nuclear reactor safety; Final report of the financially funded project BMFT] 1500 770/1.
14	14.6	E14190200		Fr. Sieghard Hellmann; Alois Bleier; Reinhard Rippel; Joachim Funcke - Experimentelle Untersuchungen zur Abgabe von Spaltprodukten (insbesondere von Jod und Cäsium) bei Auslegungsstörfällen (Wirkleitungsbruch) - Teilvorhaben II [Experimental investigation on the release of fission products (in particular of iodine and caesium) during design basis accidents (rupture of a differential pressure pipe) - Sub project II] E 141/90/200 - Siemens Bereich Energieerzeugung (KWU) - November 1990
14	14.6	ICRP60		International Commission on Radiological Protection: Recommendations of the International Commission on Radiological Protection; Publications No: 56 (Age-Dependent Doses to the Public from Intake of Radionuclides; Part 1; 1990), 60 (1990 Recommendations of the International Commission on Radiological Protection, 1992), 67 (Age-Dependent Doses to Members of the Public from Intake of Radionuclides; Part 2; Ingestion Dose Coefficients 1990; 1994), 69 (Age-Dependent Doses to Members of the Public from Intake of Radionuclides; Part 3; Ingestion Dose Coefficients ;1995), 71 (Age-Dependent Doses to Members of the Public from Intake of Radionuclides; Part 4; Ingestion Dose Coefficients ;1996), 72 (Age-Dependent Doses to Members of the Public from Intake of Radionuclides; Part 5; Compilation of Ingestion and Inhalation Dose Coefficients ;1996) (E)
14	14.6	9629EURATOM	Vol39	Council Directive 96/29/Euratom dated May 13 1996 which sets out the basic standards relating to the health protection of the population and worked against the dangers arising from ionising radiation. Official Journal of the European Communities, L159; Vol. 39; June 1996 (E)
14	14.6	EUR14179		Stephenson, W., Dutton, L.M.C., Handy, B. J., Smedley, C. Realistic Methods for Calculating the Releases and Consequences of a Large LOCA, Commission of the European Communities; Contract No. ETNU-0001/UK; Final Report, EUR 14179; 1992 (E)
14	14.6	REF0038		Kabat, M.J.: Chemical behaviour of radioiodine under loss of coolant accident condition, 16th DOE Nuclear Air Cleaning Conference, San Diego 1980 (E)
14	14.6	NUREGCR6613 / SAND970594	Vol1	D. Chanin, M.L. Young: Code Manual for MACCS2: Volume 1, User's Guide, NUREG/CR-6613, SAND97-0594, Vol. 1, Sandia National Laboratories, Albuquerque, NM, USA (1998) (E)
14	14.6	ENTEAG090116	A	G Fleury. Methodology for evaluating the radiological consequences of an accidental atmospheric release. ENTEAG090116 Revision A. EDF. April 2009. (E) ENTEAG090116 Revision A is the English translation of ENTEAG030152 Revision B.

14	14.6	DSN440		Doury. Le vademecum des transferts atmospheriques [Atmospheric Transfer Handbook]. Report DSN n° 440. February 1992
14	14.6	ENTEAG090113	A	P Jan. PSAR EPR - Sensitivity analysis about radiological consequences in fault situations ENTEAG090113 Revision A. EDF. April 2009. (E) ENTEAG090113 Revision A is the English translation of ENTEAG050091 Revision B.
14	14.6	ENFPIN0100177	A1	C Cherbonnel. Hypothesis to be used for evaluating the radiological consequences of design basis accident – Analysis of the ability of the different ventilation systems to reduce the fission products releases. ENFPIN/0100177 Revision A1. EDF. September 2009. (E) ENFPIN/0100177 Revision A1 is the English translation of ENFPIN/0100177 Revision A.
14	14.6	ENTERP090062	A	Primary Source Term of the EPR Reactor. ENTERP090062 Revision A. EDF. March 2009. (E) ENTERP090062 Revision A is the English translation of ENTERP070147 Rev A.
14	14.6	ENTERP090128	A	G Ranchoux. Primary Source Term of EPR Reactor: Additional Information. ENTERP090128 Revision A. EDF. May 2009. (E)
14	14.6	ENTEAG080029	A	P Jan. UK-EPR – Radiological consequences for SGTR (PCC3 and PCC4. ENTEAG080029 Revision A. EDF. June 2008 (E)
SUB-CHAPTER 14.7 - REFERENCES				
14	14.7	FRACTEC01	E	Prouillac. Justification of the internal events list for the design conditions of the NSSS. Fratec 01 Revision E. June 1980. AREVA. (E)
14	14.7	NLEFDC124	B	P. Martinet. Reactor Trip Concept. NLE-F DC 124 Revision B. June 2008. AREVA. (E).
14	14.7	NEPRFDC469	A	Carfantan. EPR UK - Definition of P/S I&C functions. NEPR-F DC 469 Revision A. April 2009. AREVA. (E)
APPENDIX 14A – REFERENCES				
14	App14A	EMF2100	6	S-RELAP5 Models and correlation. Code Manual EMF-2100 Revision 6 (E)
14	App14A	KWUNDS1971002		Validation of the S-RELAP5 program. KWU NDS1/97/1002 (E)
14	App14A	NGPS12004en0507	A	Topical Report - COCO containment code - Model description and validation. NGPS1/2004/en/0507 Revision A. AREVA. October 2004. (E)
14	App14A	R1585e1008		NLOOP – A multiple Loop Code to Determine PWR Plant Transients – KWU Technical Report R15/85/e 1008. August 1985. (E)
14	App14A	KWUR211892012		Beck. Verifikation des DE-Modelles UDEMO1 mit NLOOP (KONVOI-Version) – Nachrechnung des KKI 2-IBS-Versuches D-100-301 Reaktorschnellabschaltung". Siemens Arbeitsbericht [Verification of the steam generator model UDEMO1 with the NLOOP code (KONVOI-Version) – Post-test calculation of the KKI-2 (Kernkraftwerk Isar 2) commissioning test D-100-301 "reactor trip",] KWU R211-89-2012. November 1989.

14	App14A	R15842137	A	Seitz, Hofmann. Verifikation des rechenprogramms NLOOP für die Anlage Biblis-B am Verlauf der Störung – TUSA ohne vom 08.02.84 [Verification of the NLOOP code for the Biblis-B Nuclear Power Station using a turbine trip incident without main steam by pass which occurred on Feb. 8. 1984] KWU Arbeitsbericht R15-84-2137. October 1984.
14	App14A	KWU7470RT51		Rirmer, Druckhalter. Untersuchung KWO [Pressurizer. Assessment KWO (Kernkraftwerk Obrigheim)].KWU Arbeitsnotiz 74/70 – RT 51. April 1970 .
14	App14A	R15862031		Schwarz. Verifikation des Rechenmodells NLOOP für die Anlage GKN I am Beispiel “Ausfall einer Hauptkühlmittelpumpe” [Verification of the NLOOP code for the GKN I (Gemeinschaftkraftwerk Neckar 1) Nuclear power station against the incident “Loss of one main coolant pump”] KWU Arbeitsbericht R15-86-2031. February 1986.
14	App14A	U811892046		Schwarz. Verifikation des Rechenmodells NLOOP am Beispiel “Ausfall einer HKMP” [Verification of the NLOOP code against the incident “Loss of one main coolant pump”] Siemens Arbeitsbericht U8 11-89-2046.
14	App14A	U811882031		Hirmer. Parameteruntersuchung zum Bruch einer HKMP-Welle im 4-LOOP-Leistungsbetrieb. [Parametric examination of the break of the shaft of in one main coolant pump during 4-LOOP power operation] KWU Arbeitsbericht U8 11-88-2031
14	App14A	R15842024		Oelmann, Zen. Verifikation des Transientenprogramms NLOOP am Beispiel des in Grafenrheinfeld durchgefürten IBS-Versuchs “Abschaltung 1v4 Hauptkühlmittelpumpen” [Verification of the of the transient code NLOOP against the commissioning test “Switch-off of one out of four main coolant pumps”, conducted at the Grafenrheinfeld Nuclear Power Station] KWU Arbeitsbericht R15/84/2024.
14	App14A	R15862070		Schwarz. Untersuchungen zum Fehlfahren des FD-Schiebers RA01 S002 vom 02.08.1985 [Examination of spurious actuation of the main steam isolation valve RA01 S002, which occurred on August 2, 1985,] KWU Arbeitsbericht R15-86-2070.August 1985
14	App14A	KWUNDS194E2552		Richter. Validation of the CNT1 Specific NLOOP Code by Comparison to Commissioning Test D100-301. Siemens Work report KWU NDS1/94/E2552 (E)
14	App14A	KWUR211892024		Beck. Verifikation des DE-Modelles UDEMO 1 mit NLOOP (KONVOI-Version) – Nachrechnung des KKE-IBS-Versuches D-080-633 “Ausfall 2v2 Hauptspeisewasserpumpen ohne Zuschalten der Reservepumpe”. [Verification of the steam generator model UDEMO1 with NLOOP (KONVOI-Version) – Post-test calculation of the KKE (Kernkraftwerk Emsland) commissioning test D-080-633 “Loss of two out of two main feedwater pumps without using the back-up pump”] Siemens Arbeitsbericht KWU R211-89-2024.
14	App14A	ST14872125		Weidner. Nachrechnung der DAF-Transiente vom 04.07.1986 zur Verifikation von NLOOP [Post-calculation of the Pressure Drop (DAF)-Transient, which occurred on Juli 4, 1986, in support of the verification of NLOOP] KWU Arbeitsbericht ST14-87-2125.
14	App14A	KWUR211902039		Amm. Nachrechnung des KWG-IBS-Versuches D-100-305 “Lastabwurf auf Eigenbedarf” mit dem Rechenprogramm NLOOP. [Post-test calculation of the of the KWG (Kernkraftwerk Grohnde) commissioning test D-100-305 “Switch-over to house-load operation”, with the NLOOP code] Siemens Arbeitsbericht KWU R211-90-2039.

14	App14A	R15832153		Dr. Gerth. Vergleich NLOOP mit LOOP7 am Beispiel “Fehlöffnen der Frischdampfumleitstation ohne Regelungs-, Begrenzungs- u. Schutzmaßnahmen” [Comparison of NLOOP and LOOP 7 using the incident “Spurious opening of the main steam bypass without reactor control, limitation and protection functions] KWU Arbeitsbericht R15/83/2153
14	App14A	R15832138		Dr. Gerth. Vergleich der Ergebnisse von RELAP5 – mit NLOOP-Rechnungen für eine Reaktorschnellab-schaltung einer 1300-MW [Comparison of RELAP 5 and NLOOP predictions for a reactor trip of a 1300 MW PWR -DWR-Anlage] KWU Arbeitsbericht R15/83/2138.
14	App14A	R15852106		Schwarz, Hofmann. Bruch einer Hauptkühlmittelpumpenwelle – Nachrechnung der Störung vom 06.05.1985 [Break of a main coolant pump shaft – Post-calculation of an incident occurring on May 6, 1985] KWU Arbeitsbericht R15-85-2106.
14	App14A	U811892196a		Hirmer. Nachrechnung von Druckhalter-Abblaseversuchen [Post-test calculations of pressurizer blow-down tests] Siemens Arbeitsbericht U8 11/89/2196a
14	App14A	ST1487e2137		Oelmann. Verification of the NLOOP code by Comparison with the Plant Transient “Loss of External Load, 84-01-13” – KWU Work report ST14-87-e2137. (E)
14	App14A	KWUE41191E2045a		Richter. Verification of the NLOOP Code by Comparison to Commissioning Test IL01and RELAP Verification Calculation. Siemens Work report KWU E411/91/E2045a (E)
14	App14A	ST1487e2138		Richter. Verification of the NLOOP Code by Comparison with the Plant Transient “Trip of Turbine 21, 25.04.84” – KWU Work report ST14-87-e2138. April 1984. (E)
14	App14A	KWUE41191E2044		Richter, Oelmann. Verification of the NLOOP Code by Comparison to Commissioning Test RP00. Siemens Work report KWU E411/91/E2044. (E)
14	App14A	KWUE411912083		Hartlieb. Nachrechnung der Betriebstransiente “Ausfall einer Speisepumpe führt zur Reaktorabschaltung”. [Post-calculation of the operational transient “Loss of one main feed-water pump resulting in reactor trip] Siemens Arbeitsbericht KWU E411/91/2083.
14	App14A	ST1487e2168		Dr. Gerth, Dr. Ro. Verification of the Korea specific Version of NLOOP by recalculation of the “Station Blackout Accident” at KNU 1 – KWU Work-Report ST14-87-e2168. (E)
14	App14A	KWUE411912066		Richter. Vergleichsrechnung zu dem PKL III versuch “A5.2, totaler Ausfall des Speisewassers bei einem intakten DE” mit dem modifizierten Rechenprogramm NLOOP [Comparison of results from the modified NLOOP code with the test outcome of the PKL III experiment “ A5.2 Complete Loss of main feedwater in an intact steam generator]. Siemens Arbeitsbericht KWU E411/91/2066.
14	App14A	A1C13338060		R. Van Geermert, M. Grüner. Code Manual PANBOX 3.5. AREVA Technical Report A1C-1333806-0. November 2007. (E)
14	App14A	A1C13176460		R. Müller. Code Manual COBRA 3-CP 2.02. AREVA Technical Report A1C-1317646-0. October 2005. (E)
14	App14A	EPDSDC150	C	S. Ebalard. ORIGEN-S: Qualification manual. EPDS DC 150 Revision C.AREVA. November 2005. (E)

14	App14A	NGPS42004de0012		Röbig G., Merkel A. Modifikation und Validierung des Rechenprogramms PRODOS (probabilistische Dosisberechnung) – Version: PRODOS-C. [Modification and validation of the PRODOS (probabilistic dose calculation) code – Version: PRODOS-C]. NGPS4/2004/de/0012 AREVA, 2004.
14	App14A	NUREGCR6613 / SAND970594	Vol1	D. Chanin, M.L. Young: Code Manual for MACCS2: Volume 1, User's Guide, NUREG/CR-6613, SAND97-0594, Vol. 1, Sandia National Laboratories, Albuquerque, NM, USA (1998) (E)
14	App14A	EPDDC490	C	A Barbier. CATHARE 2 – Code synthetic qualification assessment. EPD DC 490 Revision C. AREVA. September 2007. (E)
14	App14A	EPTADM0482	B	F. Goux. THEMIS - Qualification report. EP/TA/DM.482 Revision B. AREVA. August 1995. (E)
14	App14A	NFPSDDC89	A	Science V2 Nuclear Code Package - Qualification Report. NFPS-D DC 89 Revision A, AREVA, March 2004, (E)
14	App14A	NFPSDDC188	A	M N Juillon. Qualification Report: FLICA IIIF – Version 3: NFPSD DC 188 Revision A. AREVA. January 2006. (E)
14	App14A	EPTCDM407		Technical description of FLICA III F - Justification for choice of basic models. EP/TC/DM.407 Revision A. AREVA. February 1991. (E)
14	App14A	EPRXDM77	A	B. Marchal. CONPATE4 – Qualification Report. EPRX DM 77 Revision A. AREVA November 1993. (E)
14	App14A	ENTEAG090128	A	M H Boschiero, F Herber, G Depond. PAREO 8 code – Sensitivity studies and validation of the physical model. ENTEAG090128 Revision A. EDF. May 2009. (E) ENTEAG090128 Revision A is the English translation of ESETC8207 Revision A.
14	App14A	NFPSDDC85	D	S. Balleydier. MANTA Code synthetic qualification assessment. NFPSD DC 85 Revision D. AREVA. September 2008. (E)
14	App14A	EPRRDC1035	D	F. Thiel. EPR Basic Design Optimization Phase – RPV Thermal-hydraulic Design: Flow and Head Loss Distribution in the RPV. EPRR DC 1035 Revision D. AREVA. November 1998. (E)
14	App14A	NEPDFDC10157	A	Adapted from report: Functional validation of the MANTA-SMART-FLICA coupling. NEPD-F DC 10157 Revision A. AREVA. October 2008. (E)
14	App14A	EPDDC259	C	COMBAT synthesis qualification report. EPD DC 259 Revision C. AREVA. (E)
APPENDIX 14C – REFERENCES				
14	App14C	NFPSDDC85	D	S. Balleydier. MANTA Code synthetic qualification assessment. NFPSD DC 85 Revision D. AREVA. September 2008. (E)
14	App14C	NFPSDDC89	A	SCIENCE V2 Nuclear Code Package - Qualification Report. NFPSD DC 89 Revision A. AREVA. March 2004. (E)
14	App14C	NFPSDDC188	A	M N Jullion. Qualification Report FLICA IIIF - Version 3. NFPSD DC 188 Revision A. AREVA. January 2006. (E)
14	App14C	PSRRDC13	B	M Petiard – H. Lafitte. EPR Preliminary Safety Analysis Report, Section 15.2.4b. 1- Steam system piping break (State A). Edition 2003. PSRR DC 13 Revision B. AREVA. November 2003. (E)
14	App14C	NFEPCDC128	B	P Dormeau. Neutronic data for transient analyses. NFEPC DC 128 Revision B. AREVA. December 2003. (E)

14	App14C	EPCDC1690	C	P Dormeau. EPR RF002 – NSSS operating parameters. EPC DC 1690 Revision C. AREVA. September 2003. (E)
14	App14C	EPRRDC1685	C	R Gagner. EPR sizing at 4500MWth. EPRR DC 1685 Revision C. AREVA. February 2004. (E)
14	App14C	NEEGFDC1210	F	D Venturi. 79/19TE Steam generator secondary side pressure vessel details right and left SGs. NEEG-F DC 1210 Revision F. AREVA. April 2008. (E)
14	App14C	EPRRDC1651	D	N. Goreaud. EPR – Lower Plenum Hydraulics Design of the Flow Distribution Device. EPRR DC 1651 Revision D. AREVA. December 2001. (E)
14	App14C	NFPSCDC286	B	S Laurent. Neutronic Data for Transient Analyses (Update 4500 MWth). NFPSC DC 286 Revision B. AREVA. February 2006. (E)
14	App14C	NFPSRDC1057	B	H Lafitte. Main Steam Line Break (States A,B). NFPSR DC 1057 Revision B. AREVA. June 2005.
14	App14C	NEPDFDC10157	A	Functional Validation of MANTA-SMART-FLICA Coupling. NEPD-F DC 10157 Revision A. AREVA. October 2008. (E)
CHAPTER 15				
SUB-CHAPTER 15.0 – REFERENCES				
15	15.0	SAP2006	1	Safety Assessment Principles for Nuclear Facilities. 2006 Edition Revision 1. UK Health and Safety Executive (HSE). January 2008. (E)
15	15.0	IAEA50P4		Procedures for Conducting Probabilistic Safety Assessments of Nuclear Power Plants (level 1). Safety Series No 50-P-4. IAEA. August 1992. (E)
15	15.0	EUR1995	B	European Utility Requirements for LWR Nuclear Power Plants, Volume 2: Generic Nuclear Island Requirements, Chapter 17: PSA Methodology, Revision B. EUR Document. November 1995. (E)
SUB-CHAPTER 15.1 – REFERENCES				
15	15.1	REF0039		RELCON SCANDPOWER AB. “RiskSpectrum® PSA Professional 2.10, Quality Manual, Test protocols and Release Approval”. 2006. (E)
15	15.1	BDR1999		EPR Basic Design Report. AREVA. February 1999. (E)
15	15.1	NEPSFDC565	A	H. Gentner.Summary of the input data for the UK EPR Probabilistic Safety Assessment. NEPS-F DC 565 Revision A FIN. AREVA. May 2010. (E)
15	15.1	VGBTW804e		Centralised Reliability and Events Database (ZEDB) – Reliability Data for Nuclear Power Plant Components – Analysis for 2004. Report VGB-TW804e. VGB PowerTech Service GmbH. ISSN 1439-7498.
15	15.1	VGBTQ805		Centralised Reliability and Events Database (ZEDB) – Reliability Data for Nuclear Power Plant Components – December 2006. Report VGB-TW805. VGB PowerTech Service GmbH. ISSN 1439-7498. (E)

15	15.1	EGGSSRE8875		S.A. Eide, S.V. Chmielewski, and T.D. Swantz. EG&G - Generic Component Failure Data Base for Light Water and Liquid Sodium Reactor PRAs. EG&G Idaho Inc., Idaho National Laboratory. EGG-SSRE-8875. February 1990.
15	15.1	EUR1995	B	European Utility Requirements for LWR Nuclear Power Plants, Volume 2: Generic Nuclear Island Requirements, Chapter 17: PSA Methodology, Revision B. EUR Document. November 1995. (E)
15	15.1	NEPSFDC576	A	C. Leroy. UKEPR: Description of the C&I modelling in the PSA. NEPS-F DC 576 Revision A FIN. AREVA. July 2010. (E)
15	15.1	NEPSFDC191	A	F. Godefroy. Human Reliability Analysis Notebook of the UK EPR Probabilistic Safety Assessment. NEPS-F DC 191 Revision A FIN. AREVA. January 2010. (E)
15	15.1	NUREGCR4772		A D Swain. Accident Sequence Evaluation Program Human Reliability Analysis Procedure. NUREG/CR-4772. Sandia National Laboratories, Albuquerque, New Mexico. February 1987.
15	15.1	SFLEFMF2006829	E1	System Design Manual - Emergency Feedwater System (EFWS) P2 – System operation SFL–EFMF 2006.829 Revision E1. Sofinel. September 2009. (E)
15	15.1	NEPRFDC241	B	A. Drevet. OL3 PSA Support Studies. NEPR-F DC 241 Revision B FIN. AREVA/SIEMENS. June 2008. (E)
15	15.1	NEPSFDC355	C	UK EPR. Probabilistic Safety Analysis Level 1 Detailed Documentation. NEPS-F DC 355 Revision C FIN. AREVA. June 2010 (E)
15	15.1	NEPSFDC377	B	S. Kahia. Analysis of the Completeness of Scope of Initiating Events for the GDA step 3 UK Probabilistic Safety Assessment. NEPS-F DC 377. Revision B. AREVA. December 2009. (E)
15	15.1	IAEATECDOC719		Defining Initiating Events for Purposes of Probabilistic Safety Assessment. IAEA TECDOC-719. IAEA, Vienna. 1993.
15	15.1	ENFPFC0200484	A	V Sorel. EPR PSA Assessment of initiating event frequencies. ENFPFC0200484 Revision A. EDF. February 2003. (E)
15	15.1	REF0042		Probabilistic Safety Assessment of reactor unit 3 in the Paluel Nuclear Power Center (1300MWe), Overall Report. EDF. May 1990. (E)
15	15.1	REF0043		Etudes probabiliste de sureté des reacteurs à eau sous pression de 900 MWe, Rapport de systhèse [Probabilistic Safety Assessment of 900 MWe (French) Pressurised Water Reactors]. IPSN. April 1990.
15	15.1	NUREGCR3862		Development of Transient Initiating Event Frequencies for Use in Probabilistic Risk Assessment. NUREG/CR-3862. NRC. May 1985. (E)
15	15.1	NUREGCR6928		Industry-Average Performance for Components and Initiating Events at U.S. Commercial Nuclear Power Plants. NUREG/CR-6928. NRC. February 2007. (E)
15	15.1	NUREG1829		Estimating Loss-of-Coolant Accident (LOCA) Frequencies Through the Elicitation Process. NUREG-1829. NRC. April 2008. (E)
15	15.1	FA3PSAR181		FA3 EPR - Preliminary Safety Analysis Report, Chapter 18.1 “Probabilistic Safety Analysis – level 1”. EDF. 2006. (E)
15	15.1	SAP2006	1	UK Health and Safety Executive (HSE). Safety Assessment Principles for Nuclear Facilities. 2006 Edition Revision 1. January 2008. (E)

15	15.1	NEPSFDC109	C	V. Roux. EPR - Reliability Study of the FA3 SEMPELL Pressurizer Safety Relief Valves. NEPS-F DC 109 Revision C. AREVA. October 2007. (E)
15	15.1	EPSEDC833	F	F. Satin. EPR – Probabilistic analysis of accident sequences caused by interfacing loss of coolant accidents. EPSE DC 833 Revision F. AREVA NP. March 2006. (E)
15	15.1	REF0097		Der Bundesminister für Forschung und Technologie (BMFT) (Hrsg.): Deutsche Risikostudie Kernkraftwerke, Phase B. Verlag TÜV Rheinland GmbH, Köln, 1990. [Minister of Research and Technology: German Safety Analysis of Nuclear Power Plants, Phase B. Publisher TÜV Rheinland GmbH. Cologne. 1990.]
15	15.1	NUREGCR6365		Steam Generator Tube Failure. NUREG/CR-6365. NRC and Idaho National Engineering Laboratory. April 1996. (E)
15	15.1	SXBIP096233		A Reassessment of the Frequency and Duration of Loss of Off-Site Power. SXB-IP-096233. April 1991. (E) (This document is a National Grid Report: NGD/SD/4/89. August 1990.)
15	15.1	NUREGCR6890	Vol1	Re-evaluation of Station Blackout Risk at Nuclear Power Plants, Analysis of Loss of Offsite Power Events: 1986 – 2004. NUREG/CR-6890 Volume 1. December 2005. (E)
15	15.1	NGPS42003en0120	B	Heterogeneous Boron Dilution - PSA demonstration of dilution accident practical elimination. NGPS4 2003 en 0120 Revision B. AREVA, TR 04/138. December 2003. (E)
15	15.1	PSSEDC901	C	C. Duval. EPR – Probabilistic Analysis of Accident Sequences Caused by Anticipated Transients Without Scram. PSSE DC 901 Revision C. AREVA. October 2004. (E)
15	15.1	IAEA50P4		Probabilistic Safety Assessments of Nuclear Power Plants (Level 1). Safety Series No. 50-P-4. IAEA. (E)
SUB-CHAPTER 15.2 – REFERENCES				
15	15.2	NUREG1774		"A Survey of Crane Operating Experience at U.S. Nuclear Power Plants from 1968 through 2002". NUREG-1774. July 2003. (E)
15	15.2	NFPMRDC1048	B	Reactor Consequences Assessment of a RPV Closure Head Accidental Drop during Lifting Operation. Topical Report No. 98. NFPMR DC 1048 Revision B FIN. AREVA. (E)
15	15.2	IAEA50P4		Probabilistic Safety Assessments of Nuclear Power Plants (Level 1). Safety Series No. 50-P-4. IAEA. (E)
15	15.2	REF0015	C	European Utility Requirements for LWR Nuclear Power Plants, Volume 2: Generic Nuclear Island Requirements, Chapter 17: PSA Methodology, Revision C. (E)
15	15.2	NUREGCR2300		PRA Procedures Guide. A Guide to the performance of probabilistic risk assessment for nuclear plants. NUREG/CR-2300. Office of Nuclear Regulatory Research U.S. Nuclear Regulatory Commission. (E)
15	15.2	NUREGCR6850		Fire PRA Methodology for Nuclear Power Facilities. NUREG/CR-6850. Electrical Power Research Institute (EPRI), Palo Alto, CA, and U.S. Nuclear Regulatory Commission, Office of Nuclear Regulatory Research (RES), Rockville, EPRI/NRC-RES. (E)

15	15.2	ENGSIN050312	B	M.Mosse. ETC-F. EPR Technical Code for Fire Protection. ETC-F Version G. ENGSIN050312 Revision B. EDF. August 2007. (E)
15	15.2	NESPG2009EN1322	C	Fire Safety Assessment of Safety Fire Barriers- Evaluation of fire resistance using experience of comparable EPR layouts. NESP-G/2009/en/1322 Revision C. AREVA. March 2011. (E)
15	15.2	EPRITR102266		Pipe Failure Study Update. EPRI TR-102266. April 1993. (E)
15	15.2	EGGSSRE9639		Component Leakage and Rupture Frequency Estimates. EG&G-SSRE—9639. Idaho National Engineering Laboratory. November 1991. (E)
15	15.2	REF0002	2	Nuclear Power Station Generic Design Assessment - Guidance to Requesting Parties. Version 2. UK Health and Safety Executive (HSE). July 2007. (E)
15	15.2	IAEA50P7		Treatment of External Hazards in Probabilistic Safety Assessment for Nuclear Power Plants. IAEA Safety Series 50-P-7. IAEA. (E)
15	15.2	IAEA50SGD5		External Man-Induced Events in Relation to Nuclear Power Plants: A Safety Standard. IAEA Safety Series 50-SG-D5, 1996. IAEA.(E)
15	15.2	IAEA50SGS11A		Extreme Meteorological Events in Nuclear Power Siting, Excluding Tropical Cyclones – A Safety Guide. IAEA Safety Series 50-SG-S11A. IAEA. (E)
15	15.2	NUREGCR5042		Evaluation of External Hazards to Nuclear Power Plants in the United States. NUREG/CR-5042. Lawrence Livermore National Laboratory, NRC. December 1987. (E)
15	15.2	1501997		J P Byrne. The calculation of aircraft crash in the UK. AEA Technology plc, Contract Research Report 150/1997. (E)
15	15.2	REF0044		PSAR Flamanville DAC, Chapter 3.3, Protection against external hazards. 2006. (E)
15	15.2	EPSEDC833	F	EPR – Probabilistic analysis of accident sequences caused by interfacing loss of coolant accidents. EPSE DC 833 Revision F. AREVA NP. March 2006. (E)
15	15.2	ECEF051311	A	EPR – Study of the reliability of the pumping station for the Flamanville site – Quantification of initiating events for total loss of cooling water. Technical Report ECEF051311 Revision A. EDF. (E)
15	15.2	NUREGCR5042		Evaluation of External Hazards to Nuclear Power Plants in the United States. NUREG/CR-5042. Lawrence Livermore National Laboratory, NRC. December 1987. (E)
15	15.2	NUREG1407		Procedural and Submittal Guidance for the Individual Plant Examination of External Events (IPEEE) for Severe Accident Vulnerabilities. NUREG-1407. NRC. June 1991. (E)
SUB-CHAPTER 15.3 – REFERENCES				
15	15.3	SFLEFMF2006751	F1	System Design Manual - Fuel Pool Purification and Cooling Systems (PTR [FPPS/FPCS]) Part 5 – Instrumentation and Control SFL EF MF 2006.751 Revision F1. Sofinel. November 2009. (E)
15	15.3	SFLEFMF2006712	G1	System Design Manual - Fuel Pool Cooling Systems (PTR [FPPS/FPCS]) Part 2 System Operation, SFL–EF-MF 2006-712 Revision G1. Sofinel. August 2009. (E)

15	15.3	ECEF080499	B1	Functional study on the treatment of PCCs and RRC-As involving spent fuel pool cooling loss and draining. ECEF080499 Revision B1. EDF. November 2012. (E)
15	15.3	FA3PSAR1832		EDF FA3 EPR - Preliminary Safety Analysis Report, Chapter 18.3.2, "Accidents in the Spent Fuel Pool". 2006. (E)
15	15.3	ENFCFF040206	C	Long term probabilistic analysis of Loss Of Off-site Power (LOOP) and Loss of Ultimate Heat Sink (LUHS) situations. EDF/SEPTEN Report ENFCFF040206 Revision C. March 2006. (E)
15	15.3	NEPSFDC565	A	Summary of the input data for the UK EPR Probabilistic Safety Assessment. NEPS-F DC 565 Revision A. AREVA. May 2010. (E)
15	15.3	RESS1988		G W Hannaman et al. Some Developments in Human Reliability Analysis Approach and Tools, Reliability Engineering and Tools. Reliability Engineering and Systems Safety 1988, Vol. 22, 235-256. (E)
SUB-CHAPTER 15.4 – REFERENCES				
15	15.4	NEPSFDC526	A	UK EPR Level 2 Supporting Analysis - Containment Fragility. NEPS-F DC 526 Revision A FIN. AREVA. December 2009. (E)
15	15.4	REF0047		EU/SARNET/NEA Joint Training Short Course on Nuclear Severe Accident Analysis: Application and Management Guidelines, Session 21 Overview of OSSA, Budapest, Hungary, 7-11 April 2008. (E)
15	15.4	NEPSFDC527	A	UK EPR Level 2 Supporting Analysis - Human Reliability Analysis. NEPS-F DC 527 Revision A FIN. AREVA. January 2010. (E)
15	15.4	NUREGCR6883		The SPAR-H Human Reliability Analysis Method. NUREG/CR-6883. Idaho National Laboratory. August 2005. (E)
15	15.4	MAAP4User		MAAP4 (Modular Accident Analysis Program) User Manual. Electric Power Research Institute (EPRI). (E)
15	15.4	NEPSFDC458	A	UK EPR Level 2 Supporting Analysis – Severe Accident Analysis. NEPS-F DC 458 Revision A. AREVA SAS. August 2009. (E)
15	15.4	REF0039		RELCON SCANDPOWER AB. "RiskSpectrum® PSA Professional 2.10, Quality Manual, Test protocols and Release Approval". 2006. (E)
15	15.4	REF0048		Crystal Ball version 7.2.1 User Guide, Decisioneering, Inc. (E)
15	15.4	XXIX61		W A Stewart, et al. Experiments on Natural Circulation Flows and Steam Generators During Severe Accident. Proceedings of the International ANS/ENS Meeting on Thermal Reactor Safety, San Diego, Paper XXIX-6-1. 1986. (E)
15	15.4	NEPSFDC541	A	UK EPR Level 2 Supporting Analysis - Induced RCS Ruptures. NEPS-F DC 541 Revision A. AREVA. February 2010. (E)
15	15.4	NEPSFDC542	A	UK EPR Level 2 Supporting Analysis – Fuel Coolant Interaction. NEPS-F DC 542. Revision A FIN. AREVA. April 2010. (E)
15	15.4	NUREG1116		A Review of the Current Understanding of the Potential for Containment Failure from In-Vessel Steam Explosions. NUREG-1116 (SERG-1 Report), USNRC. June 1985. (E)

15	15.4	SKI0216		B R Sehgal, H O Haraldsson, Z L Yang. A Review of Steam Explosions with Special Emphasis on the Swedish BWRs and PWRs. SKI Report 02:16. April 2002. (E)
15	15.4	NUREG1524		A Re-assessment of the Potential for an Alpha-Mode Containment Failure and a Review of the Current Understanding of Broader Fuel Coolant Interaction Issues. NUREG-1524. USNRC. August 1996. (E)
15	15.4	ISBN0471186368		P W Cooper. Explosives Engineering. New York: Wiley-VCH. 1996. ISBN 0-471-18636-8. (E)
15	15.4	REF0049		Injection Into a Damage Core During a Severe Accident E. Sauvage (AREVA-NP, France), Presentation at the IAEA Workshop on Analysis Support of EOP and BDBA for Accident Management Ignalina NPP, Lithuania, October 17th-21st, 2005. (E)
15	15.4	NEPSFDC548	A	UK EPR Level 2 Supporting Analysis – In-Vessel Recovery. NEPS-F DC 548 Revision A. AREVA. June 2010. (E)
15	15.4	NEPSFDC567	A	UK EPR Level 2 Supporting Analysis – Vessel Failure. NEPS-F DC 567 Revision A. AREVA. June 2010 (E)
15	15.4	FIKSCT199900011		B R Seghal et al. Assessment of reactor vessel integrity (ARVI). Report on EC contract FIKS-CT1999-00011, KTH, Royal Institute of Technology, Division of Nuclear Power Safety, Stockholm, Sweden. 1999. (E)
15	15.4	RAMAIII8904		W Frid. Containment Severe Accident Thermo-hydraulic Phenomena. RAMA III 89-04, Swedish Nuclear Power Inspectorate, Stockholm, Sweden. August 1991. (E)
15	15.4	NUREGCR6338 / SAND952381		Resolution of the Direct Containment Heating Issue for all Westinghouse Plants with Large Dry Containments or Sub-atmospheric Containments. USNRC Contractor Report, NUREG/CR-6338, SAND 95-2381. January 1996. (E)
15	15.4	SKI0123		B R Seghal. Interactions of Severe Accident Research and Regulatory Positions (ISARRP). SKI Report 01:23. December 2001. (E)
15	15.4	SKI0053		B R Seghal et al. Final report for the Melt-Vessel Interactions Project. SKI Report 00:53, EU 4th Framework MVI Project Final Report. April 1999. (E)
15	15.4	ISSN09478620		L Mayer et al. Melt Dispersion and Direct Containment Heating (DCH) Experiments in the DISCO-H Test Facility. FZK Report FZKA 6988. ISSN 0947-8620. May 2004. (E)
15	15.4	1269034739001		U.S. EPR PRA – MAAP Analysis. 126-9034739-001. AREVA Inc. (E)
15	15.4	NEACSNIR		W Breitung, C K Chan, S B Dorofeev, A Eder, B E Gelfand, M Heitsch, R Klein, A Malliakos, J E Shepherd, E Studer, P Thibault. Flame Acceleration and Deflagration to Detonation Transition in Nuclear Safety,"(State-of-the-Art Report by a Group of Experts). NEA/CSNI/R(2000)7. OECD Nuclear Energy Agency. August 2000. (E)
15	15.4	RAMAIII8904Fig		Figure 9-11 of: W Frid. Containment Severe Accident Thermo-hydraulic Phenomena. RAMA III 89-04, Swedish Nuclear Power Inspectorate, Stockholm, Sweden. August 1991. (E)
15	15.4	NEPSFDC540	A	UK EPR Level 2 Supporting Analysis – Hydrogen. NEPS-F DC 540. Revision A FIN. AREVA. February 2010. (E)
15	15.4	NEPSFDC543	A	UK EPR Level 2 Supporting Analysis – Long-Term Containment Challenges. NEPS-F DC 543 Revision A. AREVA. June 2010 (E)

15	15.4	NUREG0654		Criteria for Preparation and Evaluation of Radiological Emergency Response Plans and Preparedness in Support of Nuclear Power Plants. NUREG-0654. U.S. Nuclear Regulatory Commission. 1980.
15	15.4	NUREG1150		Severe Accident Risks: An Assessment for Five U.S. Nuclear Power Plants. NUREG-1150. U.S. Nuclear Regulatory Commission. 1991.
15	15.4	NEPSFDC544	A	UK EPR Level 2 Supporting Analysis – CET Split Fraction Summary. NEPS-F DC 544 Revision A. AREVA. August 2010.(E)
15	15.4	NEPSFDC459	A	UK EPR Level 2 Supporting Analysis – (MAAP) Supporting Analysis. NEPS-F DC 459. Revision A FIN. AREVA. August 2009. (E)
15	15.4	NEPSFDC493	A	UK EPR Level 2 Supporting Analysis – Source Term Methodology and Identification of Key Uncertainties. NEPS-F DC 493. Revision A. AREVA. September 2009. (E)
15	15.4	NEPSFDC462	A	UK EPR Level 2 Supporting Analysis – Source Term Analysis. NEPS-F DC 462. Revision A FIN. AREVA. August 2009. (E)
15	15.4	NUREG1738		Technical Study of Spent Fuel Pool Accident Risk at Decommissioning Nuclear Power Plants, NUREG-1738. February 2001.(E)
15	15.4	NUREGCR4982		Severe Accidents in Spent Fuel Pools in Support of Generic Issue 82. NUREG/CR-4982. July 1987. (E)
15	15.4	1269057349001		U.S. EPR PRA – Level 3 PRA for DC. 126-9057349-001. AREVA Inc. (E)
15	15.4	BAW2413		Level 3 PRA for Three Mile Island Unit 1" Framatome ANP, Inc. and Duke Energy Corporation (for the B&W Owners Group). BAW-2413. February 2002. (E)
15	15.4	TAST030	4	UK Health and Safety Executive (HSE). Technical Assessment Guide, Probabilistic Safety Analysis. T/AST/030 Issue 004. January 2008, Draft – under editorial review. (E)
15	15.4	SAP2006	1	UK Health and Safety Executive (HSE). Safety Assessment Principles for Nuclear Facilities. 2006 Edition Revision 1. January 2008. (E)
15	15.4	75INSAG3	1	Basic Safety Principles for Nuclear Power Plants, 75-INSAG-3 Revision. 1. INSAG-12. International Atomic Energy Agency. 1999. (E)
15	15.4	NUREGCR6595 BNLNUREG52539	1	An Approach for Estimating the Frequencies of Various Containment Failure Modes and Bypass Events, Final Report. NUREG/CR-6595, Rev. 1 / BNL-NUREG-52539, Revision. 1. Brookhaven National Laboratory, U.S. Nuclear Regulatory Commission, Office of Nuclear Regulatory Research. (E)
15	15.4	NRC1986		Safety Goals for the Operation of Nuclear Power Plants; Policy Statement. Federal Register, Vol. 51, No. 149, U.S. Nuclear Regulatory Commission. August 4, 1986. Appendix A. Definition and Potential Specification of LERF A-4. (E)
15	15.4	MEMOACSR1997		R Sherry. Considerations for Plant-Specific Site-Specific Application of Safety Goals and Definition of Subsidiary Criteria. Memorandum to ACRS Members, June 27, 1997. (E)
15	15.4	ANSCONF1985		G D Kaiser. The Implications of Reduced Source Terms for Ex-Plant Consequence Modelling. ANS Executive Conference on the Ramifications of the Source Term, March 12, 1985, Charleston, SC. (E)

15	15.4	STATE39591		Decision of the [Finnish] Council of State on the general regulations for the safety of nuclear power plants, STATE (395/91), Section 12. (E)
SUB-CHAPTER 15.5 – REFERENCES				
15	15.5	14782TR0001	1	S C Bubb, C Niculae. Screening of Initiating Events to Support the Dose Band Assessment for the Step 3 Submission. AMEC report 14782/TR/0001 Issue 01. June 2008. (E)
15	15.5	ENTEAG080122	A	Release categories for the LCHF / Level 3 PSA (supporting study for PCSR chapter 15). EDF/SEPTEN ENTEAG080122 Revision A. June 2008. (E)
15	15.5	14782TR0002	1	E Grindon. Support Study for the Societal Risk Assessment to Support the Step 3 Submission. AMEC report 14782/TR/0002 Issue 1. June 2008. (E)
15	15.5	REF0051		Safety Review Guidebook for the Gas Cooled Reactors. Nuclear Electric Report. September 1993. (E)
15	15.5	NRPBR137		G N Kelly and R H Clarke. An Assessment of the Radiological Consequences of Releases from Degraded Core Accidents for the Sizewell PWR. NRPB-R137. July 1982. (E)
15	15.5	EUR1995	C	European Utility Requirements for LWR nuclear power plants. Volume 2 - Generic nuclear island requirements. Revision C. EUR document. April 2001. (E)
SUB-CHAPTER 15.6 – REFERENCES				
15	15.6	SAP2006	1	UK Health and Safety Executive (HSE). Safety Assessment Principles for Nuclear Facilities. 2006 Edition Revision 1. January 2008. (E)
15	15.6	TAST013		UK Health and Safety Executive (HSE), Nuclear Safety Directorate. Technical Assessment Guide, "External Hazards". TAG T/AST/013.and associated Safety Assessment Principles: (P16, P21, P33, P42 to P44, P72, P119 to P140). (E)
15	15.6	TAST030		UK Health and Safety Executive (HSE), Nuclear Safety Directorate. Technical Assessment Guide, "Probabilistic Safety Analysis". TAG T/AST/030.and associated Safety Assessment Principles: (P32 to P46, P51 to P53 and P55). (E)
15	15.6	REF0015		European Utility Requirements for LWR Nuclear Power Plants, Volume 2: Generic Nuclear Island Requirements, Chapter 4, Section 2.4.1.2.1 "Seismic Hazard" and Section 2.4.6 "Seismic Design". (E)
15	15.6	SECY93087		Policy, Technical and Licensing Issues Pertaining to Evolutionary and Advanced Light Water Reactors (ALWR) Designs". SECY-93-087. U.S. Nuclear Regulatory Commission. (E)
15	15.6	REF0052	0	M.K. Ravindra. “Seismic Fragilities of Structures and Equipment of UK European Pressurised Reactor for Use in Seismic Margin Assessment”. Revision 0. March 2009 (E)
15	15.6	NEELFDC87	A	UK EPR GDA Seismic Fragilities of Primary Equipment for Use in Seismic Margin Assessment. NEEL-F DC 87 Revision A. AREVA. May 2009. (E)

15	15.6	NUREGCR4334		Budnitz et al. An Approach to the Quantification of Seismic Margins in Nuclear Power Plants. NUREG/CR-4334. U.S. Nuclear Regulatory Commission. (E)
15	15.6	RESS792003		Watanabe, Y., Oikawa, T., Muramatsu, K. Development of the DQFM method to consider the effect of correlation of component failures in seismic PSA of nuclear power plant, Reliability Engineering and System Safety, Vol. 79 (2003), 265-279, 2003. (E)
15	15.6	REF0053		UK Health and Safety Executive (HSE), Nuclear Directorate Assessment Report. New Build, Step 2 EPR Civil Engineering and External Hazard Assessment. (E)
15	15.6	EPRITR-103959		J W Reed and R P Kennedy. Methodology for Developing Seismic Fragilities. EPRI TR-103959. Electrical Power Research Institute, Palo Alto, CA. June 1994. (E)
15	15.6	ANSIANS58212003		External Events Standard. ANSI-ANS-58.21-2003. American Nuclear Society (ANSI). (E)
SUB-CHAPTER 15.7 – REFERENCES				
15	15.7	EUR1995	B	European Utility Requirements for LWR Nuclear Power Plants, Volume 2: Generic Nuclear Island Requirements, Chapter 17: PSA Methodology, Revision B. EUR Document. November 1995. (E)
15	15.7	NEPRFDC241	B	A. Drevet. OL3 PSA Support Studies. NEPR-F DC 241 Revision B FIN. AREVA/SIEMENS. June 2008. (E)
15	15.7	ENFCFF040206	C	Long term probabilistic analysis of Loss Of Off-site Power (LOOP) and Loss of Ultimate Heat Sink (LUHS) situations. ENFCFF040206 Revision C. EDF/SEPTEN. March 2006. (E)
15	15.7	USNRC2007		U.S. Nuclear Regulatory Commission, "CCF Parameter Estimations, 2007 Update", (http://nrcoe.inl.gov/results/CCF/ParamEst2007/ccfparamest.htm). September 2008. (E)
15	15.7	NEPSFDC191	A	F. Godefroy. Human Reliability Analysis Notebook of the UK EPR Probabilistic Safety Assessment. NEPS-F DC 191 Revision A. AREVA. January 2010. (E)
15	15.7	NEPSFDC383	B	EPR – Failure of the Reactor Coolant Pump Seals. NEPS-F DC 383 Revision B. AREVA. 2009. (E)
CHAPTER 16				
SUB-CHAPTER 16.1 – REFERENCES				
16	16.1	NFPSDDC85	D	MANTA Code synthetic qualification assessment. NFPSD DC 85 Revision D. AREVA. September 2008. (E)
16	16.1	NFPSDDC89	A	SCIENCE V2 Nuclear Code Package - Qualification Report. NFPSD DC 89 Revision A. AREVA. March 2004. (E)
16	16.1	NFPSDDC188	A	Qualification Report FLICA IIIF - Version 3. NFPSD DC 188 Revision A. AREVA. (E)
16	16.1	NEPD-F DC 10157	A	Functional Validation of MANTA-SMART-FLICA Coupling. NEPD-F DC 10157 Revision A. AREVA. October 2008. (E)
16	16.1	NFPSCDC285	A	S Laurent. Fuel Management - Neutronic Design Report (Science Calculations) (Update 4500 MWth). NFPSC DC 285 Revision A. AREVA. September 2004. (E)

16	16.1	1912FSa03		EPR Preliminary Safety Analysis Report, Section 19.1.2 FSa "ATWS by Rods Failure [State A]". Edition 2003. AREVA. (E)
16	16.1	1912FSb03		EPR Preliminary Safety Analysis Report, Section 19.1.2 FSb. "ATWS by PS Failure [State A]". Edition 2003. AREVA. (E)
16	16.1	EPRRDC1693		EPR Preliminary Safety Analysis Report, Sub-chapter 15.1 "Plant characteristics assumed in the accident analyses". Edition 2003. EPRR DC 1693. AREVA (E)
16	16.1	1912FSa06		EPR Preliminary Safety Analysis Report, Section 19.1.2 Fsa. "ATWS through Mechanical Locking of the Rods (State A)". Edition 2006. AREVA. (E)
16	16.1	1912FSb06		EPR Preliminary Safety Analysis Report, Section 19.1.2 FSb. "ATWS through Protection System Failure (State A)". Edition 2006. AREVA. (E)
16	16.1	1912FSc06		EPR Preliminary Safety Analysis Report, Section 19.1.2 FSc. "Total Loss of Offsite and Onsite Power Supply (State A)". Edition 2006. AREVA. (E)
16	16.1	1912FSd03		EPR Preliminary Safety Analysis Report, Section 19.1.2 FSd."Total Loss of Feedwater in State A (Failure of 1st PSV or 1 MHSI Pump)". Edition 2003. AREVA. (E)
16	16.1	1912FSd06		EPR Preliminary Safety Analysis Report, Section 19.1.2 FSd."Total Loss of Feedwater Supply to the Steam Generators (State A)". Edition 2006. AREVA. (E)
16	16.1	1912FSe06		EPR Preliminary Safety Analysis Report, Section 19.1.2 FSe "Total Loss of the Cooling Chain leading to an opening at the joints of the Primary Coolant Pumps (State A)". Edition 2006. AREVA. (E)
16	16.1	1912FSf03		EPR Preliminary Safety Analysis Report, Section 19.1.2 FSf. "LOCA (Break Size up to 20 cm^2) with Failure of Partial Cooldown Signal, in State A". Edition 2003. AREVA. (E)
16	16.1	1912FSf06		EPR Preliminary Safety Analysis Report, Section 19.1.2 FSf. "APRP (Opening smaller than 20 cm^2) with Failure of the Partial Cooling Signal (State A)". Edition 2006. AREVA. (E)
16	16.1	1912FSg03		EPR Preliminary Safety Analysis Report, Section 19.1.2 FSg. "LOCA (Break Size up to 20cm^2) without MHSI, in State A". Edition 2003. AREVA. (E)
16	16.1	1912FSg06		EPR Preliminary Safety Analysis Report, Section 19.1.2 FSg. "APRP (Opening smaller than 20 cm^2) without ISMP (State A)". Edition 2006. AREVA. (E)
16	16.1	NGPS12004en0507	A	Topical report – Comparison SRELAP/CATHARE - Topical Report - COCO containment code - Model description and validation. NGPS1/2004/en/0507 Revision A. AREVA. October 2004. (E)
SUB-CHAPTER 16.2 – REFERENCES				
16	16.2	REF0008		Technical Guidelines for the design and construction of the next generation of nuclear power plants with pressurised water reactors. Adopted during the GPR/German experts plenary meetings. October 19th and 26th 2000. (E)

16	16.2	ICRPPub63		Principles for Intervention for Protection of the Public in a Radiological Emergency, A report by a task group of Committee 4 of ICRP, adopted by the ICRP in November 1992, Oxford, New York, Seoul, Tokyo. ICRP Publication 63. ICRP. 1993. (E)
16	16.2	FZK6316		D Struwe et al. Consequence evaluation of In-vessel Fuel Coolant Interaction in the European Pressurised Water Reactor. FZK Final Report No. 6316. 1999. (E)
16	16.2	XP00NSAVN97013		B De Boeck. Introduction to Severe Accidents. O/Ref. 97-2589.DOC, Class. XP.00.NS AVN-97/013. 1997
16	16.2	NEACSNIR9924		Technical Opinion Paper on Fuel-Coolant Interaction. NEA/CSNI/R(99)24. OECD. 1999. (E)
16	16.2	REF0055		Leon Cizelj et al. Vulnerability of partially flooded PWR reactor cavity to a steam explosion. Nuclear Engineering and Design, Volume 236 (August 2006), pp. 1617-1627. “Jozef Stefan” Institute. 2005. (E)
16	16.2	WASH1400		N Rasmussen. Reactor Safety Study. WASH-1400 (NRC). 1975. (E)
16	16.2	MSCN19		R E Henry. Overview: Uncertainties Remaining in Severe Accident Phenomenology. Fauske & Associates Inc. MSC\N19. 1995. (E)
16	16.2	MSCN14		R E Henry and H K Fauske. A different approach to Fragmentation in Steam Explosions. Fauske & Associates Inc. MSC\N14. 1996. (E)
16	16.2	NURETH10		T Morii et al. Benchmark Analysis of Premixing Phase of FCI Experiment by the VESUVIUS Code. The 10th International Topical Meeting on Nuclear Reactor Thermal Hydraulics. NURETH-10. 2003. (E)
16	16.2	SECY93119		US-NRC. TMI-2 Vessel Investigation Project. SECY-93-119. 1993. (E)
16	16.2	ISBN8778931584		B R Sehgal et al. Pre-Project on Development and Validation of Melt Behaviour in Severe Accidents. NKS-99. ISBN 87-7893-158-4. 2004. (E)
16	16.2	REF0056		T G Theofanous, W W Yuen and S Angilini. The verification basis of the PM-ALPHA code. Nuclear Engineering and Design Volume 189, 59-102. 1999. (E)
16	16.2	REF0058		B R Sehgal. Accomplishments and challenges of the severe accident research. Nuclear Engineering and Design, Volume 210, 79-94. Elsevier. 2001. (E)
16	16.2	TECDOC1127		IAEA. A simplified approach to estimating reference source terms for LWR designs. TECDOC-1127. 1999. (E)
16	16.2	NEACSNIR20046		Current Severe Accident Research Facilities and Projects. NEA/CSNI/R(2004)6. OECD. 2004. (E)
16	16.2	REF0059		J M Bonnet et al. KROTOS FCI Experimental Programme at CEA Cadarache: New Features and Status. Popes’ Palace Conference Centre, Avignon, France. October 2-6, 2005
16	16.2	ICONE7359		N I Kolev. Verification of IVA5 Computer Code for Melt-Water Interaction Analysis. 7th International Conference on Nuclear Engineering, ICONE-7359, Tokyo, Japan. 1999. (E)
16	16.2	REF0057		T G Theofanous, W W Yuen, K Freeman and X Chen. The verification basis of the ESPROSE.m code. Nuclear Engineering and Design Volume 189, 103-138. Elsevier. 1999. (E)

16	16.2	NEACSNIR20047		OECD. Identification of Relevant Conditions and Experiments for Fuel-Coolant Interactions in Nuclear Power Plants. NEA/CSNI/R(2004)7. 2004. (E)
16	16.2	MACETRD13		M T Farmer, et al. MACE Test M3b, Data Report. MACE-TR-D13, Volume 1/2. Argonne Nat. Lab. November 1997. (E)
16	16.2	MACETRD16		M T Farmer, et al. MACE Test M4, Data Report. MACE-TR-D16. Argonne Nat. Lab. August 1999. (E)
16	16.2	ICAPP7544		M T Farmer, S Basu. A Summary of Findings from the Melt Coolability and Concrete Interactions (MCCI) Program. Proceedings of ICAPP 2007, Nice, France, paper 7544. May 2007. (E)
16	16.2	LANL13709M	4C	J.F. Briefmeister. MCNP – A General Monte Carlo N-Particle Transport Code. V. 4C, LANL-13709-M. (E)
16	16.2	REF0060		G. Ducros. "An overview of the VERCORS experimental programme". International VERCORS Seminar, Greoux les bains, France. October 2007. (E)
16	16.2	REF0061		Staff Reports to the President's Commission on the Accident at Three Mile Island. NRC. (E)
16	16.2	NFEPDDC59	B	A Caillaux. EPR RF002. Severe Accident. 5.5.2 In-vessel analysis. Hydrogen Source Term. NFEPD DC 59 Revision B. Framatome ANP. January 2004. (E)
16	16.2	NFEPDDC53	B	P Gandrille. EPR RF002. Severe Accident. In-vessel MAAP 4 study for containment by-pass radiological releases. NFEPD DC 53 Revision B. Framatome ANP. January 2004. (E)
16	16.2	NFEPDDC91	C	P Gandrille. EPR RF002. Severe Accidents. Dedicated Reactor Coolant System Depressurisation. Functional requirements. NFEPD DC 91 Revision C. Framatome ANP. August 2004. (E)
16	16.2	EPSLDC851		A Caillaux, G Azarian. EPR RF002. Severe Accident. 5.5.2 In-vessel analysis. Melt release conditions from the vessel. EPSL DC 851. Framatome ANP. October 2002. (E)
16	16.2	NGES22002en0006	D	M Nie, K-G Petzold. Main data for severe accident analyses. AREVA Report FANP NGES2/2002/en/0006 Revision D. Erlangen. January 2004. (E)
16	16.2	NFEPDDC56	B	P Gandrille. EPR RF002. Severe Accident. In-vessel analysis. MAAP 4 study in case of LB LOCA. Fission products into the containment. NFEPD DC 56 Revision B. Framatome ANP. January 2004. (E)
16	16.2	ETSSDC782	A	J Heliot, G Azarian. EPR BD. Severe Accidents. Assessment of steam generator tube rupture risk induced by a core melt scenario. ETSS DC 782 Revision A. Framatome ANP. February 1999. (E)
16	16.2	NEPDFDC22		Dolleans. FA3. Severe accident. Thermo-hydraulic loads on vessel support and reactor pit in case of lower head failure at 20 bar. FRA1 Contract A. NEPDF DC 22. AREVA Report. February 2008. (E)
16	16.2	FZKA6677		W Hering, Ch Homann, J-S Lamy. ISP 45-OECD Comparison Report on the Blind Phase of the OECD International Standard Problem no 45 exercise (QUENCH06). Report FZKA-6677. March 2002. (E)
16	16.2	NGPS42006en1014	D	M Bauer, M A Movahed, J Eyink. Prevention and mitigation of hydrogen risk impact of the two room concept and metallic liner, interface with civil work and layout. NGPS4/2006/en/1014 Revision D. Framatome ANP. June 2007 (E)
16	16.2	PESBG2011en1001	A	U. Schwarz. PAR Qualification Report. General Description. PESB-G/2011/en/1001 Revision A. AREVA NP. January 2011. (E)

16	16.2	PEPAG2011en1001	B	M. Hupp. Description of validation of AREVA recombiner modeling in computational programs based on extensive, international, qualificational testing. PEPA-G/2011/en/1001 Revision B. AREVA NP. January 2011. (E) *Supersedes Revison A January 2011.*
16	16.2	PEPAG2011en1021	A	M Hupp, H Dimmelmeier. Sensitivity analysis of a postulated reduced recombiner performance during severe accidents. PEPA-G/2011/en/1021 Revision A. AREVA NP. September 2011. (E)
16	16.2	NGPS42003en0020	D	J Eyink. Pressure and Temperature Loads for EPR with Steel Liner resulting from Hydrogen Combustion. NGPS4/2003/en/0020 Revision D. Framatome ANP. February 2004 (E)
16	16.2	KWUNAT1999E042	B	J Eyink, M Movahed, K-G Petzold. Justification of the H2 mitigation concept: Direct hydrogen release into the containment. Siemens Report KWU NA-T/1999/E/042 Revision B. September 2009. (E)
16	16.2	NGES22002en0018	B	J Eyink, M Movahed. Effect of Spray Activation on the Reactivity of the Hydrogen-Air-Steam Mixture. NGES2/2002/en/0018 Revision B. (TR 04/143). Framatome ANP (E)
16	16.2	KWUNAT2000E018	A	J Eyink, M Movahed, K-G Petzold. Thermal loads on the EPR containment wall due to hydrogen removal. KWU NA-T/2000/E018 Revision A. Siemens. (E)
16	16.2	KWUNAT2000E010	A	J Eyink, M Movahed. Thermal loads due to accidental ignition (SBLOCA). KWU NA-T/2000/E010 Revision A. Siemens. (E)
16	16.2	SNPNAT2000E035	A	J Eyink, M Movahed. Hydrogen risk from ex-vessel release. SNP NA T/2000/E035 Revision A. Siemens. (E)
16	16.2	KWUNAT2000E017	A	J Eyink, K-G Petzold. Thermal loads due to accidental ignition (SBLOCA/D). KWU NA-T/2000/E017 Revision A. Siemens. (E)
16	16.2	NGES22002en0007	E	M Fischer, J Eyink. R&D for severe accidents. NGES2/2002/en/0007 Revision E (TR03/131). Framatome ANP. (E)
16	16.2	NGPS42003en0192		J Eyink. The EPR Hydrogen Mitigation Concept. NGPS4/2003/en/0192 (Table 4.7.1.1, page 66). Framatome ANP (E)
16	16.2	NGPS42003en0132		J Eyink. Hydrogen Control: Status of the Justification of Tools and Methods with regard to GPR and IRSN Concerns. NGPS4/2003/en/0132. Framatome ANP. (E)
16	16.2	NEPAG2010en1022	C	M Fischer, M Klicheva. Functional principle of the core melt stabilization system of the EPR, including a description of the relevant phenomena, the related experimental background, and the impact of remaining uncertainties. NEPA-G/2010/en/1022 Revision C. AREVA NP. June 2010. (E)
16	16.2	NGPS42003en0003		M Nie, M Fischer. Analysis of the melt retention in the reactor pit for various release sequences. NGPS4/2003/en/0003. AREVA NP. July 2003. (E)
16	16.2	SKI0053		B R Sehgal et al. Final Report for the “Melt-Vessel Interactions (MVI)” Project'. SKI Report 00:53. December 2000. (E)
16	16.2	REF0062		H. Alsmeyer, et al. “The COMET concept for Cooling of Ex-Vessel Corium Melts”. 6th International Conference on Nuclear Engineering ICONE-6, San Diego CA, May 10-15 1998. (E)

16	16.2	REF0063		L Meyer, M Gargallo. Low pressure Corium Dispersion Experiments with Simulant Fluids in a Scaled Annular Cavity. Nuclear Technology, 141, pp. 257-274, 2003. (E)
16	16.2	NUREGCR5582 / SAND982047		T Y Chu, M Pilch, J Bentz et al. Lower Head Failure Experiments and Analyses. NUREG/CR-5582, SAND98-2047. 1999. (E)
16	16.2	REF0064		B R Sehgal, R R Nourgaliev and T N Dinh. Characterization of Heat Transfer Processes in a Melt Pool, Convection and Vessel- Creep. Nuclear Engineering and Design volume 211, 2002, pp. 172-187. February 2002. (E)
16	16.2	REF0065		A Theerthan, A Karbojian and B R Sehgal. FOREVER Experiments on Thermal and Mechanical Behaviour of Reactor Pressure Vessel during a Severe Accident. The EC-FOREVER-1 Test. Proceedings of the 16th International Conference on Structural Mechanics in Reactor Technology (SMIRT-16), Washington DC, August 2002. (E)
16	16.2	MACETRD6		M T Farmer, et al. MACE Test M1B, Data Report. MACE-TR-D6. Argonne Nat. Lab. September 1992. (E)
16	16.2	NEPSG2008en1157	B	M Fischer. System CMSS: Justification of the type of protective layer used in the reactor pit and melt discharge channel. NEPS-G/2008/en/1157 Revision B. AREVA NP. November 2010 (E)
16	16.2	ICAPP6330		M Nie, M Fischer. “Use of Molten Core Concrete Interactions in the Melt Stabilization Strategy of the EPR”, Proceedings of ICAPP’06, paper 6330. Reno NV USA. June 4-8, 2006. (E)
16	16.2	MPTR10		V G Asmolov et al., “Partitioning of fission products between the corium melt metallic and oxidic phases resulting from the STFM-FP tests”. OECD MASCA Project, Report MP-TR-10, Kurtschatov Institute Moscow, June 2003. (E)
16	16.2	REF0066		M Nie. Temporary Melt Retention in the Reactor Pit of the European Pressurised Water Reactor (EPR). Stuttgart University. February 2005. (E)
16	16.2	NGPS42005en1026	A	M Fischer. Description and validation of the improved gate design. NGPS4/2005/en/1026 Revision A (TR 05/219). AREVA NP. July 2005. (E)
16	16.2	REF0067		B R Sehgal, T N Dinh, R R Nourgaliev, M J Konovalikhin, V A Bui. “Assessment of Core Melt Spreading in EPR Melt Retention Device”. Sehgal Associates Inc., October 15, 1998. (E)
16	16.2	SAMECOSTARP071701		M Eddy, G Bandini. Preliminary analysis on corium spreading in reactor core-catcher with THEMA code. EU Community Research. SAM-ECOSTAR-P07-17/01. February 2002. (E)
16	16.2	REF0068		J-M Seiler, et al. European Group for Analysis of Corium Recovery Concepts – EUROCORE. Synthesis report, presented at FISA 2001, Luxembourg, Nov 12-14, 2001. (E)
16	16.2	EXVCOMAS97D10		W Koller, W Steinwarz. 2D-spreading test COMAS EU-4. EU Community Research. EXV-COMAS(97)-D10. December 1997. (E)
16	16.2	EXVCOMAS98D25		R Wittmaack. Post calculation of the COMAS EU-4 test with the CORFLOW computer code. EU Community Research. EXV-COMAS(98)-D25. March 1998. (E)

16	16.2	KWUNAT1998E077	A	R Wittmaack. EPR corium spreading analysis with the CORFLOW computer code. KWU NA-T/1998/E077 Revision A. Erlangen. AREVA NP. August 1998. (E)
16	16.2	EXVCOMAS99D27		W Steinwarz, et al. “Investigations on the Phenomenology of Ex-vessel Core Melt Behaviour”. Final report, EU 4th FWP. EXV-COMAS(99)-D27. (E)
16	16.2	NEPSG2008en1116		M Fischer. System CMSS: CMSS System Description. NEPS-G/2008/en/1116. AREVA NP. (E)
16	16.2	ACEXTRC14		D H Thompson, et al. “Compilation, Analysis, and Interpretation of ACE Phase C and MACE Experimental Data: Volume I – MCCI Thermohydraulic results”, ACEX TR-C-14, Argonne Nat. Lab. November 1997. (E)
16	16.2	FZKA7213		H Alsmeyer, et al. The COMET-L1 experiment on long-term Melt Concrete Interaction and Cooling by top flooding. Forschungszentrum Karlsruhe, Report FZKA 7213. June 2006. (E)
16	16.2	REF0069		M Fischer, O Herbst, H Schmidt. Demonstration of the heat removing capabilities of the EPR core catcher. 3rd International symposium on two-phase flow modelling and experimentation, Pisa, Italy, September 22-24, 2004. (E)
16	16.2	NGPS42004en0185	A	M Nie, M Fischer. Parametric examination of the sacrificial concrete layer thickness in the spreading compartment. NGPS4/2004/en/0185 Revision A (TR 04/166). AREVA NP. (E)
16	16.2	ICAPP4134		H Alsmeyer et al. “ECOKATS-2: A large scale experiment on melt spreading and subsequent cooling by top flooding”. Proc. ICAPP 2004, paper 4134. Pittsburgh, PA USA, June 13-17, 2004. (E)
16	16.2	NGPS42005en1061	B	W Schmidt. Mass- and Energy Release into the Containment due to Ex-Vessel Melt Stabilization. NGPS4/2005/en/1061 Revision B. AREVA NP. June 2007. (E)
16	16.2	ICAPP055001		M Fischer. Long-term Melt Stabilization as Part of the Severe Accident Mitigation Strategy of the European Pressurised Water Reactor. ICAPP05-5001, Seoul, Korea, 15-19 May 2005. (E)
16	16.2	ICONE1489088		M Fischer. The Core Melt Stabilization Concept of the EPR and its Experimental Validation. ICONE 14-89088, Miami FL, USA, 19-23 July 2006. (E)
16	16.2	NGPS42003en0059	B	M Fischer, M Nie. Updating of mass and energy release into the containment based on the latest R&D results. NGPS4/2003/en/0059 Revision B. AREVA NP. January 2004. (E)
16	16.2	NEPSG2007en0035		K-G Petzold. EPR Basic Containment Model for the "Lumped Parameter" code COCOSYS including the Combustible Gas Control System. NEPS-G/2007/en/0035. AREVA NP. December 2007. (E)
16	16.2	NGPS42005en1067	E	M Nie, et al. Loads on Containment Civil Structures resulting from Severe Accidents. NGPS4/2005/en/1067 Revision E. AREVA NP. June 2008. (E)
16	16.2	Euratom395487		European Council. Council Regulation (Euratom) 3954/87 Specifying Maximum Values for Radioactivity in Foodstuffs and Animal Feeds in the Event of a Nuclear Accident or Other Radiological Emergency. Official Journal of the European Community No. L371/11, 1987, amended by Council Regulation 2218/89. Official Journal of the European Community No. L211/1, 1989. (E)
16	16.2	NRPB14		National Radiological Protection Board. Emergency reference levels of dose for early countermeasures to protect the public. Doc NRPB 1(4). 1990. (E)

16	16.2	REF0070		Food and Environment Protection Act 1985. HMSO. 1985
16	16.2	ORNLTM200539	6	C Gauld, O W Hermann, R M Westfall. ORIGEN-S: Scale System Module to calculate fuel depletion, actinide transmutation, fission product build-up and decay, and associated radiation source terms. NUREG/CR-200, ORNL/TM-2005/39 Version 6 Volume II, Section F7. Oak Ridge National Laboratory. 2002. (E)
16	16.2	ENTTH0000277	B1	J L Droulas. Physical assumptions for calculating the source term under severe accident conditions. ENTTH/0000277 Revision B1 EDF. December 2010. (E) ENTTH/0000277 Revision B1 is the English translation of ENTTH/0000277 Revision B.
16	16.2	REF0037		Störfallberechnungsgrundlagen für die Leitlinien des BMI zur Berurteilung der Auslegung von Kernkraftwerken mit DWR gemäß §28 Abs. 3 StrlSchV, Veröffentlichung der Strahlenschutzkommision, Band 1, [Incident Calculation Bases for the Guidelines Issued by the Federal Minister of the Interior (BMI) for the assessment of the design of PWR Nuclear Power Plants pursuant to Sec. 28, para. (3) of the Radiological Protection Ordinance (StrlSchV)] 1989 (E)
16	16.2	REF0071		K H Neeb. The Radiochemistry of Nuclear Power Plants with Light Water Reactors. Walter de Gruyter, Berlin, New York. 1997. (E)
16	16.2	NUREGCR5732		Iodine Chemical forms in LWR Severe Accidents. NUREG/CR-5732. 1992. (E)
16	16.2	NUREG1465		L Soffer, et al. Accident Source Terms for Light-Water Nuclear Power Plants. NUREG-1465. 1992. (E)
16	16.2	DPAMSEMIC2011353		F.Payot, F.Cousin, N.Meynet, Dissociation of iodide aerosols into gaseous iodine in hydrogen recombiners – impact on the iodine source term. DPAM-SEMIC-2011-353. IRSN. January 2012. (E)
16	16.2	002161100DE003	A	C Seitan, Sensitivity study of the potential impact of recombiners on volatile iodine release during a severe accident. 002161-100-DE003-A BPE. Bertin Technologies. September 2012. (E)
16	16.2	ENFPIN0100177	A1	C Cherbonnel. Hypothesis to be used for evaluating the radiological consequences of design basis accident – Analysis of the ability of the different ventilation systems to reduce the fission products releases. ENFPIN/0100177 Revision A1. EDF. September 2009. (E) ENFPIN/0100177 Revision A1 is the English translation of ENFPIN/0100177 Revision A.
16	16.2	NUREGCR6613 / SAND970594	Vol1	D. Chanin, M.L. Young: Code Manual for MACCS2: Volume 1, User's Guide, NUREG/CR-6613, SAND97-0594, Volume 1, Sandia National Laboratories, Albuquerque, NM, USA (1998) (E)
16	16.2	ICRP56		International Commission on Radiological Protection: ICRP Publication 56. Age -Dependent Doses to the Public from Intake of Radionuclides; Part 1; 1990 . (E)
16	16.2	ICRP60		ICRP Publication 60. 1990 Recommendations of the International Commission on Radiological Protection. Ann. ICRP 21 (1-3). 1991. (E)
16	16.2	ICRP67		ICRP Publication 67. Age-Dependent Doses to Members of the Public from Intake of Radionuclides; Part 2; Ingestion Dose Coefficients 1990. 1994. (E)
16	16.2	ICRP69		ICRP Publication 69. Age-Dependent Doses to Members of the Public from Intake of Radionuclides; Part 3; Ingestion Dose Coefficients.1995. (E)

16	16.2	ICRP71		ICRP Publication 71. Age-Dependent Doses to Members of the Public from Intake of Radionuclides; Part 4; Ingestion Dose Coefficients. 1996. (E)
16	16.2	ICRP72		ICRP Publication 72. Age-Dependent Doses to Members of the Public from Intake of Radionuclides; Part 5; Compilation of Ingestion and Inhalation Dose Coefficients.1996. (E)
16	16.2	DSN440		Doury. Le vademecum des transferts atmospheriques [Atmospheric Transfer Handbook]. Report DSN n° 440. February 1992
16	16.2	9629EURATOM	Vol39	Council Directive 96/29/Euratom which sets out the basic standards for the protection of the health of workers and the general public against the dangers arising from ionising radiation. Official Journal of the European Communities, L159; Volume 39; European Council. June 1996. (E)
16	16.2	ENTEAG090116	A	G Fleury. Methodology for evaluating the radiological consequences of an accidental atmospheric release. ENTEAG090116 Revision A. EDF. April 2009. (E) ENTEAG090116 Revision A is the English translation of ENTEAG030152 Revision B.
16	16.2	ENSN090080	A	M. Boschiero. EPR-fleet convergence of methodologies for assessment of the radiological consequences of accidents. ENSN090080 Revision A. EDF. (E) ENSN090080 Revision A is the English translation of ENSNEA040124 Revision A.
16	16.2	ENTEAG090113	A	P Jan. PSAR EPR - Sensitivity analysis about radiological consequences in fault situations ENTEAG090113 Revision A. EDF. April 2009. (E) ENTEAG090113 Revision A is the English translation of ENTEAG050091 Revision B.
SUB-CHAPTER 16.4 – REFERENCES				
16	16.4	NFPSDDC89	A	Science V2 Nuclear Code Package - Qualification Report. NFPSD DC 89 Revision A. AREVA. March 2004. (E)
16	16.4	TFJCDC1547	E	Summary Note on Qualification of the COPERNIC/TRANSURANUS Software Version 2.4. TFJC-DC 1547 Revision E. FRAMATOME-ANP. October 2003. (E)
16	16.4	DTPSMTHLMDSEM2001063		The CATHARE Code. DTP/SMTH/LMDS/EM/2001-063. CEA. 2001. (E)
16	16.4	CEASSTHLDASEM2005043		CATHARE 2 V2.5_1: Description of the FUEL sub-module CEA SSTH-LDAS-EM-2005-043. CEA. 2005. (E)
16	16.4	FFDC00267	D	M5 Properties Data. FF DC 00267 Revision D. FRAMATOME-ANP. January 2005. (E)
16	16.4	REF0072		Doroschuk, V.E. and Lantsman, F.P. Effect of Pressure and Mass Flow Rate on Burnout Heat Fluxes in a Water and Steam-Water Mixture Flow in Tubes. International Journal of Heat and Mass Transfer V7 pp. 187-190. Pergamon Press. 1964. (E)
16	16.4	UKEPRI002		UK EPR GDA Project - Reference Design Configuration. UKEPR-I-002. EDF/AREVA. (E)
16	16.4	PEPRFDC97	A	UK EPR - Design Improvements for Heterogeneous Boron Dilution Fault. PEPR-F DC 97 Revision A. AREVA. June 2012. (E)
16	16.4	PEPRGDC100032	A	EPR™ - Heterogeneous Boron Dilution Resulting from an Improper Reactor Coolant Pump Start-up. PEPR-G DC 100032 Revision A. AREVA. November 2011. (E)

16	16.4	PEPRFDC67	A	Heterogeneous Boron Dilution Fault Schedule. PEPR-F DC 67 Revision A. AREVA. July 2011. (E)
16	16.4	PEPRFDC24	A	UK EPR – Safety case for the inherent boron dilution following LOCA. PEPR-F DC 24 Revision A. AREVA. July 2011. (E)
16	16.4	PEPRFDC70	A	UK EPR – Countermeasures Against Heterogeneous Dilution Initiators. PEPR-F DC 70 Revision A. AREVA. July 2011. (E)
16	16.4	NFPSDDC119		STAR-CD – Qualification on dilution phenomena. NFPSD DC 119. AREVA. October 2004. (E)
16	16.4	NFPSDDC85	D	MANTA – Code Synthetic Qualification Assessment. NFPSD DC 85 Revision D. AREVA. September 2008. (E)
16	16.4	PEPDFDC13	B	CFD simulation of the Juliette Tests – STAR-CD Physical Validation for external and inherent dilution. PEPD-F DC 13 Revision B. AREVA. November 2011. (E)
16	16.4	PTSDC10	C	UK EPR GDA Project – Safety Case for Spent Fuel Pool Cooling and Pool Drainage Faults. PTS DC 10 Revision C. AREVA. November 2012. (E)
16	16.4	PEPSG2012en1033	A	Safety Assessment of Consequences Due to Gross Failure of Penetrations in the Fuel Pools. PEPS-G/2012/en/1033 Revision A. AREVA. September 2012. (E)
16	16.4	PEPRG120074	B	UK-EPR™ - Calculation of Discharge Outflow Rates for Different Scenarios. PEPR-G-12-0074 Revision B. AREVA. September 2012. (E)
16	16.4	ECESN120833	A	FS03 – Impact on the Probabilistic Safety Assessment of the Initiating Events Identified for the Cask Loading Process. ECESN120833 Revision A. EDF. October 2012. (E)
16	16.4	ECESN110231	A	FS03 – Failure Modes and Effect Analysis for the Spent Fuel Pool Isolation Gates, the Spent Fuel Transfer Facility and Burn Up Device. ECESN110231 Revision A. EDF. May 2012. (E)
16	16.4	D455034114134	0	Estimated reliability parameters of PTR [FPCS], PMC [FHS] and DMK (Spent Fuel Handling) systems equipment used for Fuel Building pool leak prevention and fuel handling. D4550.34-11/4134 Revision 0. EDF. April 2012. (E)
16	16.4	ECESN120308	A	FS-03 – Quantification of the new initiating events identified for the spent fuel cask transfer facility and spent fuel cask loading process. ECESN120308 Revision A. EDF. May 2012. (E)
16	16.4	NEPRFDC584	A	UK EPR – Consistency between PSA list and PCC list. NEPR-F DC 584 Revision A. AREVA. July 2010. (E)
16	16.4	ECESN121088	A	Faults in Essential Support System – ALARP Assessments, Proposed Design Changes and Justification of Resultant Design. ECESN121088 Revision A. EDF. November 2012. (E)
16	16.4	ECES121567	A	GDA - DEL / DVL - Conceptual Design Note. ECES121567 Revision A. EDF. November 2012. (E)
16	16.4	PEPSDF12328		Safety frame for common cause failure events on the cooling chain, and analysis of classification upgrade of TLOCC mitigation means. PEPSD-F.12.328. AREVA. July 2012. (E)
16	16.4	ECESN120355	A	Design Basis Analysis of single fault on essential support systems. ECESN120355 Revision A. EDF. June 2012. (E).
16	16.4	PEPRFDC103	B	Loss of Support Systems - Design Basis Analyeses. PEPR-F DC 103 Revision A. AREVA. October 2012. (E).

16	16.4	ECESN120253	A	Screening of the HVAC systems to establish the impact of their loss on normal operation systems and on safety systems. ECESN120253 Revision A. EDF. October 2012. (E)
SUB-CHAPTER 16.5 – REFERENCES				
16	16.5	NEPRFDC584	A	UK EPR – Consistency between PSA list and PCC list. NEPR-F DC 584 Revision A. AREVA. July 2010. (E)
16	16.5	NEPRFDC551	C	EPR UK Functional Requirements on Non-Computerised Safety I&C Functions. NEPR-F DC 551 Revision C. AREVA. July 2012 (E)
16	16.5	PEPRFDC83	C	Functional analysis for sensors' Common Cause Failure. PEPR-F DC 83 Revision C. AREVA. October 2012. (E)
16	16.5	PEPCF120678	1	EPR UK GDA – FS02A7 – Development of a diverse protection system for RCV [CVCS] homogeneous boron dilution events in shutdown states. PEPCF.12.0678 Revision 1. AREVA. July 2012. (E)
16	16.5	PESSFDC27	A	UK EPR - Main Steam Isolation Valves ALARP Assessment regarding functional diversity and Single Failure Criterion. PESS-F DC 27 Revision A. AREVA. (E)
16	16.5	PEPRFDC28	A	ALARP demonstration for the design of the pressurizer Safety Valves regarding the Passive Single Failure. PEPR-F DC 28 Revision A. AREVA. November 2010 (E)
16	16.5	PESSFDC28	A	UK EPR - Containment Isolation Valves Diversification ALARP Assessment. PESS-F DC 28 Revision A. AREVA. November 2010 (E)
16	16.5	ECESN120274	A	EPR UK – Diversity for frequent faults: ATWS LOOP cumulated with EDG start-up failure. ECESN120274 Revision A. EDF May 2012. (E)
16	16.5	NFPSRDC1055	F	In-containment pressure and temperature in PCC and RRC-A accidents. NFPSR DC 1055 Revision F. AREVA. June 2008. (E) This study is referenced for information only, to provide an example of the study carried out for the Flamanville detailed design.
SUB-CHAPTER 16.6 – REFERENCES				
16	16.6	REF0019		European Nuclear Safety Regulators Group (ENSREG) "Stress test" specifications. May 2011. (E)
16	16.6	REF0020		Western European Regulators Association (WENRA) "Stress test" specifications proposed by the WENRA task force. April 2011. (E)
16	16.6	REF0021		Japanese earthquake and tsunami: Implications for the UK nuclear industry: Final Report. HM Chief Inspector of Nuclear Installations. September 2011. (E)
16	16.6	PEPSFDC151	A	UK-EPR GDA Project - Summary of the Additional Safety Evaluation Against Beyond Design Basis Earthquakes. PEPS-F DC 151 Revision A. AREVA. October 2012. (E)
16	16.6	ETDPNN120048	E	UK EPR GDA - Design against flooding events. E.T.DPNN/120048 Revision E. EDF. October 2012. (E)

16	16.6	PEPSFDC133	C	UK EPR GDA Project – Robustness of Power Sources and Long Term Cooling. PEPS-F DC 133 Revision C. AREVA. November 2012. (E)
16	16.6	ECESN120395	B	Severe Accident Management. ECESN120395 Revision B. EDF. November 2012. (E)
16	16.6	ECUK110959	A	Summary of the additional safety evaluation of the UK EPR design following Fukushima events. ECUK110959 Revision A. EDF. December 2011. (E)
APPENDIX 16A – REFERENCES				
16	App16A	NEPAG2010en1021	A	M Klicheva. Chemical aspects of the models used for the EPR severe accident analysis. NEPA-G/2010/en/1021 Revision A. AREVA NP. April 2010. (E)
16	App16A	ASME543560	Vol1	M G Plys, C Y Paik, R E Henry, C-d Wu, K Y Suh, S J Lee, M A McCartney, and Z Wang. MAAP4 model and validation status. ASME/JSME Nuclear Engineering Conference - Volume 1, ASME, 543-560, 1993. (E)
16	App16A	REF0073		K Y Suh, R E Henry. Debris interactions in reactor vessel lower plena during a severe accident. 1) Predictive model, Nuclear Engineering and Design 166, 147-163. Elsevier. 1996. (E)
16	App16A	REF0074		K Y Suh, R E Henry. Debris interactions in reactor vessel lower plena during a severe accident. 2) Integral analysis, Nuclear Engineering and Design 166, 165-178. Elsevier. 1996. (E)
16	App16A	REF0075		K Y Suh, R E. Henry, Integral analysis of debris material and heat transport in reactor vessel lower plenum, Nuclear Engineering and Design 151, 203-221. Elsevier. 1994. (E)
16	App16A	PSA96	Vol1	M Epstein, R E Henry, C Y Paik, M A Kenton, E L Fuller, Model of gas mixing in a steam generator inlet plenum, PSA 96, Volume 1, 1732-1740, American Nuclear Society. 1996. (E)
16	App16A	NEPDFDC197	A	P Gandrille, E Ducousso. UK EPR - Severe Accident Description of the In-Vessel Physical and Chemical Models of the MAAP4 Code and its Validation. NEPD-F DC 197 Revision A. AREVA. May 2012. (E)
16	App16A	NEPDFDC202	A	P Gandrille, E Ducousso. UK EPR – Severe Accident Description of Corium Behaviour in RPV Bottom Head and Corium Discharge to the Reactor Pit. NEPD-F DC 202 Revision A. AREVA. August 2010. (E)
16	App16A	GRSP31P32P33		COCOSYS V1.2, User Manual, Programme Reference Manual and Implementation Manual. GRS-P-3/1, P-3/2 and P-3/3. Gesellschaft für Anlagen-und Reaktorsicherheit mbH (GRS) [Society for Plant and Reactor Safety]. July 2000. (E)
16	App16A	GRSA1426		H Jahn, E Hofer. Description of the MOD2/85 Versions of the RALOC/FIPLOC Family, Part 2 : Physical Modelling of Thermal Hydraulics and Integration Methods. GRS-A-1426. Gesellschaft für Anlagen- und Reaktorsicherheit mbH (GRS) [Society for Plant and Reactor Safety]. 1988. (E)
16	App16A	GRSA2422		B Schwinges, M Heitsch, W Klein-Heßling, S Arndt, B Hüttermann. Weiterentwicklung und Validierung des Rechenprogramms RALOC [Enhancement and validation of the calculation programme RALOC]. GRS-A-2422. Gesellschaft für Anlagen- und Reaktorsicherheit mbh (GRS) [Society for Plant and Reactor Safety]. 1996.

16	App16A	GRSA2417		G Weber, S Schwarz, F Ewig, K Fischer. Benutzerhandbuch für FIPLOC 3.0 [User manual for FIPLOC 3.0]. GRS-A-2417. Gesellschaft für Anlagen- und Reaktorsicherheit mbH (GRS) [Society for Plant and Reactor Safety]. 1996.
16	App16A	NUREGCR1381 / SAND800822		F Gelbard. MAEROS User Manual, NUREG/CR-1381, SAND80-0822. USNRC, Sandia National Laboratories. December 1982. (E)
16	App16A	PSI128		S Güntay, R Cripps. IMPAIR/3: A computer program to analyze the iodine behaviour in multicompartments of a LWR containment. PSI-Bericht [PSI Report] Nr. 128. PSI. 1992. (E)
16	App16A	KWUNT2198228		F Funke. Erweiterung und Verbesserung der IMPAIR Modellierung (IMPAIR 3.51 für COCOSYS) [Extension and enhancement of the IMPAIR modelling (IMPAIR 3.51 for COCOSYS)]. KWU NT21/98/228. Siemens AG, Bereich Energieerzeugung (KWU) [Section Power Generation (KWU)]. 1998.
16	App16A	Semar9512		J J Foit, M Reimann, B Adroguer, G Cnerino. WECHSL – Release 3.3: A computer program for the interaction of a core melt with concrete including the long term behaviour. Reference Document Note Technique Semar 95/12. IPSN, Department de recherches en securite. February 1995. (E)
16	App16A	REF0076		CHEMAPP – The thermo-chemistry library for your software. GTT--Technologies, Herzogenrath, Germany, 1996-1999. (E)
16	App16A	GRSA2553		S Arndt, G Weber. FIPHOST – A Module to Calculate the Fission Product Transport in a LWR Containment. GRS-A-2553. Gesellschaft für Anlagen- und Reaktorsicherheit mbH (GRS) [Society for Plant and Reactor Safety]. 1998. (E)
16	App16A	GRSA2750		U Hesse. FIPISO-98 ein Rechenmodell zum Nuklidverhalten in einem Raumzellensystem nach einem Reaktorstörfall [FIPISO-98 a calculation model on nuclide behaviour in a lumped cell system after a reactor accident]. GRS-A-2750. Gesellschaft für Anlagen- und Reaktorsicherheit mbH (GRS) [Society for Plant and Reactor Safety]. 1998.
16	App16A	GRSA2737		'Entwicklung und Verifikation eines Containment-Codesystems (COCOSYS) und eines deutsch-französischen Integralcodes (ASTEC), Verifikation, Validierung und Anwendung der COCOSYS-Ausgangsversion'. Abschlußbericht Teil 2, GRS-A-2737. Gesellschaft für Anlagen-und Reaktorsicherheit mbH (GRS). ['Development and Verification of a Containment Code System (COCOSYS) and a German-French Integral Code (ASTEC), Verification, Validation and Application of the COCOSYS Original Version'. Final report, Part 2, GRS-A-2737. Society for Plant and Reactor Safety (GRS)]. October 1999.
16	App16A	GRSA2686		Schwarz, Verifikation und Validierung von COCOSYS an den Experimenten VANAM M3[Verification and Validation of COCOSYS based on VANAM M3 Experiments]GRS-A-2686, 1999
16	App16A	GRSA2357		Langhans, 'Qualitätshandbuch für COCOSYS', [Quality Manual for COCOSYS]GRS-A-2357. GRS. 1996
16	App16A	REF0066		M Nie. Temporary Melt Retention in the Reactor Pit of the EPR, Dr.-thesis. Stuttgart University. February 2005. (E)
16	App16A	INVCIT99P039		P Y Chevalier. 'TDBIV - TDBCR - Thermodynamic Databases for Nuclear Chemistry – Documentation', EU 4th Framework Programme INV-CIT(99)-P039, September 1999. (E)
16	App16A	NGPS42004en0100		M Nie. Description, Validation and Application of the MCCI Code COSACO. NGPS4/2004/en/0100 (Document produced for OL3). Framatome ANP. May 2004. (E)

16	App16A	MACETRD13		M T Farmer et al. 'Mace Test M3b Data Report Volume 1', MACE-TR-D13, EPRI TR-108806, November 1997. (E)
16	App16A	MACETRD16		M T Farmer et al. Mace Test M4 Data Report. MACE-TR-D16. August 1999. (E)
16	App16A	ACEXTRC14		D H Thompson, M T Farmer, J K Fink, D R Armstrong, B W Spencer. Compilation, Analysis and Interpretation of ACE Phase C and MACE Experimental Data, Volume 1 – MCCI Thermalhydraulics Results. ACEX TR-C-14. Argonne Nat. Lab. November 1997. (E)
16	App16A	ICAPP2004		M T Farmer, S Lomperski, S Basu. Results of Reactor Material Experiments Investigating 2-D Core-Concrete Interaction and Debris Coolability. Proceedings ICAPP 2004, Pittsburgh, PA. June 13-17, 2004. (E)
16	App16A	REF0077		B Spindler et. al. Simulation of corium concrete interaction in a 2D geometry: recent benchmarking activities concerning experiment and reactor cases. The 3rd European Review Meeting on Severe Accident Research (ERMSAR-2008), Nesseber, Bulgaria. 23-25 September 2008. (E)
16	App16A	BF400928		W Häfner. Experimentelle Untersuchungen zur Wechselwirkung von Corium-Schmelzen mit Schutz- und Opferschichten (CORESA), Abschlußbericht Reaktorsicherheitsforschung. [Experimental Studies on the Interaction of Molten Corium with Protective and Sacrificial Layers (CORESA), Final report]. Projekt-Nr. 1501210, Berichts-Nr. BF-40.092-8. October 2001.
16	App16A	NGES42002de0187a		M Fischer, S Hellmann, M Nie. Experimentelle Untersuchungen zur Wechselwirkung von Corium-Schmelzen mt Schutz- und Opferschichten (CORESA), Abschlußbericht Reaktorsicherheitsforschung. [Experimental Studies on the Interaction of Molten Corium with Protective and Sacrificial Layers (CORESA), Final report]. Projekt-Nr. 1501211, Berichts-Nr. NGES4/2002/de/0187a. November 2002.
16	App16A	SNTAB200201		W Steinwarz, W Koller, N Dyllong. Experimentelle Untersuchungen zur Wechselwirkung von Corium-Schmelzen mit Schutz- und Opferschichten (CORESA), Abschlußbericht Reaktorsicherheitsforschung.[Experimental Studies on the Interaction of Molten Corium with Protective and Sacrificial Layers (CORESA), Final report]. Projekt-Nr. 1501212, Berichts-Nr. SNT-AB-2002-01. February 2002.
16	App16A	NUREGCR4994 / SAND901022		E R Copus et al. Core-Concrete Interactions using molten steel with Zirconium on a basaltic basemat: The SURC-4 Experiment. NUREG/CR-4994, SAND90-1022. Sandia National Laboratories. April 1989. (E)
16	App16A	NEACSNIR9210		H Alsmeyer et al. Beta Experiments on Zirconium Oxidation and Aerosol Release during Melt- Concrete Interaction. Proc. Second OECD CSNI Specialist Meeting on Molten Core Debris- Concrete Interactions, Karlsruhe 1992. KfK 5108, NEA/CSNI/R(92)10. 1992. (E)
16	App16A	NEPAG2010en1023	A	A Fargette. Physical and chemical models used to analyse the interaction of molten core material with concrete and to determine the release of burnable has and fission products. NEPA-G/2010/en/1023 Revision A. AREVA NP. July 2010. (E)

16	App16A	NAT1998E102a / NAT1998E103a / NAT1998E104b		M Fischer, Description, Validation and Application of the Computer Code WALTER, Part 1 – Description, KWU NA-T/1998/E102a, EPR SNE 03899 RDR, SIEMENS, 1998; Part 2 – Verification, KWU NA-T/1998/E103a, EPR SNE 04173 RDR, SIEMENS, 1998; Part 3 – Validation, KWU NA-T/1998/E104b, EPR SNE 04463 RDR, SIEMENS, 1998. (E)
16	App16A	ANS61		L Baker, R E Faw, F A Kulacki. Post-accident Heat Removal-Part I: Heat Transfer Within an internally Heated, Non-boiling Liquid Layer, Nuclear Science and Engineering 61, 222-230. ANS. 1976. (E)
16	App16A	KWUNAM96E041a		R Wittmaack. CORFLOW, a computer program for the numerical simulation of free surface flow, KWU NA – M/96/E041a, AREVA/SIEMENS. 1996. (E)
16	App16A	DNM03572		R Wittmaack. Description, Validation and Application to EPR Corium Spreading of the CORFLOW Computer Code, EPR DNM03572, AREVA/SIEMENS. 1998. (E)
16	App16A	REF0078	Vol21B	G Eriksson et al. CHEMSAGE – A Computer Program for the Calculation Of Complex Chemical Equilibria, Metallurgical Transactions B, Vol. 21 B, 1013-1023, Springer Boston. 1990. (E)
16	App16A	REF0079		P Y Chevalier et al. Global thermodynamic approach of the molten core concrete interaction (MCCI) and selected application in the nuclear field, J. Chim Phys. 98, 849--860, EDP Sciences. 1997. (E)
16	App16A	FZK5994		J R Travis et al. GASFLOW: A Computational Fluid Dynamic Code Gases, Aerosols,And Combustion, Volume 1: Theory and Computational Model, Volume 2: User's Manual, Volume 3: Assessment Manual, LA-13357-M, FZKA-5994, October 1998. (E)
16	App16A	TR107517	V3	Generic Tests of Passive Autocatalytic Recombiners (PARs) for Combustible Gas Control in Nuclear Power Plants, EPRI TR-107517-V3, Final Report. EPRI. June 1997. (E)
16	App16A	NEA/CSNI/R(2000)7		S B Dorofeev, Chapter 3: Criteria for FA and DDT limits. Flame Acceleration and Deflagration-to-Detonation Transition in Nuclear Safety, in OECD State-of-the-Art Report by a Group of Experts, NEA/CSNI/R(2000)7. NEA. 2000. (E)
16	App16A	REF0080		S B Dorofeev, M S Kuznetsov, V I Alekseev, A A Efimenko and W Breitung. Evaluation of limits for effective flame acceleration in hydrogen mixtures, Journal of Loss Prevention in the Process Industries, Vol. 14, 583, Elsevier. 2001. (E)
16	App16A	REF0081		S B Dorofeev, V P Sidorov, A E Dvoinishnikov, and W Breitung. Deflagration to detonation transition in large confined volume of lean hydrogen-air mixtures, Combustion and Flame, Vol. 104, 95, Elsevier. 1996. (E)
16	App16A	REF0082		S B Dorofeev, V P Sidorov, M S Kuznetsov, I D Matsukov and V I Alekseev. Effect of scale on the onset of detonations, in Proceedings of the 17th International Colloquium on Dynamics of Explosions and Reactive Systems, Heidelberg, Germany, 1999. (E)
16	App16A	REF0083		J R Travis, G A Necker and P Royl. GASFLOW: A computational fluid dynamics code for gases, aerosols, and combustion, Vol. 2: User's manual, p. 246. Institute for Nuclear and Energy Technologies, Forschungszentrum Karlsruhe, 2007, http://www.gasflow.net/index.php?page=17. (E)

16	App16A	REF0084		A A Amsden and F H Harlow. Numerical calculation of almost incompressible flow. Journal Computational Physics, Volume 3, 80. Elsevier. 1968. (E)
16	App16A	REF0085		C W Hirt, A A Amsden and J L Cook. An Arbitrary Lagrangian-Eulerian Computing Method for All Flow Speeds, Journal Computational Physics, Volume 14, 227, Elsevier. 1974. (E)
16	App16A	NEACSNIR9916		H Karwat et al. State-of-the-art report on containment thermo--hydraulics and hydrogen distribution prepared by an OECD/NEA group of experts, NEA/CSNI/R(99)16. NEA. 1999. (E)
16	App16A	CFD4NRS		P Royl, U J Lee, J R Travis and W Breitung. Benchmarking of the 3D CFD code GASFLOW II with containment thermal hydraulic tests from HDR and ThAI, in Proceedings of the CFD4NRS Conference 2006, München, p. 685. (E)
16	App16A	REF0086		J W Spore, P Royl, J R Travis, E D Hughes, C Müller, H Wilkening, W Baumann, G F Niederauer. GASFLOW: A computational fluid dynamics code for gases, aerosols, and combustion, Vol. 3: Assessment manual, Institute for Nuclear and Energy Technologies, Forschungszentrum Karlsruhe, p. 121, October 1998. http://www.gasflow.net/index.php?page=18. (E)
16	App16A	REF0087		P Royl. GASFLOW analysis of the phebus FPT0 containment thermal hydraulics,in Proceedings of the Jahrestagung Kerntechnik 1995, Nürnberg, INFORUM Bonn, p. 107, 1995. (E)
16	App16A	REF0088		U J Lee, P Royl, J R Travis and T Kanzleiter. Blind 3D-calculation of the containment thermal hydraulic test TH10 in the ThAI facility using GASFLOW II.,in Proceedings of the Jahrestagung Kerntechnik 2004, Düsseldorf, INFORUM Berlin, p. 153, 2004. (E)
16	App16A	IRSNDSUSERACLEMAC0611		J Malet, E Porcheron, J Vendel, L Blumenfeld and I Tkatschenko. Specification of the SARNET spray benchmark, IRSN/DSU/SERAC/LEMAC/06-11, 2006. (E)
16	App16A	REF0089		J Malet et al. Sprays in containment: Final results of the SARNET spray benchmark, in Proceedings of the 3rd European Review Meeting on Severe Accident Research (ERMSAR-2008) Nesseber, 2008. (E)
16	App16A	REF0090		E-A Reinecke, A Bentaib, S Kelm, W Jahn, N Meynet and C Caroli. Review of LP & CFD recombiner modelling and experiments: Applicability to reactor simulations and open issues, in Proceedings of the 3rd European Review Meeting on Severe Accident Research (ERMSAR-2008) Nesseber, 2008. (E)
16	App16A	NESBG2007en0237	D	M-A. Movahed, M. Hupp. Assessment of the GASFLOW spray model based on the post-calculation of the TOSQAN experiments 101 and 113 and MISTRA experiments MASP1 and MARC2B. NESB-G/2007/en/0237 Revision D. AREVA. September 2010. (E)
16	App16A	REF0091		W Breitung et al. Integral large scale experiments on hydrogen combustion for severe accident code validation in HYCOM, Nuclear Engineering and Design, vol. 235, p253 - 270. Elsevier. 2005. (E)
16	App16A	FZK222	Vol32	A Kotchourko et al. Numerische Simulation der turbulenten Verbrennung von vorgemischten Gasen in komplexen 3D-Geometrien, FZK-Nachrichten, Forschungszentrum Karlsruhe, Jahrgang 32, Heft 3, 222 [Numerical Simulation of Turbulent Combustion of Premixed Gases in Complex. 3D-Geometries. FZK-News, Volume 32, Issue 3, 222]. Forschungszentrum Karlsruhe. 2000.

16	App16A	FZK175	Vol28	W Breitung and A Kotchourko. Numerische Simulation von turbulenter Wasserstoff-Verbrennung bei schweren Kernreaktorunfällen, FZK-Nachrichten, Forschungzentrum Karlsruhe, Jahrgang 28, Heft 2-3, 175, 1996[Numerical Simulation of Turbulent Hydrogen Combustion in Severe Reactor Accidents. FZK-News, Volume 28, Issue 2-3, 175, 1996]
16	App16A	REF0092		G Z Damköhler. Angewandte Physikalische Chemie [Applied Physical Chemistry], Volume 46, 601, 1940
16	App16A	LA3854		F H Harlow and P I Nakayama, Transport of Turbulence Energy Decay Rate, Los Alamos National Laboratory Report LA-3854. Los Alamos National Laboratory. 1968. (E)
16	App16A	REF0093		V Yakhot, S A Orszag, S Thangam, T B Gatski and C G Speziale. Development of turbulence models for shear flows by a double expansion technique, Physics of Fluids A, Vol. 4, No. 7, 1510-1520, American Institute of Physics. 1992. (E)
16	App16A	REF0094		D Baraldi et al. Application and assessment of hydrogen combustion models, Proceedings of the 10th International Topical Meeting on Nuclear Reactor Thermal Hydraulics (NURETH-10), Seoul. 2003. (E)
16	App16A	REF0095		U Bielert et al. Integral Large Scale Experiments on Hydrogen Combustion for Severe Accident Code Validation, in Proceedings of the Jahrestagung Kerntechnik 2002, Stuttgart. 2002. (E)
16	App16A	FZKA7115		A Kotchourko. Simulation of Combustion Processes in Lean H2-Air Mixtures: Conservatism of COM3D Results. FZK Report FZKA 7115. Forschungszentrum Karlsruhe. 2005. (E)
16	App16A	ENTEAG090124	A	A Dumontet. Description of the physics model.ENTEAG090124 Revision A. EDF. March 2009. (E) ENTEAG090124 Revision A is the English translation of ENTTH9900188 Revision A.
16	App16A	REF0096	2	Standard Review Plan 6.2.2.1A Revision 2, July 1981
16	App16A	NUREG0588		Tagami and Uchida correlations: NUREG-0588. US NRC. November 1979. (E)
16	App16A	ENTEAG090128	A	M H Boschiero, F Herber, G Depond. PAREO 8 code – Sensitivity studies and validation of the physical model. ENTEAG090128 Revision A. EDF. May 2009. (E) ENTEAG090128 Revision A is the English translation of ESETC8207 Revision A.
16	App16A	ENTEAG090118		Ch Cherbonnel. Validation of the PAREO Code. ENTEAG090118. EDF. August 2009. (E) ENTEAG090118 is the English translation of ENTTH89122 Revision A.
APPENDIX 16B – REFERENCES				
16	App16B	BDR1999		Small Break Loss of Coolant Accident without Low Head Safety Injection System (RRC-A). BDR chapter 19.2.1.3.6. EPR Basic Design Report, Revision D. AREVA . February 1999. (E)
CHAPTER 17				

SUB-CHAPTER 17.1 – REFERENCES				
17	17.1	ISBN0102156836		Health and Safety at Work Act, 1974. ISBN 978-010215683-6. The Stationery Office Ltd, October 1974. (E)
17	17.1	ISBN0118863681		UK Health and Safety Executive (HSE). The Tolerability of Risk from Nuclear Power Stations. ISBN 0118863681. The Stationery Office Ltd. 1992. (E)
17	17.1	TAST005	4	UK Health and Safety Executive (HSE). Technical Assessment Guide, ND Guidance on the Demonstration of ALARP (As Low As is Reasonably Practicable). T/AST/005 Issue 4 Revision 1. January 2009. (E)
SUB-CHAPTER 17.2 – REFERENCES				
17	17.2	TAST005	4	UK Health and Safety Executive (HSE). Technical Assessment Guide, ND Guidance on the Demonstration of ALARP (As Low As is Reasonably Practicable). T/AST/005 Issue 4 Revision 1. January 2009. (E)
17	17.2	REF0008		"Technical Guidelines for the design and construction of the next generation of nuclear pressurized water reactors" adopted during plenary meetings of the GPR and German experts on the 19 and 26 October 2000. (E)
17	17.2	CSFRF0893		EPR Conceptual Safety Features Review File; report jointly prepared by EDF, German Utilities, NPI, Siemens, FRAMATOME; August 1993. (E)
17	17.2	IAEATECDOC1044		Generic Safety Issues for Nuclear Power Plants with Light Water Reactors and Measures taken for their Resolution. IAEA Technical Document IAEA-TECDOC-1044. IAEA, Vienna. September 1998. (E)
17	17.2	SAP2006	1	UK Health and Safety Executive (HSE). Safety Assessment Principles for Nuclear Facilities. 2006 Edition Revision 1. January 2008. (E)
17	17.2	UKEPR0005001	0	Comparison of EPR Design with HSE/NII SAPS. UK EPR Report UKEPR-0005-001 Issue 00. AREVA/EDF. June 2008. (E)
17	17.2	GRS100		Reactor Safety Issues Resolved by the 2D/3D program GRS report 101 Sept. 1993 2D/3D Program; work summary report GRS 100. December 1992. (E)
17	17.2	SETEXLETS02166		Rapport synthétique d'évaluation BETHSY [Summary report of BETHSY Assessment]. CEA, IRSN, EDF, FRAMATOME ANP SETEX/LETS/02-166. June 2002.
17	17.2	REF0097		The CLOTAIRE International Program – Local Thermohydraulics Measurements in PWR steam Generators Wuhan Conference (China) September 1990. (E)
17	17.2	REF0098		Axial Economiser steam generator, Transient simulation NURETH 6. (E)
17	17.2	REF0099		The MEGEVE tests and their analysis by the computer code ANETH Proceedings 6th International topics – Meeting on Nuclear Thermalhydraulic Grenoble, France October 1993. (E)
17	17.2	REF0100		Starting up of new steam generator of 1450 MWe plants. 3rd International Steam Generator and Heat Exchanger Conference. Toronto, Canada. June 1998. (E)
17	17.2	REF0101		R. Krieg et al. Slug impact loading on the vessel head during a postulated steam explosion in pressurised water reactors. Nuclear Technology 111, p 369, 1995. (E)

17	17.2	REF0102		R. Krieg et al. Model experiments BERDA describing impacts of molten core materials against a PWR vessel head. International Conference on Advanced Reactor Safety ARS. Orlando, Florida, June 1997. (E)
17	17.2	REF0103		W. Schütz et al. Steam explosion experiments – CEA/FZK Seminar on EPR-related severe accidents research, Karlsruhe Nov. 1996. (E)
17	17.2	REF0104		L. Meyer et al. Investigation of the premixing phase of the steam explosion with hot spheres - International Conference on Advanced Reactor Safety ARS – Orlando, Florida June 1997. (E)
17	17.2	REF0105		W. Schütz et al. PREMIX, an experimental approach to investigate the mixing behaviour of a hot melt being poured into water - International Conference on Advanced Reactor Safety ARS – Orlando, Florida, June 1997. (E)
17	17.2	REF0106		D. Magallon et al. Status of FARO/KROTOS melt coolant interaction tests. Water Reactor Safety Meeting, Bethesda, Maryland, USA. October 1995. (E)
17	17.2	REF0107		P. Piluso et al. Steam explosion of Alumina in Krotos Facility : Is There a Material Effect. ICAPP-05, Seoul, Korea, May 15-19 2005. (E)
17	17.2	ICAPP5087		R. Meignen (IRSN), D. Magallon (CEA), Comparative Review of FCI Computer Models Used in the OECD-SERENA Program. Proceedings of ICAPP 05, Seoul, Korea, Paper 5087, May 15-19, 2005. (E)
17	17.2	REF0108		D. Magallon et al. Status of International Programme SERENA on Fuel Coolant Interaction. ICAPP 05, Seoul, Korea, May 15-19 2005. (E)
17	17.2	NEW9610		A. Tattegrain. PHEBUS probes core meltdown behaviour : the first results. Nuclear Europe Worldscan 9610, 1995 . (E)
17	17.2	JRCI9626		A. Jones et al. PHEBUS FP Test Matrix. JRC Technical Note Nr I 96.26. February 1996. (E)
17	17.2	REF0109		Proceedings of the PHEBUS FP 4th technical seminar. Marseille, March 20-22, 2000. (E)
17	17.2	REF0110		Proceedings of the 5th technical seminar. Aix-en-Provence, June 24-26, 2003. (E)
17	17.2	REF0111	Vol168	H. Hirschmann et al. CORVIS. Investigation of Light Water Reactor Lower Head Failure Modes. Nuclear Engineering and Design Vol. 168, pp. 77-104. 1997. (E)
17	17.2	REF0112	Vol331	J. Devos et al. Pressure vessel and piping design, analysis and severe accidents. ASME Vol. 331, 1996. (E)
17	17.2	REF0113		J.M. Bonnet et al. Large scale experiments for core melt retention. Workshop on large molten pool heat transfer. CEN Grenoble, France March 1994. (E)
17	17.2	REF0114		M. Barrachin, F.Defoort. Thermophysical Properties of In-Vessel Corium: MASCA Programme Related Results. Proceedings of MASCA Seminar, Aix en Provence, France. 10-11 June 2004. (E)
17	17.2	NUREGCR5746 / SAND991634		T.K. Blanchet, M.M. Pilch, R. Y. Lee, L. Meyer, M. Pellt. Direct Containment Heating Experiments at Low Reactor Coolant System Pressure in the Surtsey test facility, Scandia National Laboratory report, NUREG/CR. 5746, SAND 99-1634, 1999. (E)

17	17.2	REF0115		L. Meyer et al. Corium dispersal and direct containment heating at low system pressure. Submitted at the 10th International Topical Meeting of Nuclear Reactor Thermal Hydraulics, NURETH 10, Seoul, Korea. October 5-9, 2003. (E)
17	17.2	REF0116		D. Wilhelm. Recalculation of corium dispersal experiments at low pressure. Submitted at the 10th International Topical Meeting of Nuclear Reactor Thermal Hydraulics, NURETH 10, Seoul, Korea. October 5-9, 2003. (E)
17	17.2	NT141		J.M. Seiler et al. Theoretical Analysis for Corium Pool with Miscibility Gap. Nuclear Technology, 141, 233, 2003. (E)
17	17.2	REF0117	Vol221	B. Adroguer et al. Core loss during a severe accident (COLOSS). Nuclear Engineering and Design, Vol. 221, 55-76, 2003. (E)
17	17.2	REF0118		S. Hagen et al. Comparison of the Quench experiments CORA 12, 13, 17. International Quench Workshop Karlsruhe, FZK. 4-6 Oct. 1995. (E)
17	17.2	REF0119	Vol221	B.R. Sehgal et al. Assessment of reactor vessel integrity (ARVI). Nuclear Engineering and Design Vol. 221, 25-53, 2003. (E)
17	17.2	REF0120		G. Azarian et al. Qualification of the MAAP4.04 code on the QUENCH-03 experiment. Proceedings of the 6th International QUENCH workshop, Karlsruhe, October 2000. (E)
17	17.2	FZKA6677		W Hering, Ch Homann, J-S Lamy. ISP 45-OECD Comparison Report on the Blind Phase of the OECD International Standard Problem no 45 exercise (QUENCH-06). Report FZKA-6677. March 2002. (E)
17	17.2	REF0121		Study of the mixing of immiscible liquids. Results of the BALISE experiments Submitted at the 10th International Topical Meeting of Nuclear Reactor Thermal Hydraulics, NURETH 10, Seoul, Korea. October 5-9, 2003. (E)
17	17.2	REF0122		B. Adroguer et al. COLOSS – CORE LOSS during a severe accident. Nuclear Engineering and Design Vol. 235, 173-198, 2005. (E)
17	17.2	NEACSNIR99		H. Karwat et al. SOAR on containment thermohydraulics and hydrogen distribution. NEA/CSNI/R(99) 16 June 1999. (E)
17	17.2	REF0123		P. Royl et al. Analysis of Mitigating Measures during Steam/Hydrogen Distributions in Nuclear Reactor Containments with the 3d field Code GASFLOW. OECD workshop on the implementation of hydrogen mitigation techniques – Winnipeg, Manitoba, Canada – May 1996. (E)
17	17.2	REF0124		L. Wolf et al. Detailed Assessment of the HDR mixing experiments E11. Proc. Intern. Conf. "New trends in Nuclear System Thermohydraulics" Pisa, Italy 1994. (E)
17	17.2	BMBFBFR4007501		T. Kanzleiter. HYJET/PACOS Versuche im Modelcontainment. Abschlussbericht. [HYJET/PACOS Tests in the Model Containment. Final Report.] Battelle Report BMBF BF-R40.075-01. December 1996.
17	17.2	BFR633633		G. Langer et al. Experimental investigation of the hydrogen distribution in the containment of a light water reactor following a coolant loss accident. Battelle Report BF-R-63.363.3 NRC translation 801. 1980. (E)
17	17.2	REF0125		H. Tekuza et al. Final Comparison Report on ISP-35: NUPEC Hydrogen Mixing and distribution Test M-7-1. OECD Report OECD/GD (95) 29 NEA/CSNI/R (94) 29 Paris, December 1994. (E)

17	17.2	REF0126		P. Royl et al. Analyses of containment experiments with GASFLOW II. Submitted at the 10th International Topical Meeting of Nuclear Reactor Thermal Hydraulics, NURETH 10, Seoul, Korea. October 5-9, 2003. (E)
17	17.2	REF0127		J. Vendel et al. "ISP 47" containment hydraulic computer code exercise based on experimental facilities. Submitted at the 10th International Topical Meeting of Nuclear Reactor Thermal Hydraulics, NURETH 10, Seoul, Korea. October 5-9, 2003. (E)
17	17.2	REF0128		E. Studer et al. On the Use of the MISTRA Coupled Effect Test Facility for the Validation of Containment Thermal-hydraulic codes. Submitted at the 10th International Topical Meeting of Nuclear Reactor Thermal Hydraulics, NURETH 10, Seoul, Korea. October 5-9, 2003. (E)
17	17.2	REF0129		T. Kanzleiter et al. First results of the ThAI Thermal Hydraulic containment Tests. ICONE 9, Nice, April 2001. (E)
17	17.2	REF0130		T. Kanzleiter et al. ThAI multicompartment experiments with atmospheric stratification. NURETH 11, Avignon, October 3-6, 2005. (E)
17	17.2	NEACSNIR20007		W. Breitung et al. SOAR on flame acceleration and deflagration to detonation transition in nuclear safety. NEA/CSNI/R(2000)7, August 2000. (E)
17	17.2	REF0132		A. Veser et al. Multicompartimental simulation of Hydrogen Distribution and Turbulent Combustion in Severe Accidents (HDC). European Commission: FISA 99 EU Research in Reactor Safety, 29. November-1- December 1999. (E)
17	17.2	BMWA1501084		T. Büscher et al. Analyse Der Bedeutung der Phänomene HPME/DCH und Dampfexplosion für den Quellterm im Sicherheitsbehälter bei schweren Störfällen. Abschlussbericht zum Forschungsvorhaben BMWA 1501084, LEE-10, [Analysis of the Significance of the Phenomena HPME/DCH and Steam Explosion for the Source Term in the Containment of Light Water Reactors in Case of Severe Accidents. Final Report, Reactor Safety Research Project No. 1501084]. August 2003.
17	17.2	REF0091		W. Breitung et al. Integral large scale experiments on hydrogen combustion for severe accident code validation - HYCOM. Nuclear Engineering and Design 235 253-274, 2005. (E)
17	17.2	REF0133		M. Heitsch et al. Improvement and application of hydrogen combustion models. Submitted at the 10th International Topical Meeting of Nuclear Reactor Thermal Hydraulics, NURETH 10, Seoul, Korea. October 5-9, 2003. (E)
17	17.2	FZKNR28		W. Breitung. Numerical Simulation of Turbulent Hydrogen Deflagration in Severe Reactor Accidents. FZK Nachrichten - NR 28, Heft 213, S 175,1996. (E)
17	17.2	REF0134		B. Durst et al. Investigation of turbulent deflagrations with representative Flow obstacles. OECD workshop on the implementation of hydrogen mitigation techniques. Winnipeg, Manitoba, Canada, May 1996. (E)
17	17.2	REF0135		N. Arey et al Influence of transport phenomena on the structures of lean premixed hydrogen air flames. Winter Meeting "Thermalhydraulics of Severe Accidents" ANS 1995. (E)
17	17.2	REF0136		O. Braillard et al. Test of passive catalytic recombiners (pars) for combustible gas control in nuclear power plants. International Topical Meeting on Advanced Reactor Safety ARS, Orlando, Florida, June 1997. (E)

17	17.2	REF0137		T. Kanzleiter. Experimental Investigation on Recombiner Efficiency in the Battelle Model Containment. International Workshop on Hydrogen Research for Reactor Safety Munich, September 1994. (E)
17	17.2	CONTHYMI99		M. Carcassi et al. HYMI: final summary report. CONT-HYMI (99) – P035.1999 and references therein. (E)
17	17.2	OECD1996		E. Studer et al. The H2PAR facility for hydrogen research. OECD workshop on the implementation of hydrogen mitigation techniques. Winnipeg, Manitoba, Canada, May 1996. (E)
17	17.2	REF0138		E. Studer et al. Assessment of hydrogen risk in PWR. Eurosafe, November 1999. (E)
17	17.2	REF0139		B. Eckardt et al. Containment hydrogen control and filtered venting design and implementation. Proceedings of the OECD Workshop on the implementation of severe accident management measures held at PSI, Villingen, Switzerland, September 2001. (E)
17	17.2	NGPS52004en0076	B	Description of the PAR Qualification Tests. Framatome Technical Report NGPS5/2004/en/0076 Rev. B; EdF CP0, CPY Plants. 2004. (E)
17	17.2	LOFT6T63806		M.L. Charbonneau et al. Experimental Analysis and Summary Report for LOFT, Project Fission Product Experiment LOFT FP2. OECD LOFT6T63806. June 1989. (E)
17	17.2	AEARS2380		A.M. Beard et al. FALCON Experiments: a comparison of FALCON tests Fal 17 and Fal 18. AEA RS 2380, March 1993. (E)
17	17.2	REF0140		M. Firnhaber et al. Draft Specification of the International Standard Problem ISP 44: KAEVER, Experiments on the Behaviour of Core-Melt Aerosols in a LWR Containment. GRS, January 2000. (E)
17	17.2	JRCI9608		J.A. Capitao. 5th STORM Meeting: Analysis and First Experimental Results. JRC Technical Note Nr. I 96.08, January 1996. (E)
17	17.2	REF0141		F. Funke et al. Multi-compartmental Iodine Tests at the ThAI Facility. Eurosafe, Berlin 2004. (E)
17	17.2	NGTR2005en0308	A	G. Langrock. Synthesis of PARIS Results (Programme on Air Radiolysis, Iodine and Surfaces). Work Report NGTR/2005/en/0308 Revision A. AREVA. December 2005. (E)
17	17.2	REF0142		P. Spitz et al. Mass Transfer Programme in SISYPHE facility. Proceedings of the 5th technical seminar. Aix-en-Provence, June 24-26, 2003. (E)
17	17.2	IRSNCEAEdF		B. Clement. Towards reducing the uncertainties on Source Term Evaluations: an IRSN/CEA/EdF R&D programme. Eurosafe, Paris, 2003. (E)
17	17.2	EPRINP50545R		Alsmeyer et al. BETA experimental results on melt/concrete interaction. CSNI Specialist's Meeting on Core-Debris-Concrete Interaction. EPRI-NP-5054-5R, pp2-2 and 2-17, February 1987. (E)
17	17.2	ACEXTRC14		D H Thompson et al. Compilation, analysis and interpretation of ACE phase C and MACE experimental data – MCCI / Thermohydraulic results. ACEX TR-C-14, Vol 1 ; Argonne Nat. Lab. November 1997. (E)
17	17.2	MACETRD6		M.T. Farmer. M1b Data Report. MACE-TR-D6, Argonne Nat. Lab., September 1992. (E)
17	17.2	MACETRD13		M.T. Farmer. MACE test M3b, Data Report MACE-TR-D13, Vol. 1/2. Argonne Nat. Lab., August 1999. (E)

17	17.2	MACETRD16		M.T. Farmer. MACE Test M4, Data Report - Argonne Nat. Lab., August 1999. (E)
17	17.2	FZKA64753		J.M. Bonnet. Thermal hydraulic phenomena in corium pools for ex-vessel situations : the BALI experiment. OECD Workshop on ex-vessel coolability, Karlsruhe, Germany, FZKA 64753 , 1999. (E)
17	17.2	NGPS42004en0100	B	TR 61, 'Description, validation and application of the MCCI code COSACO', Framatome Work Report NGPS4/2004/en/0100 Rev. B, 24.05.2004. (E)
17	17.2	SNTAB200201		W. Steinwarz et al. Experimental investigations on Corium interaction with refractory and sacrificial layers (CORESA). BMWI project 1501212. Final report SNT-AB-2002-01. Siempelkamp Nukleartechnik GmbH. (E)
17	17.2	BMWI1501211		M. Fischer et al. Experimental investigations on Corium interaction with refractory and sacrificial layers (CORESA). BMWI project 1501211, Final report. FRAMATOME ANP GmbH, October. 2002. (E)
17	17.2	FZKA6453		B. Eppinger, F. Fellmoser, G. Fieg, H. Massier, G. Stern. Experiments on concrete erosion by a Corium melt in the EPR cavity: KAPOOL 6-8. FZKA 6453. Forschungszentrum Karlsruhe. March 2001. (E)
17	17.2	REF0143		J. M. Vetteau et al. CORINE Experiments and Theoretical Modelling. FISA 95 Symposium EU Research on Severe Accidents. Luxembourg. November 1995. (E)
17	17.2	REF0144		G. Fieg et al. Simulation Experiments on the spreading behaviour of Molten Core Melts. 1996 National Heat Transfer Conference. Houston, August 1996. (E)
17	17.2	REF0145		W. Scholtyssek et al. Recent Results of Melt Spreading and Cooling Experiments at Forschungszentrum Karlsruhe - the KATS and COMET Projects. CSARP Meeting, Bethseda, May 1996. (E)
17	17.2	FZKA6589		B. Eppinger et al. Simulationsexperimente zum Ausbreitungsverhalten von Kernschmelze: KATS 8-17. FZKA 6589. Forschungszentrum Karlsruhe,[Simulation experiments on the spreading Behaviour of Core Melts: KATS 8-1]. March 2001.
17	17.2	REF0146		Sappok, Steinwartz. Large Scale Experiments on Ex-vessel Core Melt Behaviour. International Conference on Advanced Reactor Safety ARS, Orlando, Florida June 1997. (E)
17	17.2	REF0147		G. Cognet et al. Corium spreading and coolability. FISA 97 Symposium on EU Research on Severe Accidents, November 1997. (E)
17	17.2	REF0148		I.M. Shepberd et al. Modelling fission product vapour transport for the FALCON facility. Nuclear Technology 110, p 181-187, 1988. (E)
17	17.2	REF0149		H. Wider. Transient Cooling of Shallow Debris Beds in the First Two FARO LWR Experiments. Nuclear Engineering and Design 157, p 395-408, 1995. (E)
17	17.2	REF0150		B.R. Seghal et al. Experimental investigation of melt spreading in one-dimensional channel. Report EU-CSC-1D1-97. Royal Institute of Technology, Stockholm. August 1997. (E)
17	17.2	REF0151		J.M. Bonnet, J.M. Seiler. Coolability of corium spread onto concrete under water, the PERCOLA model. Proceedings of the 2nd OECD (NEA) CSNI Specialist Meeting on Molten Core Debris-Concrete Interactions, pp. 347-354.Karlsruhe, Germany. April 1992. (E)

17	17.2	ICAPP5644		M. T. Farmer, S.Lomperski (ANL), S. Basu (US-NRC). Status of the Melt Coolability and Concrete Interaction (MCCI) Program at Argonne National Laboratory. Paper 5644, Proceedings of ICAPP '05; Seoul, Korea, May 15-19, 2005. (E)
17	17.2	REF0152		S. Rouge, I. Dor, G. Greffraye. Reactor Vessel External cooling for Corium Retention - SULTAN Experimental Program and Modeling with CATHARE Code. OECD Workshop on In-Vessel debris retention and coolability, Garchingen, Germany, 3-6 March 1998. (E)
SUB-CHAPTER 17.3 – REFERENCES				
17	17.3	TAST005	4	UK Health and Safety Executive (HSE). Technical Assessment Guide, ND Guidance on the Demonstration of ALARP (As Low As is Reasonably Practicable). T/AST/005 Issue 4 Revision 1. January 2009. (E)
17	17.3	ECEF072084	A1	Flamanville 3 EPR - Summary report: specific design provisions related to EPR chemistry. ECEF072084 Revision A1.EDF. February 2008. (E)
17	17.3	ECEP050315	A1	D. Tersigny. Analysis of environmental performance in the EPR France project. ECEP050315 Revision A1. EDF. March 2012. (E) ECEP050315 Revision A1 is the English translation of ECEP050315 Revision A.
17	17.3	REF0006		Radioactive Substances (Basic Safety Standards) (England and Wales) Direction 2000. Defra, UK. (E)
17	17.3	SAP2006	1	UK Health and Safety Executive (HSE). Safety Assessment Principles for Nuclear Facilities. 2006 Edition Revision 1. January 2008. (E)
17	17.3	ISBN0118863681		UK Health and Safety Executive (HSE). The Tolerability of Risk from Nuclear Power Stations. ISBN 0118863681. The Stationery Office Ltd. 1992. (E)
SUB-CHAPTER 17.4 – REFERENCES				
17	17.4	TAST005	4	UK Health and Safety Executive (HSE). Technical Assessment Guide, ND Guidance on the Demonstration of ALARP (As Low As is Reasonably Practicable). T/AST/005 Issue 4 Revision 1. January 2009. (E)
17	17.4	SAP2006	1	UK Health and Safety Executive (HSE). Safety Assessment Principles for Nuclear Facilities. 2006 Edition Revision 1. January 2008. (E)
17	17.4	ISBN0118863681		UK Health and Safety Executive (HSE). The Tolerability of Risk from Nuclear Power Stations. ISBN 0118863681. The Stationery Office Ltd. 1992. (E)
SUB-CHAPTER 17.5 – REFERENCES				
17	17.5	TAST005	4	UK Health and Safety Executive (HSE). Technical Assessment Guide, ND Guidance on the Demonstration of ALARP (As Low As is Reasonably Practicable). T/AST/005 Issue 4 Revision 1. January 2009. (E)
17	17.5	REF0153		US NRC. Regulations (10 CFR), 51.55(a), Environmental Report – Standard Design Certification. (E)

17	17.5	ANP10290	0	AREVA NP Environmental Report: Standard Design Certification, ANP-10290, Revision 0, November 2007. (E)
17	17.5	ENSNDR100088	A	F. Romanet. UK EPR ALARP methodology to support the design modification process. ENSNDR100088 Revision A. EDF/SEPTEN. July 2010. (E)
17	17.5	TDNSREP0243B	1	Nuclear Electric . 'Safety Review Guidebook for Gas Cooled Reactors'. Nuclear Electric Report TD/NS/REP/0243/B Issue 1. September 1993. (E)
17	17.5	14810TR0001	1	AMEC. UKEPR PCSR ALARP, Cost Benefit Analysis. AMEC Report 14810/TR/0001 Issue 01. April 2008. (E)
17	17.5	NUREGBR0184		Regulatory Analysis Technical Handbook. NUREG/BR-0184. USNRC. January 1997. (E)
17	17.5	PEERFDC78	A	UK EPR – ALARP Justification of Manufacturing Inspection Technique Proposed for the Main Coolant Lines Welds of the UK EPR. PEER-F DC 78 Revision A. AREVA. March 2012. (E)
CHAPTER 18				
SUB-CHAPTER 18.1 – REFERENCES				
18	18.1	SAP2006	1	Safety Assessment Principles for Nuclear Facilities. UK Health and Safety Executive (HSE). 2006 Edition Revision 1. January 2008. (E)
18	18.1	TAST058	1	Human Factors Integration. UK Health and Safety Executive (HSE). T/AST/058 - Issue 1 - September 2010. (E)
18	18.1	ISBN0110856252		Management of Health and Safety at Work Regulations 1999. ISBN 0-11-085625-2. The Stationery Office Ltd. 1999. (E)
18	18.1	ISBN0110856147		The Ionising Radiations Regulations 1999. Statutory Instrument 1999 No. 3232. ISBN 0 11-085614-7. The Stationery Office Ltd. (E)
18	18.1	ISO9241210		Ergonomics of Human-System Interaction - Part 210: Human-Centred Design for interactive Systems. ISO 9241-210. 2010 (E)
18	18.1	ISO11064		Ergonomic Design of Control Centres. ISO 11064. 2008. (E)
18	18.1	ISO9241		Ergonomics of human system interaction. ISO 9241. 2002. (E)
18	18.1	ISO80416		Basic principles for graphical symbols for use on equipment. ISO 80416. 2005. (E)
18	18.1	ISO7000		Graphical symbols for use on equipment. ISO 7000. 2004. (E)
18	18.1	ISO14617		Graphical symbols for diagrams. ISO 14617. 2004. (E)
18	18.1	ISO13406		Ergonomic requirements for work on flat panel display screens. ISO 13406. 1999. (E)
18	18.1	ISO15534		Ergonomic design for the safety of machinery. ISO 15534. 2000. (E)
18	18.1	ISO14738		Safety of machinery-anthropometric requirements for the design of workstations at machinery. ISO 14738. 2002. (E)
18	18.1	ISO6385		Ergonomics principles in the design of work systems. ISO 6385. 1990 (E)
18	18.1	NUREG0700		Human-system interface design review guidelines. NUREG-0700. Revision 2. 2002. (E)

18	18.1	NUREG0711		Human Factors Engineering Program Review Model. NUREG-0711. Revision 2. February 2004. (E)
18	18.1	NUREG0800	2	Human Factors Engineering. NUREG-0800 Chapter 18 Revision 2. 2004 (E)
18	18.1	NUREGCR6633		Advanced Information Systems: Technical Basis and Human Factors Review Guidance. NUREG/CR-6633. 2000 (E)
18	18.1	NUREGCR6634		Computer-Based Procedure Systems: Technical Basis and Human Factors Review Guidance. NUREG/CR-6634. 2000 (E)
18	18.1	NUREGCR6635		Soft Controls: Technical Basis and Human Factors Review Guidance. NUREG/CR-6635. 2000 (E)
18	18.1	NUREG1764		Guidance for the Review of Changes to Human Actions. NRC NUREG-1764. 2004 (E)
18	18.1	IEC80416		Basic principles for graphical symbols for use on equipment. IEC 80416. 2002. (E)
18	18.1	IEC61839		Nuclear Power Plants - Design of control rooms - Functional analysis and assignement. IEC 61839. November 2000 (E)
18	18.1	IEC60073		Basic and safety principles for man-machine interfaces, marking and identification. IEC 60073. 2002. (E)
18	18.1	IEC60447		Man-machine interface – actuating principles. IEC 60447. 2004. (E)
18	18.1	IEC60960		Functional design criteria for SPDS. IEC 60960. 1998. (E)
18	18.1	IEC60964		Design for control rooms of nuclear power plants. IEC 60964. (E)
18	18.1	IEC61227		Control rooms – Operator commands. IEC 61227. 2008. (E)
18	18.1	IEC61771		MCR – Design verification and validation. IEC 1995. (E)
18	18.1	IEC61772		MCR – Application of visual display units. IEC 61772. 1995. (E)
18	18.1	IEC62241		MCR – Alarm functions and presentation. IEC 62241. 2004. (E)
18	18.1	EPRI01		EPRI. Human Factors Guidance for Control Room and Digital Human-System Interface. Design and Modification. 2005. (E)
18	18.1	IEEE10231988		Guide for Application of Human Factors Engineering to Systems, Equipment and Facilities of Nuclear Power Generating Systems. IEEE-1023-1988. (E)
18	18.1	NFEN8972		Safety of machinery – Ergonomics requirements for design for means of signaling and components. NF EN 897-2
18	18.1	NFD62042		NF D 62-042: Office furniture. Tables and desk. General characteristics. Tests and requirements.
18	18.1	NFD62041		NF D 62-041: Office furniture. Furniture storage. General characteristics. Testing and specifications.
18	18.1	NFEN5271/NFEN5273		NF EN 527-1, NF EN 527-3: Work tables and desks.
18	18.1	NFEN1021		NF EN 1021 parts 1 and 2: Assessment of the Ignitability of Upholstered Furniture.
18	18.1	NFEN13351 / NFEN13352 / NFEN13353		NF EN 1335-1, NF EN 1335-2 and NF EN 1335-3: Office work seating.
18	18.1	NFEN13761		NF EN 13761: Visitors seating.
18	18.1	NFD65760		NF D 65-760: Rectangular locked metal cabinets.

18	18.1	NFX35102		AFNOR NF X35-102: Ergonomic design for the workspace in office.
18	18.1	ECEF021855	B1	ENG 2-21 Procedure: Degree of automation for plant systems. ECEF021855 Revision B1. EDF. May 2006. (E)
18	18.1	ECEF040244	D1	ENG 2-31: Principles of design of synthesis information. ECEF040244. Revision D1. EDF. January 2008. (E)
18	18.1	ECEF040683	C1	ENG 2-33 procedure: Specifications regarding the alarms. ECEF040683 Revision C1. EDF. August 2008. (E)
18	18.1	ECEF040974	B1	ENG 2-34: Operating Display format Specification. ECEF040974 Revision B1. EDF. June 2006. (E)
18	18.1	ECEF061119	A1	ENG 3-37: Design Rules for Operating Instruction (MOP). ECEF061119 Revision A1. EDF. January 2007. (E)
18	18.1	ECEF050611	A1	ENG 3-03 : EPR plant system specification-content and updates-ECEF050611 Revision A1. EDF. December 2005. (E)
18	18.1	ECEF061275	A1	ENG 3-40: Content and structure of Emergency Operating procedures instructions. ECEF061275 Revision A1. EDF. July 2009. (E) ECEF061275 Revision A1 is the English translation of ECEF061275 Revision A.
18	18.1	ECEMC0000059	A1	Writing guide for EPR system specifications. ECEMC0000059 Revision A1. EDF. February 2010. (E)
18	18.1	IAEAGSR1		IAEA GS-R-1: Safety Requirements: Legal and Governmental Infrastructure for Nuclear, Radiation, Radioactive Waste and Transport Safety. (E)
18	18.1	REF0013		Western European Nuclear Regulators' Association: WENRA Reactor Safety Reference Levels. 2008. (E)
18	18.1	IAEANSG12		IAEA - NS-G-1.2. Safety Assessment and Verification for Nuclear Power Plants. IAEA. 2005 (E)
18	18.1	IAEASF1		Safety Fundamentals No. SF-1: Fundamental Safety Principles. (E)
18	18.1	IAEAGSR3		Safety Requirements No. GS-R-3: The Management System for Facilities and Activities. IAEA. 2006. (E)
18	18.1	IAEAGSG31		Safety Guide No. GS-G-3.1: Application of the Management System Activities for Facilities and Activities. (E)
18	18.1	IAEAGSG35		Safety Guide No. GS-G-3.5: The Management System for Nuclear Installations. (E)
18	18.1	75INSAG4		Safety Series No. 75-INSAG-4: Safety Culture. (E)
18	18.1	75INSAG15		Safety Series No. 75-INSAG-15: Key Practical Issues in Strengthening Safety Culture. (E)
18	18.1	SRS11		Safety Report Series No. 11: Developing Safety Culture in Nuclear Activities. (E)
18	18.1	SRS42		Safety Report Series No. 42: Safety Culture in the Maintenance of Nuclear Power Plants. (E)
18	18.1	TECDOC1321		Self-assessment of safety culture in nuclear installations. TECDOC-1321 (E)
18	18.1	TECDOC1329		Safety culture in nuclear installations. TECDOC-1329 (E)
18	18.1	UKEPRI002		UK EPR GDA Project - Reference Design Configuration. UKEPR-I-002. EDF/AREVA. (E)
18	18.1	UKEPR0014001	0	Human Factor operating principle and benefit of computerised operation. UKEPR 0014-001 Issue 00. AREVA/EDF. December 2009. (E)
18	18.1	ENFCRI090272	A	State-Oriented Approach designer knowledge transfer. ENFCRI090272 Revision A. EDF. November 2009. (E)
18	18.1	ECEF100427	A	An Overview of the Human Factors Approach used for the EPR Design and Compliance with International Standards. ECEF100427 Revision A. EDF. March 2010. (E)
18	18.1	ISO13407		ISO. Human-centred design processes for interactive systems. ISO 13407. 1999. (E)

18	18.1	ECEF0100515	F1	Summary Report on Functional Requirements Analyses per Plant System Functional requirements analysis. ECEF0100515 Revision F1. EDF. June 2010 (E)
18	18.1	ECEF061266	A1	Procedure ENG 3-34 : Content and structure of Emergency Operating rule. ECEF061266 Revision A1. EDF. April 2009. (E) ECEF061266 Revision A1 is the English translation of ECEF061266 Revision A.
18	18.1	ECEF061353	A1	Procedure ENG 3-38: Rules for drafting normal operating rules ECEF061353 Revision A1. EDF. May 2009. (E) ECEF061353 Revision A1 is the English translation of ECEF061353 Revision A.
18	18.1	HT5403016	A1	EPR computerised operation: Protocol for evaluation tests relative to Operation Department requirements. HT-54/03/016 Revision A1. EDF. March 2010. (E) HT5403016 Revision A1 is the English translation of HT5403016 Revision A.
18	18.1	ECEF060191	A1	EPR HMI – Evaluation of the principles of computerised operation - Assessment of 2005 supplementary test campaign. ECEF060191 Revision A1. EDF. March 2010. (E) ECEF060191 Revision A1 is the English translation of ECEF060191 Revision A.
18	18.1	HT5405021	A1	EPR computerised operation protocol for complementary tests. HT-54/05/021 Revision A1. EDF. March 2010. (E).
18	18.1	ECEF050717	A1	Structuring of the requirements and specifications concerning the EPR Human-Machine Interface (HMI). ECEF050717 Revision A1. EDF. February 2010. (E) ECEF050717 Revision A1 is the English translation of ECEF050717 Revision A.
18	18.1	ECECC010055	F1	EPR - Technical Specifications and Conditions (CSCT) for Standard Instrumentation & Control General presentation report - CCF 01 and associated documents CCF02 to CCF17, ECECC010055 Revision F1. EDF. October 2009. (E) ECECC010055 Revision F1 is the English translation of ECECC010055 Revision F.
18	18.1	D40029207084		General policy – Guiding principles relating to the organisation of the Flamanville 3 shift crew. D4002.92-07/084. EDF. February 2010. (E)
18	18.1	04088300DE005	A1	EPR room layout Analysis of activity at the Chooz (N4) nuclear power plant. 04088-300-DE-005-A1. EDF. March 2012. (E)
18	18.1	ECECC041294	C1	Technical specifications and conditions for the layout and outfitting design of the control room, adjoining rooms, and the full scale simulator room of the EPR France plant ECECC041294 Revision C1. EDF. February 2010. (E) This document refers to numerous NF EN ISO or NF X standards, and CEI 60964, 61771, 61839. ECECC041294 Revision C1 is the English translation of ECECC041294 Revision C.
18	18.1	04088300DE006	A1	EPR Control Room Layout –Results of the full-scale mock-up experiment for the EPR France Control Room. 04088-300-DE006-A1. EDF. March 2012. (E)
18	18.1	ECEF031026	B1	EPR Local Maintenance and Operation Activities to be Analysed from the Human Factors Standpoint. ECEF031026 Revision B1. EDF. February 2012. (E)
18	18.1	ECEP060987	A1	Summary and Results of the FA3 and 4 ETB Ergonomic Study. ECEP060987 Revision A1. EDF. October 2009. (E)

18	18.1	ECEP071048	A1	Procedure for taking into account Human Factors in fuel handling operations. ECEP071048 Revision A1. EDF.February 2010. (E) ECEP071048 Revision A1 is the English translation of ECEP071048 Revision A.
18	18.1	ECEIG0100625	B1	MIP EPR No 2.64 Installation Guide - Consideration of Human Factors. ECEIG0100625 Revision B1. EDF. July 2011. (E)
18	18.1	ECEIG99070	B1	Fuel Building Specification. ECEIG99070 Revision B1. EDF. March 2010. (E)
18	18.1	ECECC111829	B	Class 1 Control and Display Facilities in the Main Control Room and the Remote Shutdown Station. ECECC111829 Revision B. EDF. August 2012. (E)
18	18.1	3713254		Toward a Theory of Situation Awareness in Dynamic Systems. Human Factors. 37(1). 32-54. Endsley. 1995b.
18	18.1	ECEIG0100002	A1	Sheet C5 "Control Rooms and Annexes". ECEIG0100002 Revision A1. EDF. March 2010. (E)
18	18.1	ECEF051003	A1	Habitability of the main control room in the event of fire. ECEF051003 Revision A1. EDF. October 2008. (E)
18	18.1	ECEIG0000333	E1	Specifications for Nuclear Auxiliary Building. ECEIG0000333 Revision E1. EDF. February 2008 (E)
18	18.1	ECEIG0000810	C1	Specifications for Safeguard Auxiliary and Electrical Buildings. ECEIG0000810 Revision C1. EDF. January 2010. (E)
18	18.1	ECEIG0001089	C1	A Meziere, P. Bily. Reactor Building Specifications. ECEIG0001089 Revision C1. EDF. September 2009. (E) ECEIG0001089 Revision C1 is the English translation of ECEIG0001089 Revision C.
18	18.1	ECEIG0000756	C1	Diesel Building: Specifications of the Diesels Buildings. ECEIG0000756 Revision C1. EDF. February 2009. (E)
18	18.1	ECEIG021333	A	Note EPR Optimisation des chantiers Radioprotection hors BR [EPR Optimisation of activities with Radiation Protection Level other than the BR]. ECEIG021333 Revision A. EDF. November 2003. (F/E) This document is available in French and is included for information only.
18	18.1	EYRL2008fr0003	D1	EPR Optimisation of activities with Radiation Protection Level – Thermal insulation removal and reinstallation – phase 2. EYRL/2008/fr/0003 Revision D1. EDF. June 2010. (E)
18	18.1	EYRL2008fr0048	C1	EPR Optimisation of activities with Radiation Protection Level – Site Logistics – phase 2. EYRL/2008/fr/0048 Revision C1. EDF. February 2010. (E)
18	18.1	ECEMA071469	C1	EPR - Optimisation of activities that impact radiation protection - RCP, RCV and RIS/RRA valves - volume 2. ECEMA071469 Revision C1. EDF. May 2010. (E)
18	18.1	ECEMA070805	B1	EPR - Optimizing radiation protection-related activities - preparing and inspecting steam generators - Chapter 2. ECEMA070805 Revision B1. EDF. April 2010 (E)
18	18.1	ECEMA070986	B1	EPR – Optimizing radiation protection-related activities – “opening/closing of reactor vessel head” activity – Chapter 2. ECEMA070986 Revision B1. EDF. April 2010. (E)
18	18.1	EYTM2007fr0030	C1	EPR – Optimization of activities with radiation protection impact fuel shipment – chapter 2 – YR2621. EYTM/2007/fr/0030 Revision C1. EDF. June 2010. (E)
18	18.1	D40029206123	0.1	EPR Optimisation of Radioprotection activities. Waste treatment Phases 1 and 2. D4002.92-06/123 Revision 0.1. EDF. March 2009. (E)

18	18.1	ECEIG100655	A	Review of the FA3 PDMS model of the installation levels +13.40m and +15.20m of the nuclear Auxiliary Building. ECEIG100655 Revision A. EDF. June 2010. (E)
18	18.1	ECEIG091382	A	Review of the FA3 PDMS model of the installation level +3,70m of the Nuclear Auxiliary Building. ECEIG091382 Revision A. EDF. July 2009. (E)
18	18.1	ECEIG040462	B1	EPR Optimisation of activities with Radiation Protection Level – Thermal insulation removal and reinstallation. ECEIG040462 Revision B1. EDF. January 2006. (E)
18	18.1	ECEIG041062	B1	EPR Optimisation of activities with Radiation Protection Level – Site Logistics. ECEIG041062 Revision B1. EDF. January 2009. (E)
18	18.1	ECEMA050230	B1	EPR Optimisation of activities with Radiation Protection risk – RCP, RCV,RIS/RRA. ECEMA050230 Revision B1. EDF. January 2009. (E)
18	18.1	ECEMA041034	B1	EPR Optimisation of activities with Radiation Protection Level – Preparation and control Steam Generators – Phase 1. ECEMA041034 Revision B1. EDF. March 2009. (E)
18	18.1	ECEMA050275	C1	EPR Optimisation of activities with Radiation Protection Level – Opening /Closure reactor vessel – Phase 1. ECEMA050275 Revision C1. EDF. February 2009. (E)
18	18.1	ECEMA050056	A1	EPR Optimisation of activities with Radiation Protection Level – Fuel posting out activities – Phase 1. ECEMA050056 Revision A1. EDF. March 2009. (E)
18	18.1	D40029206123	0.1	EPR Optimisation of Radioprotection activities. Waste treatment Phases 1 and 2. D4002.92-06/123 Revision 0.1. EDF CNEN. March 2009. (E)
18	18.1	ENG306		EPR ENG 3.06 - Guide de rédaction des cahiers des charges (CdC) des bâtiments [Procedure ENG 3.06 – Building Specification Writing Guide]), This document is only available in French and is provided for information only.
18	18.1	IAEANSR1		NS-R-1 - Safety of Nuclear Power Plants: Design (Requirements). 2000. (E)
18	18.1	IAEANSG11		NS-G-1.1 - Software for Computer Based Systems Important for Safety in Nuclear Power Plants. 2000. (E).
18	18.1	IAEANSG13		NS-G-1.3 - Instrumentation and Control Systems Important to Safety in Nuclear Power Plants. 2002. (E)
18	18.1	D455010054528	0	Task model analysis for operating activities. D4550.10-05/4528 Revision 0. EDF. December 2005. (E)
18	18.1	ECECC120489	A	Protection System Operator Terminal Basis of Safety Case. ECECC120489 Revision A. EDF. May 2012. (E)
18	18.1	ECEF032026	A1	Feasibility study of Operation Division specifications for EPR computerised operation. ECEF032026 Revision A1. EDF. September 2010. (E)
18	18.1	16895707000RPT0017	G	PICS to SICS Transfer: A claims, arguments and evidence-based safety case. 16895-707-000-RPT-0017 G-BPE. AMEC. August 2012. (E)
18	18.1	ECEF021069	E1	Sizing of SICS. ECEF021069 Revision E1. EDF. December 2010. (E)
18	18.1	ENFCRI090069	A	SICS Operating Principles. ENFCRI090069 Revision A. EDF. March 2009. (E) ENFCRI090069 Revision A is the English translation of ENFCRI070019 Revision B.

18	18.1	16895707000RPT0006	E	EDF/AREVA GDA Task Analysis: Entry into Severe Accident Management Guidelines (OSSA). 16895-707-000-RPT-0006 Revision E. AMEC. August 2012. (E)
18	18.1	PTLFDC5	A	Non-Computerised Safety System- Basis of Safety Case. PTL-F DC 5. Revision A. August 2012. (E)
18	18.1	NEPRFDC551	C	EPR UK Functional Requirements on Non-Computerised Safety I&C Functions. NEPR-F DC 551 Revision C. AREVA. July 2012 (E)
18	18.1	NEPSFDC555	D	Safety Requirements for Non-Computerised Safety System (NCSS). NEPS-F DC 555 Revision D. June 2012. (E)
18	18.1	16895707000RPT0010	G	EDF/AREVA GDA Task Analysis: Operator responses to decreasing RCS level (OP_SIS_INJ_80MN_NCSS) on the Non Computerised Safety System. 16895-707-000-RPT-0010 Revision G-BPE. AMEC. October 2012. (E)
18	18.1	ECEF070498	A1	EPR Emergency Operation Process – Interface with qualification. ECEF070498 Revision A1. EDF. July 2009. (E) ECEF070498 Revision A1 is the English translation of ECEF070498 Revision A.
18	18.1	16895707000RPT0015	F	Holistic Arguments and Evidence to Support Claims relating to Misdiagnosis in Emergency Operations. 16895-707-000-RPT-0015 Revision F-BPE . AMEC. August 2012. (E)
18	18.1	ECEF110313	A	UK EPR Human Factors : Potential for mis-diagnosis and potential for violation. ECEF110313. Revision A. EDF. March 2011. (E)
18	18.1	PEPSPF11304		UK EPR – Identification and categorisation of PSA 2011 type C claims. PEPSPF/11.304. AREVA. July 2011. (E)
18	18.1	PEPSPF11486	1	Confirmation of Design Features Relating to Misalignment of Automated Valves. PEPSPF/11.486 Revision 1. AREVA. December 2011. (E)
18	18.1	TAST063	1	Technical Assessment Guide - Human Reliability Analysis. T/AST063 Issue 1. UK Health and Safety Executive (HSE). 2009. (E)
18	18.1	16474TR0003	D	EDF/AREVA GDA Task Analysis : Method statement and analysis of two example operators claims. 16474/TR/0003 Issue D-BPE. AMEC. May 2012. (E)
18	18.1	NEPSFDC191	A	Human Reliability Analysis Notebook of the UK EPR Probabilistic Safety Assessment. NEPS-F DC 191 Revision A. AREVA. January 2010. (E)
18	18.1	NEPSFDC527	A	E Sauvage. UK EPR Level 2 Supporting Analysis - Human Reliability Analysis. NEPS-F DC 527 Revision A FIN. AREVA. January 2010. (E)
18	18.1	NUREGCR4772		A D Swain. Accident Sequence Evaluation Program Human Reliability Analysis Procedure. NUREG/CR-4772. Sandia National Laboratories, Albuquerque, New Mexico. February 1987. (E)
18	18.1	NUREGCR1278		Swain, A. D., & Guttman, H. E. Handbook of human reliability analysis with emphasis on nuclear power plant applications. NUREG/CR-1278. Sandia National Laboratories. August 1983. (E)
18	18.1	NUREGCR6883		The SPAR-H Human Reliability Analysis Method. NUREG/CR-6883. Idaho National Laboratory. August 2005. (E)
18	18.1	NUREG1792		NUREG 1792. Good Practices for Implementing Human Reliability Analysis. 2005. (E)
18	18.1	17163707000RPT0001	6	Task Analysis Methodology: Analysis of Type A and B Pre-fault Human Errors. 17163-707-000-RPT-0001 Issue 06. AMEC. September 2011. (E)

18	18.1	PEPSPF11336		List of Significant Equipment and Failure Modes for Pre-accident HFE analysis. PEPSPF/11.336. AREVA. August 2011. (E)
18	18.1	17163190000RPT0001	2	A. Wilkinson. UK GDA Analysis of Pre-Initiator Human Errors. Risk Significant Equipment Types Grouped by Generic Equipment Type (Including Legacy – Non-Legacy Status). 17163-190-000-RPT-0001 Issue 02. AMEC. August 2011. (E)
18	18.1	17163707000RPT0002	F	Identification of Tasks Associated with Type A/B Human Failure Events Modelled in the PSA. 17163-707-000-RPT-0002 Issue F-BPE. AMEC. February 2012. (E)
18	18.1	17163707000RPT0004	H	Substantiation of Identified Type A Human Failure Events Modelled in the PSA. 17163-707-000-RPT-0004 Revision H-BPE. AMEC. August 2012. (E)
18	18.1	ECESN120755	A	EPR UK GDA Issue HF01- report D1.7: Substantiation of Identified Type B Human Failure Events. ECESN120755 Revision A. EDF. November 2012. (E)
18	18.1	PEPSF12104	1	Confirmation of design features related to claims M3/M23/M25/M28. PEPSF/12.104 Revision 1. AREVA. March 2012. (E)
18	18.1	HFGNSR5028	B	L Ainsworth et al. Guidelines for the Selection and Use of Task Analysis Data in the UK Nuclear Industry. HF/GNSR/5028 Revision B. February 2000. (E)
18	18.1	16474TR0006	G	EDF/AREVA GDA Task Analysis: Post Fault Example 3 [OP_FEED_TK]. 16474/TR/0006 Issue G-BPE. AMEC. May 2012. (E)
18	18.1	16895707000RPT0001	E	EDF/AREVA GDA Task Analysis: Feed and Bleed Recovery Strategies [OP_BLEED_120MN] and [OP_BLEED_30MN]. 16895-707-000-RPT-0001 Revision E-BPE. AMEC. October 2011. (E)
18	18.1	16895707000RPT0003	H	EDF/AREVA GDA Task Analysis of Post Fault Claim H2 [OP_LHSI_IND_120MN]. 16895-707-000-RPT-0003 Issue H-BPE. AMEC. October 2012. (E)
18	18.1	16895707000RPT0004	F	EDF/AREVA Task Analysis of Post Fault Claims M2 [OP_EFW/MSRT_2HLOCAL] and M7. [OP_SBODG30M]. 16895-707-000-RPT-0004 Revision F-BPE. AMEC. October 2012. (E)
18	18.1	16895707000RPT0002	I	EDF/AREVA GDA Task Analysis Steam Generator Tube Rupture Recovery Strategies. 16895-707-000-RPT-0002 Issue I. AMEC. October 2012. (E)
18	18.1	16895707000RPT0005	D	EDF/AREVA GDA Task Analysis of Post Fault Claims M6 [OP_FSCD_30MN_IH], M8 [OPE_52] and M19 [OP_COMBI_240MN_LDEP]. 16895-707-000-RPT-0005 Revision D-BPE. AMEC. September 2012. (E)
18	18.1	16895707000RPT0007	D	EDF/AREVA GDA Task Analysis: Primary Circuit Depressurisation in the EOP and the OSSA. 16895-707-000-RPT-0007 Revision D-BPE. AMEC. October 2012. (E)
18	18.1	16895707000RPT0008	D	EDF/AREVA GDA Task Analysis: Operator Responses to Loss of Instrumentation and Control [OP_EFWS]. 16895-707-000-RPT-0008 Revision D-BPE. AMEC. September 2012. (E)
18	18.1	16895707000RPT0011	D	EDF/AREVA GDA Task Analysis: NCSS Action for OP_EFWS_NCSS, OP_FB_120_MDEP_NCSS, OPE_52_LOCAL, OP_SBODG_LOCAL. 16895-707-000-RPT-0011 Revision D-BPE. AMEC. October 2012. (E)

18	18.1	16895707000RPT0024	D	EDF/AREVA GDA Task Analysis: NCSS Action for OP_BLEED_30MN_NCSS. 16895-707-000-RPT-0024 Revision D-BPE. AMEC. August 2012. (E)
18	18.1	16895707000RPT0025	C	EDF/AREVA GDA Task Analysis Summary Report. 16895-707-000-RPT-0025 Revision C-BPE. AMEC. November 2012. (E)
18	18.1	PEPSFDC135	B	Identification of Dropped loads and Fuel Handling Human Based Safety Claims- Refueling Machine. PEPS-F DC 135 Issue B. AREVA. July 2012. (E)
18	18.1	PEPSFDC134	B	Identification of Dropped loads and Fuel Handling Human Based Safety Claims- Polar Crane. PEPS-F DC 134 Issue B. AREVA. June 2012. (E)
18	18.1	PEPSFDC96	D	Dropped Loads and Fuel Handling: Methodology for the Identification of the Human Based Safety Claims. PEPS-F DC 96 Revision D. AREVA. January 2012. (E)
18	18.1	16895707000RPT0014	F	EDF/AREVA GDA Human Factors Issue: Heterogeneous Boron Dilution. 16895-707-000-RPT-0014 Revision F-BPE. AMEC. August 2012. (E)
18	18.1	PEPRF101665	3	EPR™ UK - GDA - Single Tube Steam Generator Tube Rupture Analysis for the UK. PEPR-F.10.1665 Revision 3. AREVA. September 2012. (E)
18	18.1	PEPRFDC38	D	Steam Generator Tube Rupture Mitigation Strategy. PEPR-F DC 38 Revision D. AREVA. October 2012. (E).
18	18.1	ECEIG111647	B	Internal Flooding – Bounding Cases: Mitigation Measures Report. ECEIG111647 Revision B. EDF. January 2012. (E)
18	18.1	16895707000RPT0013	E	Internal Flooding. 16895-707-000-RPT-0013 Revision E-BPE. AMEC. September 2012. (E)
18	18.1	ECEIG121115	B	UK EPR - Internal flooding - Multi-legged safety case and ALARP consequence assessment analyses. ECEIG121115 Revision B. September 2012. (E)
18	18.1	ECESN120038	B	EPR UK-GDA issue HF01- Organisation note and resources identification. ECESN120038 Revision B. EDF. June 2012. (E)
18	18.1	ECEIG070082	A	EPR - Organisation de la surveillance des études d'installation et suivi des modifications des données dans la maquette PDMS par bâtiment. [Process for Monitoring of Installation Studies and Tracking of PDMS model data modifications, by Building]. ECEIG070082 Revision A. EDF. June 2007. This document is only available in French and is provided for information only.
18	18.1	ECEF012001	A	Approach for integration of human factors in EPR design. ECEF012001 Revision A. EDF. December 2001. (E)
18	18.1	HT54200701446	1	Human Factors programme for evaluation of the EPR's operating means before the 1st fuel loading. HT54200701446 Version 1. EDF. May 2009. (E)
18	18.1	UKEPRI042	1	Human Factors Tracking Registers. UKEPR-I-042. Revision 1. EDF/AREVA. November 2012. (E)
18	18.1	UKEPRI003		UK EPR GDA Project - Design Change Procedure. UKEPR-I-003. EDF/AREVA. (E)
SUB-CHAPTER 18.2 – REFERENCES				

18	18.2	ECEFC121054	B	EPR GDA Design Basis Limits and Development of Plant Operating Limits and Maintenance Schedules. ECEFC121054 Revision A. EDF. November 2012. (E)
18	18.2	ECEF0100515	F1	Summary Report on Functional Requirements Analyses per Plant System Functional requirements analysis. ECEF0100515 Revision F1. EDF. June 2010 (E)
18	18.2	ECEF110174	A1	Definition of FA3 EPR Operating Ranges and Standard States. ECEF110174 Revision A1. October 2012. EDF. (E)
18	18.2	NEERF100697	A	Relation between operating domain and (P, T) curve issued from fast fracture analysis. NEER-F 100697 Revision A. AREVA. April 2010. (E)
18	18.2	NEPSFDC557	D	Classification of Structures, Systems and Components. NEPS-F DC 557 Revision D. AREVA. October 2012. (E)
18	18.2	NEEMFDC143	C	Reduction in Primary Circuit Radioactivity SFAIRP Based Upon the Primary Circuit Chemistry. NEEM-F DC 143 Revision C. AREVA. March 2011. (E)
18	18.2	PEEMFDC4	A	Reactor Chemistry – Topical Report. PEEM-F DC 4 Revision A. AREVA. June 2010. (E)
18	18.2	PEEMFDC5	A	Secondary Coolant Chemistry – Topical Report. PEEM-F DC 5 Revision A. AREVA. June 2010. (E)
18	18.2	ECEF091642	B1	EPR FA3 – Report on Identifying Chemical and Radiochemical Parameters for Operating Technical Specification (Level 1 Specifications). ECEF091642 Revision B1. EDF. October 2010. (E)
18	18.2	EDECME100781	A	Identification of the 'Level 2' Chemical and Radiochemical Parameters, as Part of the RGE Rules of the Flamanville EPR. EDECME100781 Revision A. EDF. November 2010. (E)
18	18.2	ECEF110138	A	Zinc Injection Implementation at UK-EPR. ECEF110138. Revision A. EDF. March 2011 (E)
18	18.2	ECEF110139	A	Zinc Injection claims, arguments and evidences: overall balance for UK-EPR. ECEF110139 Revision A. EDF. March 2011 (E)
18	18.2	ECEF110141	A	Hydrogen risk management at UK-EPR. ECEF110141. Revision A. EDF. March 2011 (E)
18	18.2	ECEF110140	A	Chemistry and Radiochemistry monitoring at UK-EPR primary coolant. ECEF110140 Revision A. EDF. March 2011 (E)
18	18.2	ECEF102664	A	Activity Management Methodology Applied to UK EPR. ECEF102664 Revision A. EDF. December 2010. (E)
18	18.2	ECEF110449	A	Activity management at UK-EPR Auxiliary Systems : system performances and control actions. ECEF110449 Revision A. EDF. June 2012. (E)
18	18.2	ECEF102665	A	Crud Evaluation for UK-EPR ECEF102665. Revision A. EDF. December 2010. (E)
18	18.2	ECEF110448	A	Analysis of UK EPR™ Source Term: Identification, Quantification and Characterisation. ECEF110448 Revision A. EDF. July 2011. (E)
18	18.2	ECEF110161	A	Tritium management - provisions and limits - EPR UK. ECEF110161 Revision A. EDF. February 2011. (E)
18	18.2	ECEF110142	A	Shutdown and start-up primary coolant chemistry/radiochemistry strategy for UK-EPR. ECEF110142. Revision A. EDF. March 2011 (E)
18	18.2	ECECS120520	A	Arguments and Evidences associated with the Chemistry and Radiochemistry Control for Primary Auxiliary Systems of UK-EPR. ECECS120520 Revision A, EDF. June 2012. (E)

18	18.2	NEPRFDC81	B	EPR FA3 Category 2 Loading Conditions Manual For The Main Primary System. NEPR F DC 81 Revision B. AREVA. December 2007. (E)
18	18.2	NEPRFDC82	C	EPR FA3 Category 3 and 4 Loading Conditions Manual For The Main Primary System. NEPR F DC 82 Revision C. AREVA. December 2007. (E)
18	18.2	RCCM2007		Design and Construction Rules for mechanical components of PWR nuclear islands (RCC-M). AFCEN. 2007 edition. (E)
18	18.2	PEPCF101041	3	UK EPR GDA - Neutronic Design Safety Analysis Bounding Limits. PEPCF.10.1041 Revision 3. AREVA. December 2010. (E)
18	18.2	FS10000607	3.0	Nuclear Fuel - Specification and Justification of Design and Safety Criteria. FS1-0000607 Revision 3.0. AREVA. October 2010. (E)
18	18.2	ENCNCA100114	B	Long term storage of spent fuel - Design criteria. ENCNCA100114 Revision B. EDF. August 2010. (E)
18	18.2	UKEPRI002		UK EPR GDA Project - Reference Design Configuration. UKEPR-I-002. EDF/AREVA. (E)
18	18.2	NEPSFDC576	A	UKEPR: Description of the C&I modelling in the PSA. NEPS-F DC 576 Revision A. AREVA. July 2010. (E)
18	18.2	NEPSFDC555	D	Safety Requirements for Non-Computerised Safety System (NCSS). NEPS-F DC 555 Revision D. June 2012. (E)
18	18.2	PEPCFDC14	A	General Principles for Definition of Core Physics Tests. PEPC-F DC 14 Revision A. AREVA. July 2010. (E)
18	18.2	ECEMA101028	A	F Chavigny. Demonstration of the accessibility and controllability for in-service inspection of the structural integrity components. ECEMA101028 Revision A. EDF. April 2010. (E)
18	18.2	NQSFDC1026	D	EPR - Flamanville accessibility and inspectability for the in-service inspection of the break preclusion areas - girth welds of the main primary loops and main steam lines. NQS-F DC 1026 Revision D. AREVA. October 2007. (E)
18	18.2	PEEMF110505	C	Ultrasonic examination of MCL homogeneous and dissimilar metal welds. PEEM-F 11.0505 Revision C. AREVA. March 2012. (E)
18	18.2	PEEMF110959	B	Ultrasonic Examination of MSL Girth Welds. PEEM-F 11.0959 Revision B. AREVA. March 2012. (E)
SUB-CHAPTER 18.3 – REFERENCES				
18	18.3	ECEF061266	A1	Procedure ENG 3-34 : Content and structure of Emergency Operating rule. ECEF061266 Revision A1. EDF. April 2009. (E) ECEF061266 Revision A1 is the English translation of ECEF061266 Revision A.
18	18.3	ECEF061275	A1	Procedure ENG 3-40: Content and structure of Emergency Operating methods. ECEF061275 Revision A1. EDF. July 2009. (E) ECEF061275 Revision A1 is the English translation of ECEF061275 Revision A.
18	18.3	ENFCRI090272	A	State-Oriented Approach designer knowledge transfer. ENFCRI090272 Revision A. EDF. November 2009. (E) ENFCRI090272 Revision A is the English translation of ENFCRI080034 Revision A
18	18.3	ENFCRI090295	A	SOA designer knowledge transfer - Appendix 1. ENFCRI090295 Revision A. EDF. December 2009. (E) ENFCRI090295 Revision A is the English translation of ENFCRI080224 Revision A

18	18.3	ECEF060191	A1	EPR HMI – Evaluation of the principles of computerised operation - Assessment of 2005 supplementary test campaign. ECEF060191 Revision A1. EDF. March 2010. (E) ECEF060191 Revision A1 is the English translation of ECEF060191 Revision A.
18	18.3	ECEF050143	C1	Automatic diagnosis principles for EPR emergency operation. ECEF050143 Revision C1. EDF. February 2012. (E)
18	18.3	ECEF100659	A	Principles to be used for the implementation of the Non-Computerised Safety System for Emergency Operating Procedure. ECEF100659 Revision A. EDF. March 2010. (E)
18	18.3	NEPRFDC551	C	EPR UK Functional Requirements on Non-Computerised Safety I&C Functions. NEPR-F DC 551 Revision C. AREVA. July 2012 (E)
18	18.3	NEPSFDC457	A	Severe Accident Management Operating Strategies for Severe Accidents FA3. NEPS F DC 457 Revision A. AREVA. June 2009 (E)
18	18.3	ISBN0717622401		A guide to the Radiation (Emergency Preparedness and Public Information) Regulations 2001 , ISBN 0 7176 2240 1. HSE (E)
18	18.3	ISBN0110856147		The Ionising Radiations Regulations 1999. Statutory Instrument 1999 No. 3232. ISBN 0 11-085614-7. The Stationery Office Ltd. (E)
CHAPTER 19				
SUB-CHAPTER 19.0 – REFERENCES				
19	19.0	ISBN0108502163		Nuclear Installations Act 1965. TSO (The Stationery Office). January 1965. ISBN 0108502163. (E)
19	19.0	ISBN0110729455		Regulatory Reform (Fire Safety) Order 2005 (SI 2005 No. 1541). TSO. June 2005. ISBN 0110729455. (E)
19	19.0	ISBN0110757890		Construction (Design and Management) Regulation 2007 (SI 2007 No. 320). TSO. February 2007. ISBN 0110757890. (E)
19	19.0	ISBN0110429575		Dangerous Substances and Explosive Atmospheres Regulations 2002, (SI 2002 No. 2776) (DSEAR). TSO. November 2002. ISBN 0110429575. (E)
19	19.0	ISBN0110827902		Pressure Equipment Regulations 1999, as amended in 2002. TSO. July 1999. ISBN 0110827902. (E)
19	19.0	ISBN0105437743		Health and Safety at Work Act 1974. TSO. October 1974. ISBN 0105437743. (E)
19	19.0	ISBN0110856252		Management of Health and Safety at Work Regulations 1999. (SI 1999 3242). TSO. April 2000. ISBN 0-11-085625-2. (E)
CHAPTER 20				
SUB-CHAPTER 20.1 – REFERENCES				

20	20.1	UKEPR0016001	1	GDA UK EPR Decommissioning. UKEPR-0016-001 Issue 01. EDF/AREVA. March 2011. (E)
SUB-CHAPTER 20.2 – REFERENCES				
20	20.2	REF0003		Process and Information Document for Generic Assessment of Candidate Nuclear Power Plant Designs. The Environment Agency. January 2007. (E)
20	20.2	UKEPR0017001	0	Decontamination processes and techniques for the UK EPR. UKEPR-0017-001 Issue 00. EDF/AREVA. March 2011. (E)
20	20.2	UKEPR0016001	1	GDA UK EPR Decommissioning. UKEPR-0016-001 Issue 01. EDF/AREVA. March 2011. (E)
20	20.2	ELIDC0801302	A	EPR UK – Decommissioning waste inventory. ELIDC0801302 Revision A. EDF. November 2008. (E)
20	20.2	NESHG2008en0123	A	Solid Radioactive Waste Strategy Report (SRWSR). AREVA NP. NESH-G/2008/en/0123 Rev A. November 2008. (E)
20	20.2	TN11261814		Generic Design Assessment: Summary of Disposability Assessment for Wastes and Spent Fuel arising from Operation of the UK EPR. NDA. Radioactive Waste Management Directorate. Technical Note no. 11261814. October 2009. (https://www.nda.gov.uk/documents/upload/TN-17548-Generic-Design-Assessment-Summary-of-Disposability-Assessment-for-Wastes-and-Spent-Fuel-arising-from-Operation-of-the-EPWR.pdf). (E)
CHAPTER 21				
SUB-CHAPTER 21.2 – REFERENCES				
21	21.2	UKEPRO001		UK EPR GDA Project Quality Assurance Plan. UKEPR-O-001. (E)
21	21.2	UKEPRI001		UK EPR GDA Project QA Documentation Master List and Format. UKEPR-I-001. (E)
21	21.2	SAP2006	1	UK Health and Safety Executive (HSE). Safety Assessment Principles for Nuclear Facilities. 2006 Edition Revision 1. January 2008. (E)
21	21.2	JPO003	2	Nuclear Power Station Generic Design Assessment. Interface Protocol between HSE Nuclear Directorate / Environment Agency and Requesting Parties. JPO/003 Issue 2. August 2008. (E)

APPENDIX 6: UK EPR GDA LEVEL 2 DOCUMENTS: PCER REFERENCES

Chapter	Sub-chapter	Reference	Issue	Title
CHAPTER 1				
SUB-CHAPTER 1.1 – REFERENCES				
1	1.1	REF0003	v1	Process and Information Document for Generic Assessment of Candidate Nuclear Power Plant Designs. The Environment Agency. January 2007. (E)
SUB-CHAPTER 1.4 – REFERENCES				
1	1.4	REF0005		Office for Nuclear Regulation Framework Document, as amended on 9 August and 23 November 2011. http://www.hse.gov.uk/nuclear/onr-framework.pdf
1	1.4	ISBN9780102156836		Health and Safety at Work etc Act, 1974. ISBN 978-010215683-6. The Stationery Office Ltd. (E)
1	1.4	ISBN0108502163		The Nuclear Installations Act 1965 (as amended). ISBN 0-10-850216-3. The Stationery Office Ltd. (E)
1	1.4	ISBN9780105425953		Environment Act 1995. ISBN 978-010542595-3. The Stationery Office Ltd. (E)
1	1.4	ISBN0105412937		The Radioactive Substances Act 1993. ISBN 0-10-541293-7. The Stationery Office Ltd (E)
1	1.4	9629EURATOM	Vol39	Council Directive 96/29/EURATOM of 13 May 1996 laying down basic safety standards for the protection of the health of workers and the general public against the dangers arising from ionizing radiation. Official Journal of the European Communities, L159, Vol. 39. June 1996. (E)
1	1.4	ICRP60		1990 Recommendations of the International Commission on Radiological Protection. ICRP Publication 60. Ann. ICRP 21 (1-3). 1991. (E)
1	1.4	ISBN0110856147		The Ionising Radiations Regulations 1999. Statutory Instrument 1999 No. 3232. ISBN 0-11-085614-7. The Stationery Office Ltd (E)
1	1.4	ISBN0717617467		Approved Code of Practice: Work with ionising radiation. Ionising Radiations Regulations 1999. Approved Code of Practice and guidance L121 HSE Books 2000 ISBN 0-71-761746-7. (E)
1	1.4	ISBN0110299086		The Radiation (Emergency Preparedness and Public Information) Regulations 2001 (REPPIR). Statutory Instrument 2001 No. 2975. ISBN 0-11-029908-6.HM Stationery Office. The Stationery Office Ltd (E)
1	1.4	REF0006		Radioactive Substances (Basic Safety Standards) (England and Wales) Direction 2000. Defra, UK. (E)
1	1.4	ISBN9780110996219		The Pollution Prevention and Control Regulations 2000 (PPC). ISBN 978-011099621-9. The Stationery Office Ltd. (E)
1	1.4	ISBN9780110440569		Waste Management Licensing Regulations 1994. ISBN 978-011044056-9. The Stationery Office Ltd. (E)

1	1.4	ISBN9780110726854		Hazardous Waste Regulations 2005. ISBN 978-011072685-4. The Stationery Office Ltd. (E)
1	1.4	ISBN9780105408901		The Town and Country Planning Act (1990) (TCP Act). ISBN 978-010540890-1. The Stationery Office Ltd. (E)
1	1.4	ISBN0110804740		Town and Country Planning EIA (England and Wales) Regulations 1999. ISBN 0-11-080474-0. The Stationery Office Ltd. (E)
1	1.4	ISBN0110186907		Town and Country Planning EIA (England and Wales) (Amended) Regulations 2000. ISBN 0-11-018690-7. The Stationery Office Ltd. (E)
1	1.4	ISBN0110754719		Town and Country Planning EIA (England and Wales) (Amended) Regulations 2006. ISBN 0-11-075471-9. The Stationery Office Ltd. (E)
1	1.4	ISBN0105429890		The Electricity Act 1989 ISBN 978-010542989-0. The Stationery Office Ltd. (E)
1	1.4	ISBN0110996752		The Electricity Works (Environmental Impact Assessment) (England and Wales) Regulations 2000 (SI 1927). ISBN 978-011099675-2. The Stationery Office Ltd. (E)
1	1.4	ISBN0110853956		The Nuclear Reactors (Environmental Impact Assessment for Decommissioning) Regulations 1999 (EIADR) (SI 2892). ISBN 978-011085395-6. The Stationery Office Ltd. (E)
1	1.4	ISBN0110858360		The Pressure Systems Safety Regulations 2000 (PSSR). Statutory Instruments 2000 No. 1281, ISBN 0-11-085836-0. The Stationery Office Ltd, (E)
1	1.4	ISBN0110827902		The Pressure Equipment Regulations 1999 (PER). Statutory Instruments 1999/2001, ISBN 978-011082790-2. The Stationery Office Ltd, (E)
1	1.4	REF0007		The Nuclear Installations (Dangerous Occurrences) Regulations 1965. Statutory Instruments 1965 No.1824. The Stationery Office Ltd. (E)
1	1.4	ISBN9780110966359		The Electricity at Work Regulations 1989. ISBN 9780110966359. The Stationery Office Ltd. (E)
1	1.4	ISBN0110429206		The Electricity Safety, Quality and Continuity Regulations 2002. ISBN 0 11 042920 6. The Stationery Office Ltd. (E)
1	1.4	ISBN0110729455		Regulatory Reform (Fire Safety) Order 2005. ISBN 0-11-072945-5. The Stationery Office Ltd. (E)
1	1.4	ISBN0110429575		Dangerous Substances and Explosive Atmospheres Regulations 2002 (DSEAR). ISBN 0-11-042957-5. The Stationery Office Ltd. (E)
1	1.4	ISBN9780110539997		The Equipment and Protective Systems (EPS) Intended for Use in Potentially Explosive Atmospheres Regulations 1996. ISBN 978-011053999-7. The Stationery Office Ltd. (E)
1	1.4	ISBN9780110821924		The Control of Major Accident Hazards (Amendment) Regulations 1999 (COMAH). ISBN 978-011082192-4. The Stationery Office Ltd. (E)
1	1.4	ISBN0110727665		Control of Major Accident Hazards (Amendment) Regulations 2005 (COMAH). ISBN 0-110-72766-5. The Stationery Office Ltd. (E)
1	1.4	ISBN9780105424017		The Anti-Terrorism, Crime and Security Act 2001 4. ISBN 978-010542401-7. The Stationery Office Ltd. (E)
1	1.4	ISBN0110453972		The Nuclear Industries Security Regulations 2003 (NISR). ISBN 0-11-045397-2. The Stationery Office Ltd. (E)
1	1.4	ISBN9780105457916		Water Resources Act 1991. ISBN 978-010545791-6. The Stationery Office Ltd. (E)

1	1.4	ISBN0110795997		Provision and Use of Work equipment Regulations 1998 (PUWER). ISBN 0-11-079599-7.The Stationery Office (E)
1	1.4	ISBN0110795989		Lifting Operations and Lifting Equipment Regulations 1998. ISBN 0-11-079598-9. The Stationery Office Ltd. (E)
1	1.4	ISBN0110258320		The Personal Protective Equipment at Work Regulations 1992.ISBN 0-11-025832-0. The Stationery Office Lt (E)
1	1.4	ISBN0110856252		Management of Health and Safety at Work Regulations 1999. ISBN 0-11-085625-2. The Stationery Office Ltd. (E)
1	1.4	ISBN0110757896		The Construction (Design And Management) Regulations 2007. ISBN 978-011075789-6. The Stationery Office Ltd. (E)
1	1.4	ICRPPub63		ICRP. Principles for intervention for protection of the public in a radiological emergency. ICRP Publication 63, Annals of the ICRP 22 (4). 1993 (E)
1	1.4	ISBN9201042957		IAEA. International Basic Safety Standards for protection from ionising radiation and security with respect to radiation sources. Safety series No 115. ISBN 92-0-104295-7. IAEA Vienna, 1996. (E)
1	1.4	IAEA50CSGQ		IAEA. Quality Assurance for Safety in Nuclear Power Plants and other Nuclear Installations, Code and Safety Guides Q1–Q14. Safety Series No. 50-C/SG-Q, IAEA, Vienna (1996). (E)
1	1.4	IAEANSR1		Safety of Nuclear Power Plants: Design (Requirements for Design). ISSN 1020-525X IAEA Safety Standards Series N° NS-R-1. IAEA. 2000. (E)
1	1.4	ASNGUIDE201		ASN Guide 2/01. Considering the risk of an earthquake when designing civil engineering structures of nuclear installations, excluding longterm storage of radioactive waste. May 2006. (E)
1	1.4	REF0008		Technical Guidelines for the design and construction of the next generation of nuclear pressurised water plant units adopted during plenary meetings of the GPR and German experts on the 19 and 26 October 2000. (E)
1	1.4	RFSGB200101		Basic Safety Rule - Fundamental safety rule n°2001-01 concerning basic nuclear installations. RFS-GB-2001-01. ASN. (E)
1	1.4	RFS12d		Basic Safety Rule – Rule 1.2.d. Protection against risks associated to the industrial environment and the roads. May 1982. (E)
1	1.4	RFS12e		Basic Safety Rule – Rule 1.2.e. Protection against external flooding. April 1984. (E)
SUB-CHAPTER 1.5 – REFERENCES				
1	1.5	2007AV0016		Statement concerning the draft decree authorising the creation of the basic Flamanville 3 nuclear installation, including an EPR type nuclear reactor, on the Flamanville (Channel) site. Statement No. 2007-AV-0016. (French) Nuclear Safety Authority, 16 February 2007.(Avis relatif au projet de décret autorisant la création de l'installation nucléaire de base dénommée Flamanville 3, comportant un réacteur nucléaire de type EPR, sur le site de Flamanville (Manche). Avis n° 2007-AV- 0016. Autorité de Sûreté Nucléaire. 16 février 2007)

1	1.5	REF0008		"Technical Guidelines for the design and construction of the next generation of nuclear pressurized water plant units" adopted during plenary meetings of the GPR and German experts on the 19 and 26 October 2000. (French) Nuclear Safety Authority, 20 October 2000. (E)
1	1.5	2008DC0114		Decision with regard to Électricité de France - Société Anonyme (EDF-SA) setting the requirements concerning the nuclear power reactor site at Flamanville (Manche department) for the design and construction of the "Flamanville 3" reactor (BNI 167) and for operation of the "Flamanville 1" (BNI 108) and "Flamanville 2" (BNI 109) reactors. Decision No. 2008-DC-0114. (French) Nuclear Safety Authority, 26 September 2008. (Décision fixant à Électricité de France – Société Anonyme (EDF-SA) les prescriptions relatives au site électronucléaire de Flamanville (Manche) pour la conception et la construction du réacteur « Flamanville 3 » (INB n°167) et pour l'exploitation des réacteurs « Flamanville 1 » (INB n°108) et « Flamanville 2 » (INB n°109). Décision n°2008-DC-0114. ASN (Autorité de sûreté nucléaire). 26 septembre 2008)
1	1.5	ECEP102828		Robert Pays. EPR FA3 – Transmission of the first working draft version of the start-up authorisation file. Letter ECEP102828. EDF Direction Production Ingénierie. 29 October 2010. (Robert Pays. EPR FA3 – Transmission de la première version de travail du dossier de demande de mise en service. Lettre ECEP102828. EDF Direction Production Ingénierie. 29 Octobre 2010)
1	1.5	REF0009		WENRA Reactor Safety Reference Levels. Western European Nuclear Regulators' Association, Reactor Harmonization Working Group, January 2007. (E)
1	1.5	EURVOL1	B	European Utility Requirements for LWR nuclear power plants) Volume 1, Chapter C3, Revision B. EUR, Villeurbanne, France. (E)
1	1.5	EURVOL3EPR	B	European Utility Requirements for LWR nuclear power plants) Volume 3 – Standard EPR Subset. Revision B. EUR, Villeurbanne, France. July 2009. (E)
1	1.5	EURVOL1	C	European Utility Requirements for LWR nuclear power plants) Volume 1, Revision C. EUR, Villeurbanne, France. April 2001. (E)
CHAPTER 2				
SUB-CHAPTER 2.2 – REFERENCES				
2	2.2	UKEPRO001		UK EPR GDA Project Quality Assurance Plan. UKEPR-O-001. (E)
2	2.2	UKEPRI001		UK EPR GDA Project QA Documentation Master List and Format. UKEPR-I-001. (E)
2	2.2	SAP2006	1	UK Health and Safety Executive (HSE). Safety Assessment Principles for Nuclear Facilities. 2006 Edition Revision 1. January 2008. (E)
2	2.2	JPO003	2	Nuclear Power Station Generic Design Assessment. Interface Protocol between HSE Nuclear Directorate / Environment Agency and Requesting Parties. JPO/003 Issue 2. August 2008. (E)
CHAPTER 3				

SUB-CHAPTER 3.1 – REFERENCES				
3	3.1	REF0003	v1	Process and Information Document for Generic Assessment of Candidate Nuclear Power Plant Designs. The Environment Agency. January 2007. (E)
SUB-CHAPTER 3.3 – REFERENCES				
3	3.3	ISBN0110726847		The List of Wastes (England) Regulations 2005 . The Stationery Office Ltd. SI No. 895. ISBN0110726847.(E)
3	3.3	ISBN9780110726854		Hazardous Waste (England and Wales) Regulations 2005 . ISBN 978-011072685-4. The Stationery Office Ltd. (E)
3	3.3	ECEF110139	A	Zinc Injection claims, arguments and evidences: overall balance for UK-EPR. ECEF110139 Revision A. EDF. March 2011 (E)
3	3.3	200317EC		Directive 2003/17/EC of the European Parliament and of the Council of 3 March 2003 amending Directive 98/70/EC relating to the quality of petrol and diesel fuels. (E)
3	3.3	REF0003	v1	Process and Information Document for Generic Assessment of Candidate Nuclear Power Plant Designs. The Environment Agency. January 2007. (E)
3	3.3	ISBN0110996219		Pollution Prevention and Control Regulations 2000 (PPC). SI No. 1973. The Stationery Office Ltd. ISBN 978-011099621-9. (E)
3	3.3	UKEPR0004001	0	GDA UK EPR - PPC application – Generic information for UK EPR diesel generators. UKEPR-0004-001. Issue 00. EDF/AREVA. June 2008. (E)
3	3.3	IPPCS101	v2.03	IPPC Technical Guidance for the Combustion Sector S1.01. V2.03. Environment Agency & Environment and Heritage Service & Scottish Environment Protection Agency. July 2005 . (E)
3	3.3	ISBN9780105457916		Water Resources Act 1991. ISBN 978-0105457916. The Stationery Office Ltd.(E)
3	3.3	ISBN9780110821924		The Control of Major Accident Hazards Regulations 1999 (COMAH). ISBN 978-011082192-4. The Stationery Office Ltd. (E)
3	3.3	ISBN0110727665		Control of Major Accident Hazards (Amendment) Regulations 2005. ISBN 01-10-72766-5. The Stationery Office Ltd. (E)
SUB-CHAPTER 3.4 – REFERENCES				
3	3.4	ECEF110139	A	Zinc Injection claims, arguments and evidences: overall balance for UK-EPR. ECEF110139 Revision A. EDF. March 2011 (E)
CHAPTER 4				
SUB-CHAPTER 4.1 – REFERENCES				
4	4.1	REF0003	v1	Process and Information Document for Generic Assessment of Candidate Nuclear Power Plant Designs. The Environment Agency. January 2007. (E)

SUB-CHAPTER 4.3 – REFERENCES				
4	4.3	ISBN9780110726854		Hazardous Waste Regulations 2005 . ISBN 978-011072685-4. The Stationery Office Ltd. (E)
4	4.3	ISBN0105457916		Water Resources Act 1991. ISBN 978-0105457916. The Stationery Office Ltd.(E)
CHAPTER 5				
SUB-CHAPTER 5.1 – REFERENCES				
5	5.1	UKEPR0016001	1	GDA UK EPR Decommissioning. UKEPR-0016-001 Issue 01. EDF/AREVA. March 2011. (E)
SUB-CHAPTER 5.2 – REFERENCES				
5	5.2	REF0003		Process and Information Document for Generic Assessment of Candidate Nuclear Power Plant Designs. The Environment Agency. January 2007. (E)
5	5.2	UKEPR0017001	0	Decontamination processes and techniques for the UK EPR. UKEPR-0017-001 Issue 00. EDF/AREVA. March 2011. (E)
5	5.2	UKEPR0016001	1	GDA UK EPR Decommissioning. UKEPR-0016-001 Issue 01. EDF/AREVA. March 2011. (E)
5	5.2	ELIDC0801302	A	EPR UK – Decommissioning waste inventory. ELIDC0801302 Revision A. EDF. November 2008. (E)
5	5.2	NESHG2008en0123	A	Solid Radioactive Waste Strategy Report (SRWSR). NESH-G/2008/en/0123 Rev A. AREVA NP. November 2008. (E)
5	5.2	TN11261814		Generic Design Assessment: Summary of Disposability Assessment for Wastes and Spent Fuel arising from Operation of the UK EPR. NDA. Radioactive Waste Management Directorate. Technical Note no. 11261814. (https://www.nda.gov.uk/documents/upload/TN-17548-Generic-Design-Assessment-Summary-of-Disposability-Assessment-for-Wastes-and-Spent-Fuel-arising-from-Operation-of-the-EPWR.pdf.). October 2009. (E)
CHAPTER 6				
SUB-CHAPTER 6.0 – REFERENCES				
6	6.0	ISBN0105443905		Environmental Protection Act 1990. HM Stationery Office. November 1990, reprinted January 2004. ISBN 0105443905.
6	6.0	ISBN9780105457916		Water Resources Act 1991 . ISBN 978-010545791-6. The Stationery Office Ltd. (E)
6	6.0	ISBN0105424994		The Pollution Prevention and Control Act 1999 (as amended). HM Stationery Office. August 1999. ISBN 0105424994.
6	6.0	ISBN0105412937		The Radioactive Substances Act 1993. HM Stationery Office. ISBN 0-10: 0105412937.
6	6.0	ISBN0108502163		The Nuclear Installations Act 1965 (as amended). HM Stationery Office. ISBN 0108502163. (E)
6	6.0	ISBN9780105425953		Environment Act 1995. ISBN 978-010542595-3. The Stationery Office Ltd. (E)
6	6.0	ISBN 0105427918		Radioactive Material (Road Transport) Act 1991. HM Stationery Office. July 1991. ISBN 0105427918. (E)

6	6.0	REF0006		Radioactive Substances (Basic Safety Standards) (England and Wales) Direction 2000. DEFRA. (E)
6	6.0	ISBN0110856147		The Ionising Radiations Regulations 1999. Statutory Instrument 1999 No. 3232. HM Stationery Office. ISBN 0-11-085614-7. (E)
6	6.0	ISBN0110774698		The Carriage of Dangerous Goods and Use of Transportable Pressure Equipment Regulations 2007. HM Stationery Office. July 2007. ISBN 0110774698. (E)
6	6.0	1999829EURATOM		Treaty establishing the European Atomic Energy Community (1957). EUR-Lex. See also COMMISSION RECOMMENDATION of 6 December 1999 on the application of Article 37 of the Euratom Treaty (1999/829/Euratom). (E)
6	6.0	TAST024		UK Health and Safety Executive (HSE). Technical Assessment Guide, Management of Radioactive Materials and Radioactive Waste on Nuclear Licensed Sites. T/AST/024 Issue 003. March 2001. (E)
6	6.0	9723EC		Directive 97/23/EC of the European Parliament and of the Council of 29 May 1997 on the approximation of the laws of the Member States concerning pressure equipment. (E)
SUB-CHAPTER 6.1 – REFERENCES				
6	6.1	REF0003		Process and Information Document for Generic Assessment of Candidate Nuclear Power Plant Designs. The Environment Agency. January 2007. (E)
6	6.1	ENTERP090062	A	Primary Source Term of the EPR Reactor. ENTERP090062 Revision A. EDF. March 2009. (E) ENTERP090062 Revision A is the English translation of ENTERP070147 Rev A.
6	6.1	ECEIG101686	B	Use of source term in the different GDA areas. ECEIG101686 Revision B. EDF. November 2010. (E)
6	6.1	ECEF110139	A	Zinc Injection claims, arguments and evidences: overall balance for UK-EPR. ECEF110139 Revision A. EDF. March 2011 (E)
6	6.1	ECEF110448	A	Analysis of UK EPR™ Source Term: Identification, Quantification and Characterisation. ECEF110448 Revision A. EDF. July 2011. (E)
6	6.1	ECEF111022	A	Corrosion product characterization under PWR/EPR primary coolant conditions: thermodynamic assessments and power plant feedback. ECEF111022 Revision A. EDF. July 2011. (E)
SUB-CHAPTER 6.2 – REFERENCES				
6	6.2	REF0003		Process and Information Document for Generic Assessment of Candidate Nuclear Power Plant Designs. The Environment Agency. January 2007. (E)
6	6.2	SFLEZS030008	G	System Design Manual – Coolant Purification, Degasification, Storage and Treatment System (TEP [CSTS]), P2 – System Operation. SFL-EZS-030008 Revision G. Sofinel. October 2008. (E)
6	6.2	SFLEZS030009	F	System Design Manual – Coolant Purification, Degasification, Storage and Treatment System (TEP [CSTS]) P3 – Sizing of the system and its components. SFL-EZS030009 Revision F. Sofinel. October 2008. (E)
6	6.2	SFLEZS030010	E	System Design Manual – Coolant Purification, Degasification, Storage and Treatment System (TEP [CSTS]) P4 – Flow diagrams. SFL-EZS030010 Revision E. SOFINEL. September 2008. (E)

6	6.2	EZS2006en0008	A	System Design Manual – Coolant Purification, Degasification, Storage and Treatment System (TEP [CSTS]) P4.1 – Functional flow diagrams. EZS2006/en/0008 Revision A. SOFINEL. October 2006. (E)
6	6.2	EZS2006en0009	F	System Design Manual – Coolant Purification, Degasification, Storage and Treatment System (TEP [CSTS]) P4.2 – Detailed flow diagrams. EZS/2006/en/0009 Revision F. Sofinel. October 2008. (E)
6	6.2	SFLEZS030077	F	System Design Manual – Coolant Purification, Degasification, Storage and Treatment System (TEP [CSTS]) P5 – Instrumentation and Control. SFL-EZS-030077 Revision F. Sofinel. November 2008. (E)
6	6.2	DARPEFLABII3		DARPE FLA. Authorization request for water intake and liquid and gaseous releases of the Flamanville site. Part B. Chapter II.3 - Gaseous and liquid radioactive releases. (E)
6	6.2	SFLEZS030046	G	System Design Manual – Steam Generator Blowdown System (APG [SGBS]) P2 –System Operation. SFL-EZS-030046 Revision G. Sofinel. December 2008. (E)
6	6.2	SFLEZS030047	G	System Design Manual – Steam Generator Blowdown System (APG [SGBS]) P3 – Sizing of the system and its components. SFL-EZS-030047 Revision G. Sofinel. December 2008. (E)
6	6.2	ETDOIG060270	A1	System Design Manual – Specifications for describing the (SEK [CILWDS]) plant system in the turbine hall and site galleries in the Flamanville 3 EPR. ETDOIG060270 Revision A1. EDF. November 2009. (E)
6	6.2	23952SEK00014SH	C	Dossier de Système Elémentaire – SEK DSE P4.2: Schéma mécanique détaillé. [System Design Manual – Conventional Island liquid waste discharge system SEK [CILWDS]), Part 4.2 – Detailed Flow Diagrams] 23952SEK00014SH Revision C. Assystem. September 2008.
6	6.2	23952SEK00025DSE	C1	System Design Manual – Conventional Island liquid waste discharge system (SEK [CILWDS]), Part 5 – Instrumentation and Control. 23952SEK00025DSE Revision C1. Assystem. December 2008. (E)
6	6.2	SFLEZS030050	D	System Design Manual - Gaseous Waste Processing System (TEG [GWPS]), P1 - History of the System Manual. SFL–EZS 030050 Revision D. Sofinel. October 2008. (E)
6	6.2	SFLEZS030051	E	System Design Manual - Gaseous Waste Processing System (TEG[GWPS]), P2 – System operation. SFLEZS030051 Revision E. Sofinel. September 2008. (E)
6	6.2	SFLEZS030052	E	System Design Manual - Gaseous Waste Processing System (TEG[GWPS]), P3 – Sizing of the System and its Components. SFLEZS030052 Revision E. Sofinel. September 2008. (E)
6	6.2	SFLEZS030053	D	System Design Manual - Gaseous Waste Processing System (TEG[GWPS]) P4 – Flow diagrams. SFL–EZS 030053 Revision D. Sofinel. September 2008. (E)
6	6.2	ECEF080592	A	System Design Manual - Gaseous Waste Processing System (TEG[GWPS]) P4.1 – Functional flow diagrams. ECEF080592 Revision A. EDF. April 2008. (E)
6	6.2	EZS2007en0007	D	System Design Manual - Gaseous Waste Processing System (TEG[GWPS]) P4.2 – Detailed flow diagrams. EZS/2007/en/0007 Revision D. Sofinel. September 2008. (E)
6	6.2	EZS2007en0015	C	System Design Manual - Gaseous Waste Processing System (TEG[GWPS]) P5 – Instrumentation and Control. EZS 2007/en/0015 Revision C. Sofinel. October 2008. (E)
6	6.2	EZL2006EN0093	E	System Design Manual – Nuclear Auxiliary Building Ventilation System (DWN [NABVS]), P2 – System operation. EZL2006/en/0093 Revision E. Sofinel. November 2008. (E)

6	6.2	EZL2006EN0092	F	System Design Manual – Nuclear Auxiliary Building Ventilation System (DWN [NABVS]), P3 - Sizing Report. EZL2006/en/0092 Revision F. Sofinel. November 2008. (E)
6	6.2	SFLEFMF2006164	D1	System Design Manual – Fuel Building Ventilation System (DWK [FBVS]), P2 - System Operation. SFL-EF MF 2006.164 Revision D1. Sofinel. October 2009. (E)
6	6.2	SFLEZL030008	G	System Design Manual – Safeguard Building Controlled Area Ventilation System (DWL [CSBVS)], P2 – System operation (Stage 1)SFL-EZL-030008 Revision G. EDF. November 2008. (E)
6	6.2	EHZ2007EN0033	E	System Design Manual – Safeguard Building Controlled Area Ventilation System (DWL [CSBVS]) P3 – Sizing of the system and its components (Stage 1). EZH/2007/en/0033 Revision E. Sofinel. November 2008. (E)
6	6.2	EYTS2007FR0131	B1	System Design Manual – Containment Sweep Ventilation System (EBA [CSVS]) P2 - System Operation. EYTS2007/fr/0131 Revision B1. Sofinel. November 2009. (E)
6	6.2	SFLEFMF2006502	B1	System Design Manual – Containment Sweep Ventilation System (EBA [CSVS]), P3 – Design of System and its Components. SFL-EFMF-2006.502 Revision B1. Sofinel. March 2009. (E)
6	6.2	SFLEFMF2006131	D1	System Design Manual – Operating Building Contaminable Room Ventilation System (8DWB), P2 - System Operation (Stage 2). SFL-EF MF 2006.131 Revision D1. Sofinel. August 2009. (E)
6	6.2	SFLEFMF2006132	E1	System Design Manual – Operating Building Contaminable Room Ventilation System (8DWB), P3 - Design of System and Components (Stage 2). SFL-EF MF 2006.132 Revision E1. Sofinel. August 2009. (E)
6	6.2	EYTS2007FR0108	B1	System Design Manual – Effluent Treatment Building Ventilation System (8DWQ) P2 - System Operation. EYTS2007/fr/0108 Revision B1. Sofinel. November 2009. (E)
6	6.2	EYTS2007FR0109	B1	System Design Manual – Effluent Treatment Building Ventilation System (8DWQ) P3 - System and component design. EYTS2007/fr/0109 Revision B1. Sofinel. November 2009. (E)
6	6.2	EYTS2007FR0187	D1	System Design Manual – Peripheral Room Ventilation System (DWW [ABVS]) P2 - System Operation. EYTS2007/fr/0187 Revision D1. Sofinel. November 2009. (E)
6	6.2	DARPEFLABII41		DARPE FLA. Authorization request for water intake and liquid and gaseous releases of the Flamanville site. Part B. Chapter II.4.1 - Chemicals associated with liquid radioactive effluents and drainage water discharged into the sea. (E)
6	6.2	UKEPR0011001	6	GDA UK EPR - BAT Demonstration. UKEPR-0011-001 Issue 06. EDF/AREVA. August 2012. (E)
6	6.2	UKEPR0010001	3	GDA UK EPR – Integrated Waste Strategy Document. UKEPR-0010-001 Issue 03. EDF/AREVA. June 2012. (E)
6	6.2	UKEPR0012001	2	GDA UK EPR - Mapping Document for Radioactive Waste Management Case. UKEPR-0012-001 Issue 02. EDF/AREVA. March 2011. (E)
6	6.2	NESHG2008en0123	A	Solid Radioactive Waste Strategy Report (SRWSR). AREVA NP. NESH-G/2008/en/0123 Rev A. November 2008. (E)
6	6.2	GUIDANCE2010		Guidance from the Health and Safety Executive, the Environment Agency and the Scottish Environment Protection Agency to nuclear licensees . The management of higher activity radioactive waste on nuclear licensed sites, radioactive waste management cases. February 2010. (E)

6	6.2	REF0016		Policy for the Long Term Management of Solid Low Level Radioactive Waste in the United Kingdom. DEFRA, March 2007. (E)
6	6.2	ECUK110016App4	A	EPR - Radioactive waste conditioning - ECUK110016 Revision A. Appendix 4. EDF. January 2011. (E)
6	6.2	ECEF080440	A1	System Design Manual – Effluent Treatment Building Solid Waste Treatment System (8TES [SWTS]) P1. ECEF080440 Revision A1. EDF. January 2010. (E)
6	6.2	ECEF070007	B1	System Design Manual – Effluent Treatment Building Solid Waste Treatment System (8TES [SWTS]) P2 – System operation. ECEF070007 Revision B1. EDF. January 2010. (E)
6	6.2	ECEF070008	B1	System Design Manual – Effluent Treatment Building Solid Waste Treatment System (8TES [SWTS]), P3 – System dimensioning. ECEF070008 Revision B1. EDF. January 2010. (E)
6	6.2	ECEF080573	A1	System Design Manual – Effluent Treatment Building Solid Waste Treatment System (8TES [SWTS]), P4 – Flow diagrams. ECEF080573 Revision A1. EDF. January 2010. (E)
6	6.2	ECEF080624	A	Dossier de Système Elémentaire - Traitement des effluents solides du BTE 8TES [SWTS] – Part 4.1 – Schéma Mécanique Fonctionnel. [System Design Manual - Effluent Treatment Building Solid Waste Treatment System 8TES [SWTS], P4.2 – Functional Flow Diagrams]. ECEF080624 Revision A. EDF. April 2008.
6	6.2	ECEF071498	B	Dossier de Système Elémentaire - Traitement des effluents solides du BTE 8TES [SWTS]– Part 4.2 - Schéma Mécanique Detaille [System Design Manual - Effluent Treatment Building Solid Waste Treatment System 8TES [SWTS], P4.2 - Detailed Flow Diagrams]. ECEF071498 Revision B. EDF. January 2008.
6	6.2	ECEF080380	A1	System Design Manual – Effluent Treatment Building Solid Waste Treatment System (8TES [SWTS]), P5 – Instrumentation and Control. ECEF080380 Revision A1. EDF. January 2010. (E)
6	6.2	SFLEZM200118	B	System Design Manual – Solid Waste Treatment System (TES [SWTS]) – Filter changing equipment TES11 P2 – System operation. SFLEZM200118 Revision B. Sofinel. September 2006. (E)
6	6.2	SFLEZM200119	B	System Design Manual – Solid Waste Treatment System (TES [SWTS]) – Filter changing equipment TES11 P3 – Sizing of the system and its components. SFLEZM200119 Revision B. Sofinel. September 2006. (E)
6	6.2	SFLEZS030039	D	System Design Manual – Solid Waste Treatment System (TES [SWTS]) – Transfer of the spent resins (TES21, TES31) P1. SFLEZS030039 Revision D. Sofinel. March 2007. (E)
6	6.2	SFLEZS030040	D	System Design Manual – Solid Waste Treatment System (TES [SWTS])– Transfer of the spent resins (TES21, TES31) P2 – System operation. SFL-EZS030040 Revision D. Sofinel. March 2007. (E)
6	6.2	SFLEZS030041	D	System Design Manual – Solid Waste Treatment System (TES [SWTS])– Transfer of the spent resins (TES21, TES31) P3 – Sizing of the system and its components. SFL-EZS030041 Revision D. Sofinel. March 2007. (E)
6	6.2	SFLEZS030042	D	System Design Manual – Solid Waste Treatment System (TES [SWTS]) – Transfer of the spent resins (TES21, TES31) P4 – Flow diagrams. SFL-EZS030042 Revision D. Sofinel. March 2007. (E)

6	6.2	EZS2006en0019	A	System Design Manual – Solid Waste Treatment System (TES [SWTS]) – Transfer of the spent resins (TES21, TES31), P4.1 – Simplified flow diagrams. EZS/2006/en/0019 Revision A. SOFINEL. March 2007. (E)
6	6.2	EZS2006en0020	B	System Design Manual – Solid Waste Treatment System (TES [SWTS]) – Transfer of the spent resins (TES21, TES31), P4.2 – Detailed flow diagrams. EZS/2006/en/0020 Revision B. SOFINEL. March 2007. (E)
6	6.2	320L027		Form D1 Application: UK EPR Project, from LLW Repository Ltd. 320.L.027. December 2008. (E)
6	6.2	ECUK110016App1	A	EPR - Radioactive waste conditioning - ECUK110016 Revision A. Appendix 1. EDF. January 2010. (E)
6	6.2	CoRWM700		Managing our Radioactive Waste Safely - CoRWM's Recommendations to Government. CoRWM Document 700. July 2006. (E)
SUB-CHAPTER 6.3 – REFERENCES				
6	6.3	REF0003		Process and Information Document for Generic Assessment of Candidate Nuclear Power Plant Designs. The Environment Agency. January 2007. (E)
6	6.3	C20034832		Commission recommendation of 18 December 2003 on standardised information on radioactive airborne and liquid discharges into the environment from nuclear power reactors and reprocessing plants in normal operation. C(2003)4832, 2004/2/Euratom. Official Journal of the European Union. 6 January 2004. (E)
6	6.3	UKEPR0011001	6	GDA UK EPR - BAT DEMONSTRATION. UKEPR-0011-001 Issue 06. EDF/AREVA. August 2012. (E)
6	6.3	NESHG2008en0123	A	Solid Radioactive Waste Strategy Report (SRWSR). AREVA NP. NESH-G/2008/en/0123 Rev A. November 2008. (E)
6	6.3	ECMT050107	A1	Projected assessment of solid radioactive waste from the EPR. ECMT050107 Revision A1. EDF. November 2009. (E) ECMT050107 Revision A1 is the English translation of ECMT050107 Revision A.
6	6.3	D450711001253	0	B Lantes. EPR waste arising over the operating life. D450711001253 Version 0. EDF. March 2011 (E)
6	6.3	SFLEZM200118	B	System Design Manual – Solid Waste Treatment System (TES [SWTS]) – Filter changing equipment TES11 P2 – System operation. SFL-EZM200118 Revision B. Sofinel. September 2006. (E)
6	6.3	SFLEZM200119	B	System Design Manual – Solid Waste Treatment System (TES [SWTS]) – Filter changing equipment TES11 P3 – Sizing of the system and its components. SFL-EZM200119 Revision B. Sofinel. September 2006. (E)
6	6.3	SFLEZS030040	D	System Design Manual – Solid Waste Treatment System (TES [SWTS])– Transfer of the spent resins (TES21, TES31) P2 – System operation. SFL-EZS030040 Revision D. Sofinel. March 2007. (E)
6	6.3	SFLEZS030041	D	System Design Manual – Solid Waste Treatment System (TES [SWTS])– Transfer of the spent resins (TES21, TES31) P3 – Sizing of the system and its components. SFL-EZS030041 Revision D. Sofinel. March 2007. (E)
6	6.3	SFLEZS030039	D	System Design Manual – Solid Waste Treatment System (TES [SWTS]) – Transfer of the spent resins (TES21, TES31) P1. SFL-EZS030039 Revision D. Sofinel. March 2007. (E)
6	6.3	SFLEZS030042	D	System Design Manual – Solid Waste Treatment System (TES [SWTS]) – Transfer of the spent resins (TES21, TES31) P4 – Flow diagrams. SFL-EZS030042 Revision D. Sofinel. March 2007. (E)

6	6.3	EZS2006en0019	A	System Design Manual – Solid Waste Treatment System (TES [SWTS]) – Transfer of the spent resins (TES21, TES31), P4.1 – Simplified flow diagrams. EZS/2006/en/0019 Revision A. SOFINEL. March 2007. (E)
6	6.3	EZS2006en0020	B	System Design Manual – Solid Waste Treatment System (TES [SWTS]) – Transfer of the spent resins (TES21, TES31), P4.2 – Detailed flow diagrams. EZS/2006/en/0020 Revision B. SOFINEL. March 2007. (E)
6	6.3	UKEPR0017001	0	Decontamination Processes and Techniques for the UKEPR. UKEPR-0017-001 Issue 00. EDF/AREVA. March 2011. (E)
6	6.3	ISODIS21238		Nuclear Energy - Nuclear Fuel Technology - The Scaling Factor method to determine the radioactivity of low and intermediate level radioactive waste packages generated at nuclear power plant. ISO/DIS 21238. (E)
6	6.3	ENTERP090190		Methods for limiting the production and discharge of tritium for the EPR. ENTERP090190. EDF. August 2009. (E) ENTERP090190 is the English translation of ENTERP040216 Revision A
6	6.3	ECEP050315	A1	Analysis of environmental performance in the EPR France project. ECEP050315 Revision A1. EDF. March 2012. (E) ECEP050315 Revision A1 is the English translation of ECEP050315 Revision A.
6	6.3	ECEF110138	A	Zinc Injection Implementation at UK-EPR. ECEF110138. Revision A. EDF. March 2011 (E)
6	6.3	ECEF110139	A	Zinc Injection claims, arguments and evidences: overall balance for UK-EPR. ECEF110139 Revision A. EDF. March 2011 (E)
6	6.3	ECEF110448	A	Source Term: Identification, Quantification and Characterisation. ECEF110448 Revision A. EDF. June 2011. (E)
6	6.3	ECEF110449	A	Activity management at UK-EPR Auxiliary Systems : system performances and control actions. ECEF110449 Revision A. EDF. June 2012. (E)
6	6.3	ENTERP090183		EPR – Production and discharge of C14. ENTERP090183. EDF. August 2009. (E) ENTERP090183 is the English translation of ENTERP050168 Revision C.
6	6.3	DARPEFLACIV1TABLEC		DARPE FLA (Authorization request for water intake and liquid and gaseous releases of the Flamanville site) Part C. Chapter IV-1 Table C-IV.1.1-j - Radionuclides and activities discharged considered in the maximum ("requested limits") gaseous discharge calculations for the EPR unit. (E)
6	6.3	ECEF060592	A1	DARPE FLA (Authorization request for water intake and liquid and gaseous releases of the Flamanville site) Annexe B5e. Expected results, excluding hazards of discharges of liquid chemical substances associated with radioactive effluents and waste water from the machine rooms for the EPR nuclear reactor. ECEF060592 Revision A1. EDF. May 2009. (E) ECEF060592 Revision A1 is the English translation of ECEF060592 Revision A
6	6.3	ECEF050301	C1	DARPE FLA (Authorization request for water intake and liquid and gaseous releases of the Flamanville site) Annexe B5c. ECEF050301 Revision C1. EDF. May 2009. (E) ECEF050301 Revision C1 is the English translation of ECEF050301 Revision C.
6	6.3	DARPEFLABIII4		DARPE FLA (Authorization request for water intake and liquid and gaseous releases of the Flamanville site) Part B Chapter III.4 Table B-III.4.1.4.a - Expected performance excluding contingencies and maximum annual additional discharges for chemicals associated with radioactive effluents. (E)

6	6.3	EDEAPC050176	D1	DARPE FLA (Authorization request for water intake and liquid and gaseous releases of the Flamanville site) Annexe B5d. EDEAPC050176 Revision D1. EDF. November 2009. (E) EDEAPC050176 Revision D1 is the English translation of EDEAPC050176 Revision D.
6	6.3	EDEAPC050094	D1	DARPE FLA (Authorization request for water intake and liquid and gaseous releases of the Flamanville site) Annexe B5a. EDEAPC050094 Revision D1. EDF. November 2009. (E) EDEAPC050094 Revision D1 is the English translation of EDEAPC050094 Revision D.
6	6.3	DARPEFLABIII42		DARPE FLA (Authorization request for water intake and liquid and gaseous releases of the Flamanville site) Part B Chapter III.4.2 ~~2~~ - Other chemicals discharged into the sea - and associated annexes. (E)
SUB-CHAPTER 6.4 – REFERENCES				
6	6.4	REF0003		Process and Information Document for Generic Assessment of Candidate Nuclear Power Plant Designs. The Environment Agency. January 2007. (E)
6	6.4	ECUK110016App4	A	EPR - Radioactive waste conditioning - ECUK110016 Revision A. Appendix 4. EDF. January 2011. (E)
6	6.4	ECUK110016App3	A	EPR - Radioactive waste conditioning - ECUK110016 Revision A. Appendix 3. EDF. January 2011. (E)
6	6.4	SFLEZS030018	F	System Design Manual – Nuclear Island Vent and Drain System (RPE [NVDS]) - P2 - System Operation. BNI - SFLEZS030018 Revision F. Sofinel. October 2008. (E)
6	6.4	NESSFDC608	A	System Design Manual – Nuclear Island Vent and Drain System (RPE [NVDS]) - P2 - System Operation. RB - NESS-F DC 608 Revision A. AREVA. November 2009. (E)
6	6.4	EYTF2008FR0010	A1	System Design Manual – Nuclear Island Vent and Drain System (8RPE [ETBVDS]) P2 - System Operation. ETB - EYTF2008/fr/0010 Revision A1. EDF. September 2009. (E)
6	6.4	SFLEZS030019	F	System Design Manual - Nuclear Island Vent and Drain System (RPE [NVDS]) - P3 – Sizing of the System and its Components. BNI - SFLEZS030019 Revision F. Sofinel. July 2008. (E)
6	6.4	NESSFDC609	A	System Design Manual - Nuclear Island Vent and Drain System (RPE [NVDS]) P3 – Sizing of the System and its Components. RB - NESS-F DC 609 Revision A. AREVA. November 2009. (E)
6	6.4	SFLEZS030020	D	System Design Manual - Nuclear Island Vent and Drain System (RPE [NVDS]) P4 – Flow Diagrams. BNI - SFLEZS030020 Revision D. SOFINEL. October 2007. (E)
6	6.4	EZS2007EN0021	A	System Design Manual - Nuclear Island Vent and Drain System (RPE [NVDS]) - P4.1 : Functional Flow Diagrams, BNI - EZS2007EN0021. Revision A. Sofinel. October 2007. (E)
6	6.4	EZS2007EN0022	C	System Design Manual - Nuclear Island Vent and Drain System (RPE [NVDS]) - P4.2 : Detailed Flow Diagrams : BNI - EZS2007EN0022 Revision C. Sofinel. November 2008. (E)
6	6.4	NESSFDC610	A	System Design Manual - Nuclear Island Vent and Drain System (RPE [NVDS]) P4 – Flow Diagrams. RB - NESS-F DC 610 Revision A. AREVA. November 2009. (E)
6	6.4	EYTF2007FR0044	C1	System Design Manual - Nuclear Island Vent and Drain System (8RPE [ETBVDS]) P4 – Flow Diagrams. ETB - EYTF2007/fr/0044 Revision C1. EDF. October 2009. (E)
6	6.4	EYTF2007FR0043	C	System Design Manual - Nuclear Island Vent and Drain System (8RPE [ETBVDS]) P4.2 – Detailed Flow Diagrams. ETB - EYTF2007/fr/0043 Revision C. EDF. October 2008.

6	6.4	SFLEZS030055	F	System Design Manual - Nuclear Island Vent and Drain System (RPE [NVDS]) - P5 – Instrumentation and Control. BNI - SFLEZS030055 Revision F. Sofinel. November 2008. (E)
6	6.4	NESSFDC616	A	System Design Manual - Nuclear Island Vent and Drain System (RPE [NVDS]) P5 – Instrumentation and Control. RB - NESS-F DC 616 Revision A. AREVA. November 2009. (E)
6	6.4	EYTF2008FR0047	B1	System Design Manual - Nuclear Island Vent and Drain System (RPE [ETBVDS]) P5 – Instrumentation and Control. ETB - EYTF2008/fr/0047 Revision B1. EDF. October 2009. (E)
6	6.4	ECEF070502	A1	System Design Manual – Liquid Waste Processing System (TEU [LWPS]) Part 2: Operation of the TEU [LWPS] EPR FA3 system ECEF070502 Revision A1. EDF. January 2010. (E)
6	6.4	ECEF070503	A1	System Design Manual – Liquid Waste Processing System (TEU [LWPS]) Part 3 : Design basis of the system and its components. ECEF070503 Revision A1. EDF. January 2010. (E)
6	6.4	ECEF070504	A1	System Design Manual – Liquid Waste Processing System (TEU [LWPS]) Part 4 : Mechanical Flow Diagrams ECEF070504 Revision A1. EDF. January 2010. (E)
6	6.4	ECEF072208	A	Dossier Système Elémentaire - Traitement Des Effluents Uses, P4.1: Schéma Mécanique Fonctionnel. [System Design Manual – Liquid Waste Processing System (TEU [LWPS]) P4.1, Functional flow diagram]. ECEF072208 Revision A. EDF. November 2007.
6	6.4	ECEF071497	B	Dossier Système Elémentaire - Traitement Des Effluents Uses, P4.2: Schéma Mécanique Detaille. [System Design Manual – Liquid Waste Processing System (TEU [LWPS]) P4.2, Detailed flow diagram]. ECEF071497 Revision B. EDF. January 2008.
6	6.4	ECEF071379	A	Dossier de Système Elémentaire – TEU DSE P5: Contrôle-commande. [System Design Manual – Liquid Waste Processing System (TEU [LWPS]) P5 : Instrumentation and Control] ECEF071379 Revision A. EDF. December 2008.
6	6.4	EYTS2008fr0016	A1	System Design Manual – Effluent Treatment Building Sampling System (TEN) P1. EYTS2008/fr/0016 Revision A1. EDF. August 2009. (E)
6	6.4	EYTS2007fr0038	B1	System Design Manual – Effluent Treatment Building Sampling System (TEN) P2 - System Operation. EYTS2007/fr/0038 Revision B1. EDF. August 2009. (E)
6	6.4	EYTS2007fr0174	B1	System Design Manual - Effluent Treatment Building Sampling System (TEN) P3 – System and component design basis. EYTS2007/fr/0174 Revision B1. EDF. August 2009. (E)
6	6.4	EYTS2007fr0040	B1	System Design Manual - Effluent Treatment Building Sampling System (TEN) P4 – Mechanical Diagrams. EYTS2007/fr/0040 Revision B1. EDF. August 2009. (E)
6	6.4	EYTS2007fr0101	A	Dossier Système Elémentaire – Circuit d'échantillonnage des effluents du BTE (TEN), P4.1 - Schéma Mécanique Fonctionnel. [System Design Manual – Effluent Treatment Building Sampling System (TEN), P4.1 - Functional flow diagram]. EYTS2007/fr/0101 Revision A. EDF. November 2007.
6	6.4	EYTS2007fr0039	B	Dossier Système Elémentaire – Circuit d'échantillonnage des effluents du BTE (TEN), P4.2 - Schéma Mécanique Detaille. [System Design Manual – Effluent Treatment Building Sampling System (TEN), P4.2 - Detailed flow diagram.] EYTS2007/fr/0039 Revision B. EDF. November 2007.
6	6.4	DARPEFLABII3		DARPE FLA (Authorization request for water intake and liquid and gaseous releases of the Flamanville site) Part B Chapter II.3 - Gaseous and liquid radioactive releases. (E)

6	6.4	23952SEK00014SH	C	Dossier de Système Elémentaire – SEK DSE P4.2: Schéma mécanique détaillé [System Design Manual – Site Liquid Waste Discharge System (SEK [CILWDS]) Part 4.2 : Detailed Flow Diagrams] 23952 SEK 00014SH Revision C. Assystem. September 2008.
6	6.4	23952SEK00025DSE	C1	System Design Manual – Conventional Island Liquid Waste Discharge System (SEK [SiteCIL]) P5 : Instrumentation and Control. 23952SEK00025DSE Revision C1. Assystem. December 2008. (E)
6	6.4	EYTS2007FR0228	B1	System Design Manual - Waste Building Component Cooling System (TRI) - Part 1 History. EYTS/2007/fr/0228 Revision B1. Sofinel. November 2009. (E)
6	6.4	EYTS2007FR0036	B1	System Design Manual - Waste Building Component Cooling System (TRI) - Part 2 System Operation. EYTS/2007/fr/0036 Revision B1. EDF. November 2009. (E)
6	6.4	EYTS2007FR0037	C1	System Design Manual - Waste Building Component Cooling System (TRI) - Part 3 System and component design basis. EYTS/2007/fr/0037 Revision C1. EDF. November 2009. (E)
6	6.4	EYTS2007FR0070	C1	System Design Manual - Waste Building Component Cooling System (TRI) - Part 4 Mechanical diagrams. EYTS/2007/fr/0070 Revision C1. EDF. November 2009. (E)
6	6.4	EYTS2007FR0069	C	Dossier Système Elémentaire – (TRI), P4.2 - Schéma Mécanique Detaille. [System Design Manual – Waste Building Component Cooling System (TRI), P4.2 - Detailed flow diagram.] EYTS2007/fr/0069 Revision C. EDF. August 2008.
6	6.4	EYTS2007FR0115	D1	System Design Manual - Waste Building Component Cooling System (TRI) - Part 5 Instrumentation and Control. EYTS/2007/fr/0115 Revision D1. EDF. November 2009. (E)
6	6.4	SFLEZS030050	D	System Design Manual - Gaseous Waste Processing System (TEG [GWPS]), P1 - History of the System Manual. SFL–EZS 030050 Revision D. Sofinel. October 2008. (E)
6	6.4	SFLEZS030051	E	System Design Manual - Gaseous Waste Processing System (TEG[GWPS]), P2 – System operation. SFLEZS030051 Revision E. Sofinel. September 2008. (E)
6	6.4	SFLEZS030052	E	System Design Manual - Gaseous Waste Processing System (TEG [GWPS]), P3 – System design. SFL–EZS 030052 Revision E. Sofinel. September 2008. (E)
6	6.4	SFLEZS030053	D	System Design Manual - Gaseous Waste Processing System (TEG[GWPS]) P4 – Flow diagrams. SFL–EZS 030053 Revision D. Sofinel. September 2008. (E)
6	6.4	ECEF080592	A	System Design Manual - Gaseous Waste Processing System (TEG [GWPS]) P4.1 – Functional flow diagrams. ECEF080592 Revision A. EDF. April 2008. (E)
6	6.4	EZS2007en0007	D	System Design Manual - Gaseous Waste Processing System (TEG [GWPS]) P4.2 – Detailed flow diagrams. EZS/2007/en/0007 Revision D. Sofinel. September 2008. (E)
6	6.4	EZS2007en0015	C	System Design Manual - Gaseous Waste Processing System (TEG [GWPS]), P5 – Instrumentation and Control EZS 2007/en/0015 Revision C. Sofinel. October 2008. (E)
6	6.4	ENTERP070291	A	EPR: activity concentrations in the TEP, REA, TEG, RPE, TEU and TES auxiliary systems. ENTERP070291 Revision A. EDF. November 2007. (E). ENTERP070291 Revision A is the English translation of ENTERP070070 Revision A.
6	6.4	SFLEZM200118	B	System Design Manual – Solid Waste Treatment System (TES [SWTS]) – Filter changing equipment TES11, P2 – System operation. SFLEZM200118 Revision B. Sofinel. September 2006. (E)

6	6.4	SFLEZM200119	B	System Design Manual – Solid Waste Treatment System (TES [SWTS]) – Filter changing equipment TES11, P3 – Sizing of the system and its components. SFLEZM200119 Revision B. Sofinel. September 2006. (E)
6	6.4	SFLEZS030039	D	System Design Manual – Solid Waste Treatment System (TES [SWTS]) – Transfer of the spent resins (TES21, TES31), P1. SFLEZS030039 Revision D. Sofinel. March 2007. (E)
6	6.4	SFLEZS030040	D	System Design Manual – Solid Waste Treatment System (TES [SWTS])– Transfer of the spent resins (TES21, TES31) P2 – System operation. SFL-EZS030040 Revision D. Sofinel. March 2007. (E)
6	6.4	SFLEZS030041	D	System Design Manual – Solid Waste Treatment System (TES [SWTS])– Transfer of the spent resins (TES21, TES31) P3 – Sizing of the system and its components. SFL-EZS030041 Revision D. Sofinel. March 2007. (E)
6	6.4	SFLEZS030042	D	System Design Manual – Solid Waste Treatment System (TES [SWTS]) – Transfer of the spent resins (TES21, TES31) P4 – Flow diagrams. SFL-EZS030042 Revision D. Sofinel. March 2007. (E)
6	6.4	EZS2006en0019	A	System Design Manual – Solid Waste Treatment System (TES [SWTS]) – Transfer of the spent resins (TES21, TES31), P4.1 – Simplified flow diagrams. EZS/2006/en/0019 Revision A. SOFINEL. March 2007. (E)
6	6.4	EZS2006en0020	B	System Design Manual – Solid Waste Treatment System (TES [SWTS]) – Transfer of the spent resins (TES21, TES31), P4.2 – Detailed flow diagrams. EZS/2006/en/0020 Revision B. SOFINEL. March 2007. (E)
6	6.4	NESHG2008en0123	A	Solid Radioactive Waste Strategy Report (SRWSR). AREVA NP. NESH-G/2008/en/0123 Rev A. November 2008. (E)
SUB-CHAPTER 6.5 – REFERENCES				
6	6.5	REF0003		Process and Information Document for Generic Assessment of Candidate Nuclear Power Plant Designs. The Environment Agency. January 2007. (E)
6	6.5	WASTEPLANTS		The required level of design of waste plants for new build reactors in the Generic Design Assessment. May 2009. (http://www.hse.gov.uk/newreactors/wasteplants.pdf) (E)
6	6.5	WASTEMAN2010		The management of higher activity radioactive waste on nuclear licensed sites, radioactive waste management cases, Guidance from the Health and Safety Executive, the Environment Agency and the Scottish Environment Protection Agency to nuclear licensees, February 2010. (http://www.hse.gov.uk/nuclear/wastemanage/RWM-part2.pdf) (E)
6	6.5	UKEPR0012001	2	GDA UK EPR - Mapping Document for Radioactive Waste Management Case. UKEPR-0012-001 Issue 02. EDF/AREVA. March 2011. (E)
6	6.5	ISBN0108502163		The Nuclear Installations Act 1965 (as amended). HM Stationery Office. ISBN 0108502163. (E)
6	6.5	ISBN0110856147		The Ionising Radiations Regulations 1999. Statutory Instrument 1999 No. 3232. HM Stationery Office. ISBN 0-11-085614-7. (E)
6	6.5	ISBN0110299086		The Radiation (Emergency Preparedness and Public Information) Regulations 2001 (REPPIR). Statutory Instrument 2001 No. 2975. ISBN 0-11-02990- 6.HM Stationery Office. The Stationery Office Ltd. (E)

6	6.5	ISBN0110856252		Management of Health and Safety at Work Regulations 1999. ISBN 0-11-085625-2. The Stationery Office Ltd. (E)
6	6.5	ISBN9780110742427		The Nuclear Reactors (Environmental Impact Assessment for Decommissioning) (Amendment) Regulations 2006 (EIADR) (SI 2006 No. 657). ISBN 978-0110742427. The Stationery Office Ltd. (E)
6	6.5	ISBN0105412937		The Radioactive Substances Act 1993. HM Stationery Office. ISBN 0-10: 0105412937. (E)
6	6.5	REF0017		Management of Radioactive Materials And Radioactive Waste On Nuclear Licensed Sites. (http://www.hse.gov.uk/nuclear/waste1.pdf). (E)
6	6.5	ISBN0110795997		Provision and Use of Work equipment Regulations 1998 (PUWER). ISBN 0-11--9599--7.The Stationery Office Ltd. (E)
6	6.5	ISBN0110795989		Lifting Operations and Lifting Equipment Regulations 1998. ISBN 0-11-079598-9. The Stationery Office Ltd. (E)
6	6.5	ISBN0110258320		The Personal Protective Equipment at Work Regulations 1992.ISBN 0-11-025832--. The Stationery Office Ltd. (E)
6	6.5	ISBN0110858360		The Pressure Systems Safety Regulations 2000 (PSSR). Statutory Instruments 2000 No. 1281, ISBN 0-11-085836-0. The Stationery Office Ltd. (E)
6	6.5	ISBN0110727665		Control of Major Accident Hazards (Amendment) Regulations 2005 (COMAH). ISBN 0--10-72766-5. The Stationery Office Ltd. (E)
6	6.5	ISBN0110429575		Dangerous Substances and Explosive Atmospheres Regulations 2002 (DSEAR). ISBN 0-11-042957-5. The Stationery Office Ltd. (E)
6	6.5	UKEPR0008001	1	ILW Interim Storage Facility. UKEPR-0008-001 Issue 01. EDF/AREVA. November 2009. (E)
6	6.5	R10006A	2	Human Factors in Long Term Waste Management. R10-006 (A) Issue 2. AREVA. February 2010. (E)
6	6.5	ELIDC0902019	B	Plan for the Development of Waste Management Facilities over the EPR Lifetime. ELIDC0902019 Revision B. EDF. March 2010. (E)
6	6.5	ELI0800226	A	Dry Interim Storage facility for ILW. ELI0800226 Rev A. November 2008. (E)
6	6.5	NESHG2008en0123	A	Solid Radioactive Waste Strategy Report (SRWSR). AREVA NP. NESH-G/2008/en/0123 Rev A. November 2008. (E)
6	6.5	UKEPR0016001	1	GDA UK EPR Decommissioning. UKEPR-0016-001 Issue 01. EDF/AREVA. March 2011. (E)
6	6.5	UKEPR0009001	1	Spent Fuel Interim Storage Facility. UKEPR-0009-001 Issue 01. EDF/AREVA. November 2009. (E)
6	6.5	SGNNT100330200004	A	Maintenance of Interim Storage Facilities. SGN NT 100330 20 0004 Revision A. AREVA. February 2010. (E)
6	6.5	ELI0800224	A	Interim storage facility for spent fuel assemblies coming from an EPR plant. ELI0800224 Revision A. EDF. November 2008. (E)
6	6.5	ELIDC1000094	A	The safety of long-term spent fuel storage in ponds. ELIDC1000094 Revision A. EDF. April 2010. (E)
6	6.5	NT10033020005	A	Current French and International Research and Development Programmes for Interim Storage of Spent Fuel. SGN NT 100330 20 005 Revision A. AREVA. February 2010. (E)
6	6.5	320L027		Form D1 Application: UK EPR Project, from LLW Repository Ltd. 320.L.027. December 2008. (E)

6	6.5	REF0018		Disposability Assessment of Solid Waste Arisings from New Build. NDA, Radioactive Waste Management Directorate. April 2008. (E)
6	6.5	TN11261814		Generic Design Assessment: Summary of Disposability Assessment for Wastes and Spent Fuel arising from Operation of the UK EPR. NDA. Radioactive Waste Management Directorate. Technical Note no. 11261814. (https://www.nda.gov.uk/documents/upload/TN-17548-Generic-Design-Assessment-Summary-of-Disposability-Assessment-for-Wastes-and-Spent-Fuel-arising-from-Operation-of-the-EPWR.pdf.). October 2009. (E)
6	6.5	NXA10747397	2	Generic Design Assessment: Disposability Assessment for Wastes and Spent Fuel arising from Operation of the UK EPR. Part 1: Main Report. NXA/10747397 Issue 2. January 2010. (E)
6	6.5	NXA10777960	2	Generic Design Assessment: Disposability Assessment for Wastes and Spent Fuel arising from Operation of the UK EPR. Part 2: Data Sheets and Inventory Tables. NXA/10777960 Issue 2. January 2010. (E)
6	6.5	R10017	1	The case for disposability of spent fuel and ILW. R10-017 (A) Issue 1. AREVA. February 2010. (E)
6	6.5	REGEPR00637R		Letter dated 14th January 2011 from EDF/AREVA to HSE/EA “Consolidation of the Plan for the Disposability of Waste and Spent Fuel”, REG EPR00637R. (E)
6	6.5	NT100330000003	B	Technical note. Encapsulation Facilities for Spent Fuel. NT 100330 00 0003 Revision B. AREVA/EDF. September 2010. (E)
6	6.5	R010002A	3	Management of Records for Long Term Management of Spent Fuel and ILW. R010 002 (A) Issue 3. AREVA. February 2010. (E)
CHAPTER 7				
SUB-CHAPTER 7.1 – REFERENCES				
7	7.1	UKEPR0007001	3	Monitoring of liquid and gaseous discharges: Prospective arrangements for the UK EPR. UKEPR-0007-001 I03. EDF/AREVA. August 2012.
7	7.1	REF0003	v1	Process and Information Document for Generic Assessment of Candidate Nuclear Power Plant Designs. The Environment Agency. January 2007. (E)
7	7.1	EATGNM11		Monitoring of radioactive releases to Atmosphere from Nuclear Facilities, Technical Guidance Note (Monitoring) M11. Environment Agency. 1999
7	7.1	EATGNM12		Monitoring of radioactive releases to Water from Nuclear Facilities, Technical Guidance Note (Monitoring) M12. Environment Agency. 1999
7	7.1	EATGNM18	v1	Monitoring of discharges to water and sewer, Technical Guidance Note (Monitoring) M18, Version 1. Environment Agency. 2004
SUB-CHAPTER 7.2 – REFERENCES				
7	7.2	ISBN0105412937		The Radioactive Substances Act 1993. HM Stationery Office. ISBN 0-10 : 0105412937. (E)

7	7.2		v1	Process and information Document for Applications for New Authorisations issued under the Radioactive Substances Act 1993 to Nuclear Sites in England and Wales, Version 1. Environment Agency. December 2007. (E)
7	7.2	EATGNM11		Monitoring of radioactive releases to Atmosphere from Nuclear Facilities, Technical Guidance Note (Monitoring) M11. Environment Agency. 1999. (E)
7	7.2	EATGNM12		Monitoring of radioactive releases to Water from Nuclear Facilities, Technical Guidance Note (Monitoring) M12. Environment Agency. 1999. (E)
7	7.2	IPPCGuide	4	Integrated Pollution Prevention and Control Practical Guide Edition 4. DEFRA. 2005. (E)
7	7.2	ISBN0110996219		Pollution Prevention and Control Regulations 2000. SI No. 1973. The Stationery Office Ltd. ISBN 978-011099621-9. (E)
7	7.2	REF0154		Environmental Permitting Core Guidance for the Environmental Permitting (England and Wales) Regulations 2007. DEFRA. 2008. (E)
7	7.2	ISBN9780105425953		Environment Act 1995. ISBN 978-010542595-3. The Stationery Office Ltd. (E)
7	7.2	CMD7169		The Air Quality Strategy for England, Scotland, Wales and Northern Ireland. DEFRA. 2007. (E)
7	7.2	EATGNM1	v4	Sampling requirements for stack-emission monitoring. Technical Guidance Note (Monitoring) M1. Version 4. Environment Agency. 2006. (E)
7	7.2	EATGNM2	v4.2	Monitoring of stack emissions to air. Technical Guidance Note (Monitoring) M2. Version 4.2. Environment Agency. 2007. (E)
7	7.2	EATGNM8		Environmental Monitoring Strategy- Ambient Air. Technical Guidance Note (Monitoring) M8. Environment Agency. 2000. (E)
7	7.2	EATGNM9		Monitoring Methods for Ambient Air. Technical Guidance Note (Monitoring) M9. Environment Agency. 2000. (E)
7	7.2	EATGNM15	v1	Monitoring PM10 and PM2.5. Technical Guidance Note (Monitoring) M15. Version 1. Environment Agency. 2006
7	7.2	EATGNM16	v1	The measurement and monitoring of volatile organic compounds to air from industrial installations. Technical Guidance Note (Monitoring) M16. Version 1. Environment Agency. 2005. (E)
7	7.2	ISBN9780105457916		Water Resources Act, 1991. ISBN 978-010545791-6. The Stationery Office Ltd (as amended)
7	7.2	ISBN978011079799X		Groundwater Regulations 1998, SI No. 2746. The Stationery Office Ltd. ISBN 978-011079799X. (E)
7	7.2	ISBN0110620518		Surface Waters (Fishlife) Classification Regulations 1997, SI No. 1331. The Stationery Office Ltd. ISBN 0110620518. (E)
7	7.2	ISBN0110457161		The Conservation (Natural Habitats, & C.) Regulations 1994. SI No. 2716. The Stationery Office Ltd. ISBN 0110457161. (E)
7	7.2	ISBN0110637089		Surface Waters (Shellfish) Classification Regulations 1997, SI No. 1332. The Stationery Office Ltd. ISBN 0110637089. (E)
7	7.2	EATGNM18	v1	Monitoring of discharges to water and sewer. Technical Guidance Note (Monitoring) M18. Version 1. Environment Agency. 2004. (E)
SUB-CHAPTER 7.3 – REFERENCES				

7	7.3	ISBN0100105412937		The Radioactive Substances Act 1993. HM Stationery Office. ISBN 0-10: 0105412937. (E)
7	7.3	EATGNM11		Monitoring of radioactive discharges to Atmosphere from Nuclear Facilities. Technical Guidance Note (Monitoring) M11. Environment Agency. 1999. (E)
7	7.3	20042EuratomC20034832		Official Journal of the European Union. Commission recommendation of 18 December 2003 on standardised information on radioactive airborne and liquid discharges into the environment from nuclear power reactors and reprocessing plants in normal operation. (E)
7	7.3	EATGNM12		Monitoring of radioactive discharges to Water from Nuclear Facilities. Technical Guidance Note (Monitoring) M12. Environment Agency. 1999. (E)
SUB-CHAPTER 7.4 – REFERENCES				
7	7.4	ISBN0100105412937		The Radioactive Substances Act 1993. HM Stationery Office. ISBN 0-10: 0105412937. (E)
7	7.4	EATGNM18	v1	Monitoring of discharges to water and sewer. Technical Guidance Note (Monitoring) M18. Version 1. Environmental Agency, 2004. (E)
7	7.4	IPPCGuide	4	Integrated Pollution prevention and Control Practical Guide. Edition 4. DEFRA. 2005. (E)
CHAPTER 8				
SUB-CHAPTER 8.1 – REFERENCES				
8	8.1	REF0003		Process and Information Document for Generic Assessment of Candidate Nuclear Power Plant Designs. The Environment Agency. January 2007. (E)
8	8.1	UKEPR0011001	6	GDA UK EPR - BAT DEMONSTRATION. UKEPR-0011-001 Issue 06. EDF/AREVA. August 2012. (E)
8	8.1	UKEPR0010001	3	GDA UK EPR – Integrated Waste Strategy Document. UKEPR-0010-001 Issue 03. EDF/AREVA. June 2012. (E)
8	8.1	9661EC		European Community Directive 96/61/EC, Integrated Pollution Prevention and Control, 1996 . (E)
8	8.1	ISBN0105424994		The Pollution Prevention and Control Act 1999 (as amended). HM Stationery Office. August 1999. ISBN 0105424994. (E)
8	8.1	ISBN0110996219		The Pollution Prevention and Control Regulations 2000. SI No. 1973. The Stationery Office Ltd. ISBN 978-011099621-9. (E)
8	8.1	EAHGNH1		Integrated Pollution Prevention and Control (IPPC) – Environmental Assessment and Appraisal of BAT. Horizontal Guidance Note IPPC H1. Environment Agency & Environment and Heritage Service & Scottish Environment Protection Agency. July 2003. (E)
8	8.1	BPEOGUIDE		Guidance for the Environment Agencies' Assessment of Best Practicable Environmental Options Studies at Nuclear Sites. Environment Agency and Scottish Environment Protection Agency. February 2004. (E)
8	8.1	PB7288		UK Strategy for Radioactive Discharges 2001-2020. DEFRA. July 2002. (E)
8	8.1	ISBN9264021469		Effluent release options from nuclear installations. Technical background and regulatory aspects. OECD 2003. (E)

SUB-CHAPTER 8.2 – REFERENCES				
8	8.2	REF0003		Process and Information Document for Generic Assessment of Candidate Nuclear Power Plant Designs. The Environment Agency. January 2007. (E)
8	8.2	CM7296		Meeting the Energy Challenge – a White Paper on Nuclear Power. Department for Business Entreprise and Regulatory Reform (DBERR), January 2008 . (E)
8	8.2	URN07970		The Future of Nuclear Power – The Role of Nuclear Power in a Low Carbon UK Economy. Consultation Document First published May 2007, Department of Trade and Industry. (www.dti.gov.uk) , © Crown Copyright. DTI/Pub 8519/4k/05/07/NP. URN 07/970 . (E)
8	8.2	POSTNOTE268		Postnote number 268. Carbon Footprint of electricity generation. Parliamentary Office of Science and Technology. October 2006. (E)
8	8.2	ASN132193		French Nuclear Regulatory Agency (ASN) letter 1321/93, dated July 22th 1993 . (E)
8	8.2	REF0008		Technical Guidelines for the Design and Construction of the Next Generation of Nuclear Power Plants with PWRs, Group for Nuclear Reactor (GPR)/German experts plenary meetings October 19th and 26th, 2000 . (E)
8	8.2	ASN9892004 / ASN7292004		French Nuclear Regulatory Agency (ASN) letters 989/2004, dated January 14th 2004, and 729/2004, dated September 28th 2004
8	8.2	ECEP040220	A1	Minutes of EPR Environment Review Meeting of October 14th, 2004. ECEP040220 Revision A1. EDF. July 2009. (E)
8	8.2	ECEP071263		Environment Committee. Rapid meeting minutes. ECEP071263. EDF. July 2009. (E)
8	8.2	ECEP081223		Environment Committee. Rapid meeting minutes. ECEP081223. EDF. July 2009. (E)
8	8.2	ECEP081892		Environment Committee. Rapid meeting minutes. ECEP081892. EDF. July 2009. (E)
8	8.2	ISBN9264021469		Effluent release options from nuclear installations. Technical background and regulatory aspects. OECD. 2003. (E)
8	8.2	9661EC		European Community Directive 96/61/EC Integrated Pollution and Control. 1996. (E)
8	8.2	NEEMFDC143	C	Reduction in Primary Circuit Radioactivity SFAIRP Based Upon the Primary Circuit Chemistry. NEEM-F DC 143 Revision C. AREVA. March 2011. (E)
8	8.2	NEEMFDC140	A	Reduction in primary circuit radioactivity SFAIRP based upon the Primary Circuit Materials. NEEM-F DC 140 Revision A. AREVA. March 2010. (E)
8	8.2	EDECME100223	A	Zinc injection benefits and its implementation feasibility for the UK-EPR. EDECME100223 Revision A. EDF. February 2010. (E)
8	8.2	PEEMFDC19	A	Reactor Chemistry – Zinc Assessment for EPR UK. PEEM-F DC 19 Revision A. AREVA. September 2010. (E)
8	8.2	ECEF110139	A	Zinc Injection claims, arguments and evidences: overall balance for UK-EPR. ECEF110139 Revision A. EDF. March 2011 (E)
8	8.2	ECEF110138	A	Zinc Injection Implementation at UK-EPR. ECEF110138. Revision A. EDF. March 2011 (E)

8	8.2	ECEF110448	A	Analysis of UK EPR™ Source Term: Identification, Quantification and Characterisation. ECEF110448 Revision A. July 2011. (E)
8	8.2	PB7288		UK Strategy for Radioactive Discharges 2001-2020. Defra. July 2002. (E)
8	8.2	ISBN0100105412937		The Radioactive Substances Act 1993. HM Stationery Office. ISBN 0-10: 0105412937. (E)
8	8.2	DETR2000		Statutory Guidance on the Regulation of Radioactive Discharges into the Environment from Nuclear-Licensed Sites, Consultation Paper. Department of the Environment, Transport and the Regions (DETR). London, 2000.
8	8.2	ECEF110449	A	Activity management at UK-EPR Auxiliary Systems : system performances and control actions. ECEF110449 Revision A. June 2012. (E)
8	8.2	EDCME100828	D	UK EPR Generic Design Assessment – Assessment for the filtration and demineralisation systems of the CSTS, LWPS and SGBS systems. EDCME100828 Revision D. EDF. January 2011. (E)
8	8.2			Abatement of Radioactive Releases to Water from Nuclear Facilities. Technical Guidance Note (Abatement) A6. Environment Agency. 1999. (E)
8	8.2	IAEATECDOC1336		Combined methods for liquid radioactive waste treatment – final report of a co-ordinated research project 1997-2001. IAEA-TECDOC-1336. IAEA 2003. (E)
8	8.2	SC010034SR		Developing Guidance for setting limits on radioactive discharges to the environment from nuclear-licensed sites. Science Report: SC010034/SR. Environment Agency 2005. (E)
8	8.2	HE2304023	A1	Decontamination of active effluents by membrane technology or electrolysis EDF HE-23/04/023/A1. March 2012. (E). EDF HE-23/04/023 Revision A1 is the English translation of EDF HE-23/04/023 Revision A.
8	8.2	EDLCHM080118	A1	Tests of an Isotopic Retention Technology – Case for Tritium, carbone-14, radiochemical and chemical compounds. EDF EDLCHM080118 A1. April 2012. (E). EDF EDLCHM080118 Revision A1 is the english translation of EDF EDLCHM080118 Revision A.
8	8.2	REF0155		Decision Document and Authorisations for future regulation of disposals of radioactive waste under the Radioactive Substances Act 1993 at British Energy Generation Limited's Nuclear Sites. Dungeness B Power Station; Hartlepool Power Station; Heysham 1 Power Station; Heysham 2 Power Station; Hinkley Point B Power Station; Sizewell B Power Station. Environment Agency. December 2006. (E)
8	8.2	REF0156		Generic Design Assessment of New Nuclear Power Plant Designs. Statements of findings following preliminary assessment of the submission by AREVA NP SAS and Electricité de France SA for their UK EPR design. Environment Agency. March 2008. (E)
8	8.2	OSPAR914		The Application of BAT in UK Nuclear Facilities Report (UK's Report on the Implementation of PARCOM Recommendation 91/4). OSPAR Commission, 2005.
8	8.2	20042Euratom		Commission recommendation of 18 December 2003 on standardised information on radioactive airborne and liquid discharges into the environment from nuclear power reactors and reprocessing plants in normal operation. (2004/2 Euratom). Off. J. Eur. Union, L2, 6.1.2004. (E)
SUB-CHAPTER 8.3 – REFERENCES				

8	8.3	REF0003		Process and Information Document for Generic Assessment of Candidate Nuclear Power Plant Designs. The Environment Agency. January 2007. (E)
8	8.3	ISBN0110996219		Pollution Prevention and Control Regulations 2000. SI No. 1973. The Stationery Office Ltd. ISBN 978-011099621-9. (E)
8	8.3	EAHGNH1		Integrated Pollution Prevention and Control (IPPC) – Environmental Assessment and Appraisal of BAT. Horizontal Guidance Note IPPC H1. Environment Agency & Environment and Heritage Service & Scottish Environment Protection Agency. July 2003. (E)
8	8.3	ISBN9780105457916		Water Resources Act, 1991. ISBN 978-0105457916. The Stationery Office Ltd. (E)
8	8.3	ISBN978011079799X		Groundwater Regulations 1998. SI No. 2746. The Stationery Office Ltd. ISBN 978-011079799X.(E)
8	8.3	9661EC		European Community Directive 96/61/EC, Integrated Pollution Prevention and Control, 1996 . (E)
8	8.3	IPPCS506	v4	Guidance for the Recovery and Disposal of Hazardous and Non Hazardous Waste, Sector Guidance Note IPPC S5.06. Issue 4. Environment Agency & Environment and Heritage Service.2004. (E)
SUB-CHAPTER 8.4 – REFERENCES				
8	8.4	REF0003		Process and Information Document for Generic Assessment of Candidate Nuclear Power Plant Designs. The Environment Agency. January 2007. (E)
8	8.4	EATGNM11		Monitoring of radioactive discharges to Atmosphere from Nuclear Facilities, Technical Guidance Note (Monitoring) M11. Environment Agency. 1999. (E)
8	8.4	EATGNM12		Monitoring of radioactive discharges to Water from Nuclear Facilities. Technical Guidance Note (Monitoring) M12. Environment Agency. 1999. (E)
8	8.4	EATGNM18	v1	Monitoring of discharges to water and sewer. Technical Guidance Note (Monitoring) M18. Version 1. Environment Agency. 2004. (E)
8	8.4	20042EuratomC20034832		Commission Recommendation of 18 December 2003 on standardised information on radioactive airborne and liquid discharges into the environment from nuclear power reactors and reprocessing plants in normal operation. C(2003)4832 & (2004/2/Euratom). Official Journal of the European Union. 6 January 2004. (E)
8	8.4	REF0222		Decision Document and Authorisations for future regulation of disposals of radioactive waste under the Radioactive Substances. Act 1993 at British Energy generation Limited's nuclear sites. Environment Agency. 2006. (E)
8	8.4	EATGNM1	v4	Sampling requirements for monitoring stack emissions to air from industrial installations. Technical Guidance Note (Monitoring) M1. Version 4. Environment Agency. 2006. (E)
8	8.4	EATGNM2	v4	Monitoring of stack emissions to air. Technical Guidance Note (Monitoring) M2. Version 4.2. Environment Agency. 2007. (E)
8	8.4	IPPCS506	v4	Guidance for the Recovery and Disposal of Hazardous and Non Hazardous Waste, Sector Guidance Note IPPC S5.06. Issue 4. Environment Agency & Environment and Heritage Service. 2004. (E)
8	8.4	ISBN0100105412937		The Radioactive Substances Act 1993. HM Stationery Office. ISBN 0-10: 0105412937. (E)

8	8.4	ISBN9780110996219		Pollution Prevention and Control Regulations 2000. SI No. 1973. The Stationery Office Ltd. ISBN 978-011099621-9. (E)
CHAPTER 9				
SUB-CHAPTER 9.1 – REFERENCES				
9	9.1	REF0003		Process and Information Document for Generic Assessment of Candidate Nuclear Power Plant Designs. The Environment Agency. January 2007. (E)
9	9.1	SC030162SR1		B Lambers, R W Allott and J G Titley. Initial Radiological Assessment Methodology – Part 1 User Report. Science Report: SC030162/SR1. Environment Agency. May 2006. (E)
9	9.1	EAHGNH1		Integrated Pollution Prevention and Control (IPPC) – Environmental Assessment and Appraisal of BAT. Version 6. Environment Agency. July 2003. (E)
SUB-CHAPTER 9.2 – REFERENCES				
9	9.2	WFD2009		The River Basin Districts Surface Water and Groundwater Classification (Water Framework Directive) (England and Wales) Direction 2009. The Stationery Office Ltd. (E)
9	9.2	WFD2010		The River Basin Districts Typology, Standards and Groundwater threshold values (Water Framework Directive) (England and Wales) Direction 2010. The Stationery Office Ltd. (E)
SUB-CHAPTER 9.3 – REFERENCES				
9	9.3	SC030162SR1		B Lambers, R W Allott and J G Titley. Initial Radiological Assessment Methodology – Part 1 User Report. Science Report: SC030162/SR1. Environment Agency. May 2006. (E)
9	9.3	SC030162SR2		B Lambers B and M C Thorne. Initial Radiological Assessment Methodology – Part 2 Methods and Input Data. Science Report: SC030162/SR2. Environment Agency, May 2006. (E)
9	9.3	EUR17791EN / NRPBSR296		A Mayall, T Cabianca, C Attwood, C A Fayers, J G Smith, J Penfold, D Steadman, G Martin, T P Morris and J R Simmonds. PC CREAM Installing and using the PC system for assessing the radioalogical impact of routine releases. EUR 17791 EN, NRPB-SR296. National Radiological Protection Board. 1997. (E)
9	9.3	EUR15760EN		J R Simmonds, G Lawson and A Mayall. Methodology for Assessing the Radiological Consequences of Routine Releases of Radionuclides to the Environment. EUR 15760 EN. Radiation Protection 72. European Commission. 1995. (E)
9	9.3	NRPBW54		J G Smith, P Bedwell, C Walsh and S M Haywood. A Methodology for Assessing Doses from Short-Term Planned Discharges to Atmosphere. NRPB-W54. National Radiological Protection Board. March 2004. (E)
9	9.3	NRPBW36		W B Oatway and S F Mobbs. Methodology for Estimating the Doses to members of the Public from the Future Use of Land Previously Contaminated with Radioactivity. NRPB-W36. National Radiological Protection Board. May 2003. (E)

9	9.3	RSRIG1		Radioactive Substances Regulation, Authorisation of Discharges of Radioactive Waste to the Environment - Principles for the Assessment of Prospective Public Doses. Interim Guidance 1. Environment Agency. December 2002. (E)
9	9.3	F16RCT2004508847		D-ERICA: An Integrated Approach to the assessment and management of environmental risks from ionising radiation – Description of purpose, methodology and application. (E)
SUB-CHAPTER 9.4 – REFERENCES				
9	9.4	EAHGNH1		Integrated Pollution Prevention and Control (IPPC), Environmental Assessment and Appraisal of BAT, Horizontal Guidance Note IPPC H1. Environment Agency & Environment and Heritage Service & Scottish Environment Protection Agency. July 2003. (E)
CHAPTER 10				
SUB-CHAPTER 10.1 - REFERENCES				
10	10.1	REF0003		Process and Information Document for Generic Assessment of Candidate Nuclear Power Plant Designs. The Environment Agency. January 2007. (E)
SUB-CHAPTER 10.3 - REFERENCES				
10	10.3	WFD2009		The River Basin Districts Surface Water and Groundwater Classification (Water Framework Directive) (England and Wales) Direction 2009. The Stationery Office Ltd. (E)
10	10.3	WFD2010		The River Basin Districts Typology, Standards and Groundwater threshold values (Water Framework Directive) (England and Wales) Direction 2010. The Stationery Office Ltd. (E)
SUB-CHAPTER 10.4 - REFERENCES				
10	10.4	REF0013		Wildlife and Countryside Act 1981 (Chapter 69). The UK Statute Law Database. Office of Public Sector Information, London. (E)
10	10.4	ISBN0110457161		The Conservation (Natural Habitats, &c.) Regulations 1994. The UK Statute Law Database. Office of Public Sector Information, London. ISBN 0 11045 716 1. (E)
SUB-CHAPTER 10.5 - REFERENCES				
10	10.5	ISBN0859515133		K R Smith and A L Jones. Generalised Habit Data for Radiological Assessments. NRPB-W41. National Radiological Protection Board. ISBN 0 85951 513 3. May 2003. (E)
10	10.5	ISBN9780080411545		ICRP Publication 66: Human Respiratory Tract Model for Radiological Protection, 66. Annals of the ICRP Volume 24/1-3. International Commission on Radiological Protection. ISBN 978-0-08-041154-5. Elsevier, 1995. (E)

CHAPTER 11				
SUB-CHAPTER 11.1 - REFERENCES				
11	11.1	REF0003		Process and Information Document for Generic Assessment of Candidate Nuclear Power Plant Designs. The Environment Agency. January 2007. (E)
11	11.1	SC030162SR1		B Lambers, R W Allott and J G Titley. Initial Radiological Assessment Methodology – Part 1 User Report. Science Report: SC030162/SR1. Environment Agency. May 2006. (E)
11	11.1	SC030162SR2		B Lambers B and M C Thorne. Initial Radiological Assessment Methodology – Part 2 Methods and Input Data. Science Report: SC030162/SR2. Environment Agency, May 2006. (E)
11	11.1	EUR15760EN		J R Simmonds, G Lawson and A Mayall. Methodology for Assessing the Radiological Consequences of Routine Releases of Radionuclides to the Environment. EUR 15760 EN. Radiation Protection 72. European Commission. 1995. (E)
11	11.1	ICRP88		ICRP publication 88. Doses to the Embryo and Fetus from Intakes of Radionuclides by the Mother. (E)
11	11.1	NRPBW41		K R Smith and A L Jones. Generalised Habit Data for Radiological Assessments. NRPB-W41. National Radiological Protection Board. ISBN 0 85951 513 3. May 2003. (E)
11	11.1	EUR17791EN / NRPBSR296		A Mayall, T Cabianca, C Attwood, C A Fayers, J G Smith, J Penfold, D Steadman, G Martin, T P Morris and J R Simmonds. PC CREAM Installing and using the PC system for assessing the radioalogical impact of routine releases. EUR 17791 EN, NRPB-SR296. National Radiological Protection Board. 1997. (E)
11	11.1	NRPBR91		R H Clarke Chairman of the Working Group. A Model for Short and Medium Range Dispersion of Radionuclides Released to the Atmosphere. NRPB-R91. National Radiological Protection Board. 1979. (E)
11	11.1	ISBN9780080411545		ICRP Publication 66: Human Respiratory Tract Model for Radiological Protection, 66. Annals of the ICRP Volume 24/1-3. International Commission on Radiological Protection. ISBN 978-0-08-041154-5. Elsevier. 1995. (E)
11	11.1	HPARPD001		S J Watson, A L Jones, W B Oatway and J S Hughes. Ionising Radiation Exposure of the UK Population: 2005 Review. HPA-RPD-001. Health Protection Agency, Centre for Radiation Chemical and Environmental Hazards, Radiation Protection Division. 2005. (E)
11	11.1	NRPBW54		J G Smith, P Bedwell, C Walsh and S M Haywood. A Methodology for Assessing Doses from Short-Term Planned Discharges to Atmosphere. NRPB-W54. National Radiological Protection Board. March 2004. (E)
11	11.1	ADMSUG2007	v4	Atmospheric Dispersion Modelling System User Guide Version 4.0. Cambridge Environmental Research Consultants Ltd. June 2007. (E)
11	11.1	NRPBR273		J Brown and J R Simmonds. FARMLAND A Dynamic Model for the Transfer of Radionuclides through Terrestrial Foodchains. NRPB-R273. National Radiological Protection Board.1995. (E)
11	11.1	NRPBW46		P Teal and J Brown. Modelling Approach for the Transfer of Actinides to Fruit Species of Importance in the UK. NRPB-W46. National Radiological Protection Board. 2003. (E)

11	11.1	NRPBW36		W B Oatway and S F Mobbs. Methodology for Estimating the Doses to members of the Public from the Future Use of Land Previously Contaminated with Radioactivity. NRPB-W36. National Radiological Protection Board. May 2003. (E)
11	11.1	RP144		Guidance on the calculation, presentation, and use of collective doses for routine discharges. Radiation Protection 144. European Commission. 2007. (E)
11	11.1	RSRIG1		Radioactive Substances Regulation, Authorisation of Discharges of Radioactive Waste to the Environment - Principles for the Assessment of Prospective Public Doses. Interim Guidance 1. Environment Agency. December 2002. (E)
SUB-CHAPTER 11.2 - REFERENCES				
11	11.2	F16RCT2004508847		D-ERICA : An Integrated Approach to the assessment and management of environmental risks from ionising radiation – Description of purpose, methodology and application. 2007. (E)
11	11.2	REF0003		Process and Information Document for Generic Assessment of Candidate Nuclear Power Plant Designs. The Environment Agency. January 2007. (E)
11	11.2	RDR128		D Copplestone et al. Impact Assessment of Ionising Radiation on Wildlife. R&D Report, N° 128. Environment Agency. 2001. (E)
11	11.2	IAEA422		Sediment Distribution Coefficients and Concentration Factors for Biota in the Marine Environment. Technical Reports Series N° 422. IAEA. Vienna. 2004. (E)
SUB-CHAPTER 11.3 - REFERENCES				
11	11.3	REF0003		Process and Information Document for Generic Assessment of Candidate Nuclear Power Plant Designs. The Environment Agency. January 2007. (E)
11	11.3	EUR17791EN / NRPBSR296		A Mayall, T Cabianca, C Attwood, C A Fayers, J G Smith, J Penfold, D Steadman, G Martin, T P Morris and J R Simmonds. PC CREAM Installing and using the PC system for assessing the radioalogical impact of routine releases. EUR 17791 EN, NRPB-SR296. National Radiological Protection Board. 1997. (E)
11	11.3	NRPBW54		J G Smith, P Bedwell, C Walsh and S M Haywood. A Methodology for Assessing Doses from Short-Term Planned Discharges to Atmosphere. NRPB-W54. National Radiological Protection Board. March 2004. (E)
11	11.3	NRPBW36		W B Oatway and S F Mobbs. Methodology for Estimating the Doses to members of the Public from the Future Use of Land Previously Contaminated with Radioactivity. NRPB-W36. National Radiological Protection Board. May 2003. (E)
11	11.3	RP144		Guidance on the calculation, presentation, and use of collective doses for routine discharges. Radiation Protection 144. European Commission. 2007. (E)
SUB-CHAPTER 11.4 - REFERENCES				
11	11.4	RIFE12		Radioactivity in food and the Environment, 2006. RIFE-12. Centre for Environment, Fisheries and Aquaculture Science (CEFAS). 2006. (E)

11	11.4	ISBN0117522619		Sampling and measurement of radionuclides in the environment. RADRE. ISBN 0117522619. HMSO. 1989. (E)
11	11.4	SZBTHR042		Sizewell 'B' Power Station Environmental Support Group Report. SZB/THR/042. British Energy Generation Ltd. 2005. (E)
11	11.4	NNCLARnet		LARnet. Annual Report, 2005, NNC. (E)
11	11.4	ISBN011751324		Routine measurement of gamma ray air kerma rate in the environment. Technical guidance note M5. HMIP ISBN 011751324. HMSO. 1995. (E)
SUB-CHAPTER 11.5 - REFERENCES				
11	11.5	HPARPD001		S J Watson, A L Jones, W B Oatway and J S Hughes. Ionising Radiation Exposure of the UK Population: 2005 Review. HPA-RPD-001. Health Protection Agency, Centre for Radiation Chemical and Environmental Hazards, Radiation Protection Division. 2005. (E)
11	11.5	RSRIG1		Radioactive Substances Regulation, Authorisation of Discharges of Radioactive Waste to the Environment - Principles for the Assessment of Prospective Public Doses. Interim Guidance 1. Environment Agency. December 2002. (E)
CHAPTER 12				
SUB-CHAPTER 12.1 - REFERENCES				
12	12.1	REF0003		Process and Information Document for Generic Assessment of Candidate Nuclear Power Designs. The Environment Agency. January 2007. (E)
12	12.1	EAHGNH1		Integrated Pollution Prevention and Control (IPPC), Environmental Assessment and Appraisal of BAT, Horizontal Guidance Note IPPC H1. Environment Agency & Environment and Heritage Service & Scottish Environment Protection Agency. July 2003. (E)
12	12.1	DEFRAnoise2005		Updated noise database for prediction of noise on construction and open sites. DEFRA. July 2005. (E)
12	12.1	BS5228		British Standard (BS)5228 : Noise and vibration control on construction and open sites. (E)
SUB-CHAPTER 12.2 - REFERENCES				
12	12.2	REF0003		Process and Information Document for Generic Assessment of Candidate Nuclear Power Designs. The Environment Agency. January 2007. (E)
12	12.2	EAHGNH1		Integrated Pollution Prevention and Control (IPPC), Environmental Assessment and Appraisal of BAT, Horizontal Guidance Note IPPC H1. Environment Agency & Environment and Heritage Service & Scottish Environment Protection Agency. July 2003. (E)
12	12.2	EAHGNH12008		Integrated Pollution Prevention and Control (IPPC), Environmental Assessment and Appraisal of BAT, Horizontal Guidance Note IIPPC H1. Environment Agency & Environment and Heritage Service & Scottish Environment Protection Agency. March 2008. (E)

12	12.2	EIFAC1986		Report of the working group on terminology, format and units of measurement as related to flow-through and recirculation system. European Inland Fisheries Advisory commission. Tech. Pap., 49. 100 pp. EIFAC (1986). (E)
SUB-CHAPTER 12.3 - REFERENCES				
12	12.3	REF0003		Process and Information Document for Generic Assessment of Candidate Nuclear Power Designs. The Environment Agency. January 2007. (E)
SUB-CHAPTER 12.4 - REFERENCES				
12	12.4	REF0003		Process and Information Document for Generic Assessment of Candidate Nuclear Power Plant Designs. The Environment Agency. January 2007. (E)
SUB-CHAPTER 12.5 - REFERENCES				
12	12.5	EAHGNH1		Integrated Pollution Prevention and Control (IPPC), Environmental Assessment and Appraisal of BAT, Horizontal Guidance Note IPPC H1. Environment Agency & Environment and Heritage Service & Scottish Environment Protection Agency. July 2003. (E)
12	12.5	EAHGNH12008		Integrated Pollution Prevention and Control (IPPC), Environmental Assessment and Appraisal of BAT, Horizontal Guidance Note IIPPC H1. Environment Agency & Environment and Heritage Service & Scottish Environment Protection Agency. March 2008. (E)
12	12.5	OSIL1991		Seawater general composition grids used: OSIL seawater standards and Li, YH (1991). Li, 1991. Distribution patterns of the elements in the ocean: a synthesis. Geochim. Cosmochim. Acta 55 : 3223–3240.
SUB-CHAPTER 12.6 - REFERENCES				
12	12.6	EATGNM1	v4	Sampling requirements for monitoring stack emissions to air from industrial installations. Technical Guidance Note (Monitoring) M1. Version 4. Environment Agency. 2006. (E)
12	12.6	EATGNM2	v4.2	Monitoring of stack emissions to air. Technical Guidance Note (Monitoring) M2. Version 4.2. Environment Agency. 2007. (E)
12	12.6	EATGNM8		Environmental Monitoring Strategy- Ambient Air. Technical Guidance Note (Monitoring) M8. Environment Agency. 2000. (E)
12	12.6	EATGNM9		Monitoring Methods for Ambient Air. Technical Guidance Note (Monitoring) M9. Environment Agency. 2000. (E)
12	12.6	CMD7169		The Air Quality Strategy for England, Scotland, Wales and Northern Ireland. DEFRA. 2007. (E)
SUB-CHAPTER 12.7 - REFERENCES				

12	12.7	EAHGNH1		Integrated Pollution Prevention and Control (IPPC), Environmental Assessment and Appraisal of BAT, Horizontal Guidance Note IPPC H1. Environment Agency & Environment and Heritage Service & Scottish Environment Protection Agency. July 2003. (E)
12	12.7	EAHGNH12008		Integrated Pollution Prevention and Control (IPPC), Environmental Assessment and Appraisal of BAT, Horizontal Guidance Note IIPPC H1. Environment Agency & Environment and Heritage Service & Scottish Environment Protection Agency. March 2008. (E)

APPENDIX 7: UK EPR GDA LEVEL 3 DOCUMENTS: SUPPORTING DOCUMENTS

Document title	Document number / identifier	Document revision number
ABOUT DESIGN COMPRESSIVE AND TENSILE STRENGTHS OF CONCRETE		
INCIDENCE OF THE COEFFICIENTS □ CC OR □ CT IN THE DESIGN OF NUCLEAR CIVIL STRUCTURES		
Relation between the ETC-C rules and the results of EDF studies on the behaviour of spent fuel pool liners		
NPP Project 9609. Case for the justification of the mechanical resistance of spent fuel pool liners equipped with «double-angle» anchors. Version 2.	ENRETM040005	A
EPR PROPOSAL with steel liner Detailed Design Verification study on application of the criteria for a containment with steel liner	0 6 D 0 0 3 0 1 0 1 0 C I G C	C1
Justification of the ETC-C shear calculation methodology		
EPR Basic Design Prestressing Consolidation Singular Zones of the Inner Containment	11815-28B03-NT002	C1
EPR Prestressing Design Status and Construction Sequence	ECEIG090998	A
Soil-structure interaction with the CODE_Aster-ProMISS3D interface	U2.06.07-C	9.5
Code_Aster® version 8.7 qualification note	ENGSDS080103	A1
Deconvolution and Soil-Structure Interaction with ProMiss3D software based on seismic recordings on Hualien array		
EPR computer models for NI buildings		
Seismic analysis with consideration of SSI and SFSI on large structures	AFPS2007EN	
Validation of calculation tools used in soil-structure interaction – Homogeneous Soil – Case C1 and C2	30-05 2	B1
Validation of calculation tools used for Soil- Structure Interaction – Case C4 Tricastin – Case C5 Fessenheim	30-05 3	B1
Validation of calculation tools used for Soil-Structure Interaction – Case C7 : homogenous soil, 10m embedment depth with lateral contact – Case C8 : homogenous soil, 10m embedment depth without lateral contact	30-05 5	B1
UK EPR – Inner containment wall – Rupture of cement-grouted tendons – Structural justification and detectability	12 391 439 RP 003	A
Summary of Analyses of Prestressing Relaxation and Cable Tension Monitoring at Blayais and Civaux NPPs	ENGSGC090373	A
Details of the Prestressing System for the UKEPR Inner Containment	ENGSGC090138	B
Response to HSE Queries Concerning Containment Monitoring & Testing	D4171/NT/2009-00340	B
Declinaison du programme de surveillance des etudes detaillees de l'EPR FA3 (niveau 2 et 3) au sein du groupe GU - Phase realisation	ECEIG061114	B
Organisation des Etudes de Genie Civil Et De Leur Surveillance Pour L'ilot Nucleaire	ECEP061425	A

EPR - Consideration of Health & Safety during the design phase	ECN4100201	A1
Inner and Outer Containments List of Anchor plates and embedded parts from Batch1A “common base slab”	EYRC/2007/fr/0078	B1
NUCLEAR ISLAND General raft foundation RB and IS zone – detailing note – Justification of the pre-stressing gallery beneath the raft foundation	10439–NT–28B01-0110	C
Design of the steel liner (without defects) under operating conditions. Containment current zone calculations (dome area)	892 CD 01005	C1
Design of the steel liner under operating conditions. Containment current zone (cylindrical section) calculation note. (elevation – 7.85 m).	892 CD 01006	B1
Reactor building – Containment internals Design of RPS1 wall anchor Reactor coolant pump – Loops 1 & 4	11787 YR1221 NT 28B01 0078	E
Reactor building – Containment internals Design of RPS2 anchor on wall Reactor Coolant Pump – Loops 1 & 4	11787 YR1221 NT 28B01 0079	E
Reactor Building – Containment Internals Verification of steam generator passive anchorages Calculations for anchorages on walls at elevation + 8.80 m	11787 YR1221 NT 28B01 0084	C
EPR – FLAMANVILLE NPP REACTOR BUILDING – DETAILED DESIGN CALCULATIONS OF INNER CONTAINMENT GUSSET REINFORCEMENTS	11815–28B03–NT 017	D
EPR - FLAMANVILLE NPP REACTOR BUILDING – DETAILED DESIGN FOR THE INNER CONTAINMENT WALL DOME REINFORCEMENT	11815-28B03–NT 023	C
EPR – FLAMANVILLE NPP EPR – FLAMANVILLE NPP REINFORCEMENT AROUND THE FUEL BUILDING PERSONNEL AIRLOCK (CENTRELINE AT z = 20.60 M)	11815–28B03–NT 032	E
EPR - FLAMANVILLE NPP – REACTOR BUILDING - DETAILED CALCULATIONS OF THE PRESTRESSED CONCRETE INNER CONTAINMENT WALL - REINFORCEMENT OF EQUIPMENT ACCESS HATCH PLATES ZONE	11815–28B03–NT 036	C
Reactor building – Containment internals Verification of the RPS3 anchor – level +6.425m	11787 YR1221 NT 28B01 0075	E
FUEL BUILDING DETAILED DESIGN OF POOL SLAB AND WALLS	10439 NT 28B01 316	D
Reactor Building – Detail report: Lower section of reactor pit - up to level -2.30 m	11787 YR1221 NT 28B01 0011	E
Reactor Building Containment Internals Foundation Raft Uplift Calculation	11787 YR1221 NT 28b01 0023	D
EPR - FLAMANVILLE NPP REACTOR BUILDING – DETAILED CALCULATIONS OF THE INNER CONTAINMENT REINFORCEMENTS AROUND THE EQUIPMENT HATCH (AXIS AT DIMENSION Z - 23.15 M)	11815–28B03–NT 020	D
Design of the steel liner under operating conditions. Containment current zone (cylindrical section) calculation note	892 CD 01004	B1
Design of polar crane brackets and their anchors	892 CD 01015	F1
EPR - FLAMANVILLE NPP – REACTOR BUILDING - DETAILED DESIGN OF THE PRESTRESSED CONCRETE INNER CONTAINMENT WALL - REINFORCEMENT OF THE POLAR CRANE BRACKETS ZONE	11815–28B03–NT 022	E
EPR - Flamanville NPP REACTOR BUILDING – DETAILED DESIGN OF THE INNER CONTAINMENT WALL INNER CONTAINMENT WALL STRESS ANALYSIS OF INNER CONTAINMENT WALL AND DOME UNDER CONSTRUCTION	11815–28B03–NC 025	C
REACTOR BUILDING ANALYSIS OF INNER CONTAINMENT WALL STRESSES DURING PRE-STRESSING	11815–28B03–NC 027	F

Section II - TOME 1 - VOLUME Z - ANNEXE X3.4S : caractéristiques pour l'acier inoxydable austénitique X2CrNi18-9 ou X2CrNi19-11 hypertrempé (RCC-MX 2007 Extract, Annex X3.4S – Material Properties)		
A3.4S: PROPERTIES GROUP APPLICABLE TO PRODUCTS AND PARTS IN X2CrNi18-9, X2CrNi19-11 SOLUTION ANNEALED AUSTENITIC STAINLESS STEELS (304L) (RCC-MR 2007 Extract, Annex A3.4S)		
SYSTUS: Verification and Physical Validation Synthesis Report	NEER-F DC 10296	B
Concept Design of the MCR for EPR - France specific arrangements	SFL-EZC-00-8016	B
Inner Containment Thermal behaviour of standard area in accidental cases	CNEN/COB 10 157 412 03	B
Response HSE, Action 20.CE.2 : Pool Liner Studies: To confirm criteria for maximum plastic strain in the pool liner	ENGSGC100273	A
VD3 900 – Theme A1 – Approach Used for Verification of Civil Engineering Works – Scope of Approach	ENGSGC030065	A
900 Series Third Ten Year Inspection – Theme A1 – Civil Engineering Works Verification Procedure – Implementation and Results	ENGSGC040362	A1
Methodology for Passive Anchorage Calculation	11787 YR1221 NT 28B99 0004	C
List of design basis floor response spectra: Nuclear Auxiliary Building for the EPR plant series	ENGSDS060114	A1
Civil Engineering Topic: proposition de reponse a l'Action 21.7 et 21.10	ENGSDS100219	
Answer from COYNE et BELLIER to HSE Initial comments on Draft Response to Regulatory Action A3 on Regulatory Observation 17	12391 4312 RP 001-4	A
Answer from COYNE et BELLIER to HSE Initial comments on Draft Response to Regulatory Action A4 Regulatory Observation 17	12 391 4312 RP 001-5	A
Answer to HSE Regulatory Observation RO-UKEPR-17.003	12 680 RP 001-1	D
Answer to HSE Regulatory Observation RO-UKEPR -17.004	12680 RP001-2	C
Enveloped Design Response Spectra for the MCR	EZC/2007/en/0003	B
MCR Internal Force Output	EZC/2008/en/0053	E
MCR SDNF - and Static Model	EZC/2009/en/0001	B
Procedure ENG1.02: classification code for incoming and outgoing Documents in the framework of the EPR project	ECEP040048	B1
SASSI benchmark (comparison) graph: Fonctions de Transfert, Excitation X, Résponse X, Cas C1 sol Homogène Vs=300m/s		
SASSI benchmark (comparison) graph: Spectre de Réponse à 5%, Excitation Z, Réponse Z, Cas C4 Tricastin		
SASSI benchmark (comparison) graph: Spectre de Réponse à 5%, Excitation Z, Réponse Z, Cas C7 Vs=300m/s, enfoncement 10m		
Answer from COYNE et BELLIER to HSE Action 22.2 : EDF/AREVA to demonstrate concrete stress limits and why the analysis is independent of section	12 680 RP 001-16	A
Answer to HSE Regulatory Observation RO-UKEPR -17.005	12 391 4312 RP 001-3	A
Answer from COYNE et BELLIER to HSE Action 23.9 : to provide the software name & validation report for mesh transfer with UK company NDA (associated with COBEF to NASTRAN)	12 680 RP 001-17	A

Eurocode 1 - Actions on structures - Part 1-3: General actions - Snow loads	NF EN 1991-1-3	
Projet EPR UK – Civil Engineering Topic : proposition de réponse à l'Action 21.8	ENGSDS100263	
Batiment combustible, plancher +3.70/Note de calcul d'ancrages des materiels REA 2110/3110 BA	EYRS/2008/FR/0111	B
Justification of the Partial Factor for Prestressing Actions γP	ENGSGC100402	A
ETC-C PART1: DESIGN UPDATE –MAPPING OF CHANGES FROM ETC-C REVISION B TO AFCEN ETC-C	ENGSGC100475	A
UK EPR – ETC-C PART 3– Testing and monitoring - Comparison between 2006 and 2010 Issues	D4171/RAP/2010-00793	A
Answer to HSE Regulatory Observation RO-UKEPR-17 Action 4, Failures of Horizontal Tendons Around the Equipment Hatch	12680 RP 01-37	B
Answer to HSE Regulatory Observation RO-UKEPR-17 Action 5, Detectability Zones for Horizontal Tendon Failures With a Short Re-anchorage Length (4.0 m)	12680 RP 01-36	C
UK EPR Project: Regulatory Observation 17 Actions 6 and 7: Monitoring of Containment	D4171/NT/2010-00670	A
Review of Prestressing Applied in Construction of the OL3 Containment Building	PEES-G/2010/en/1237	A
OECD Nuclear Energy Agency Monthly News Bulletin (January 2010)	NEA/CSNI/R(2009)9	
GSP 241 Methods for Inspection of Nuclear Industry Concrete Structures from Doosan Babcock Energy (February 2009)	TR-08-195	1
UK EPR: GDA Civil Engineering Topic – Cement Grouting for Prestressing	EDT GC 090132	E
UK EPR Project – Civil Engineering Topic: Response to Action 21.6	ENGSDS110050	
Answer from Coyne Bellier and IOSIS to HSE Action 21.9: EDF/AREVA to Check the Effective Mass Participation for Each Building Located on the Common Raft	12 680 RP 01-26	A
Projet EPR UK – Civil Engineering Topic : proposition de réponse à l'Action 21.9	ENGSDS100233	
EDF – CNEN Technical Assistance for the EPR General Design,Assessment Responses to HSE Action 21.9: Check the Effective Mass Participation for Each Building Located on the Common Raft		
Projet EPR UK – Civil Engineering Topic : réponse complémentaire à l'Action 21.9 suite à demande du projet UK: Check the Effective Mass Participation for Each Building Located on the Common Raft	ENGSDS110251	
EPR UK – GDA – Answer to Action 35.2	ECEIG110130	
Mapping of Changes from ETC-C Revision B to AFCEN ETC-C 2010 for Chapters 2.7 – Leaktight Metal Parts on Containment and 2.9 – Metal Liner of Watertight Pools and Tanks	EDESFR110156	A
UK EPR – GDA – Global approach about methodology for consideration of shrinkage and crack limitation	ENGSGC110025	A
UK EPR – GDA – Methodology for Consideration of Shrinkage for EPR Concrete Structures	ENGSGC100426	B
EPR UK - GDA - Verification of Crack Width for EPR Concrete Structures	ENGSGC100428	B
Combination of modal responses: A closed-form formulation for rigid response coefficient		

Answer to HSE Regulatory Observation RO-UKEPR-37 - Ultimate Pressures in EPR Containment - Comparison of Simplified Method (EPRI Method) to Method Based on Statistical Numerical Simulation	12 680 RP 01-41	A
Answer to HSE Regulatory Observation RO-UKEPR-37 Determination of Failure Mode Ultimate Pressures – Comparison of Simplified (EPRI) or Fully Statistical Methods – Special Case of Low Probabilities"	12 680 RP 01-45	B
EPR UK – GDA – Prestressing Tendons Participation in Reinforced Concrete Calculations for the Inner Containment	ENGSGC100416	C
UK EPR GDA – Presentation and Justification of Factors Ψ2 of ETC-C for Variable Actions in Accidental Situations	ENGSGC100394	C
List of P13 Drawings Used for the EPR Civil Works Design	ECEIG101662	A
Analysis of Inner Containment	12680-RP01-39	D
Damping Ratio of the Pre-stressed Concrete Containment Structure	12680-RP01-46	B
Comparison Between Equivalent Static Seismic Analysis of the Pre-stressed Inner Containment and Seismic Spectrum Analysis with Global NI Model	12680-RP01-49	C
Analysis of EPR Inner Containment - GDA/Step4 – Inner Containment Seismic Calculations in Relation with ETC-C Requirements	ENGSGC110030	A
Justification of the Concrete Maximum CompressiveStress Under Bi-axial / Tri-axial Behaviour (Accidental Thermal Conditions for the Inner Containment)	ENGSGC100415	B
Assessment File of the UK Companion Document to AFCEN ETC-C – Part 1	ENGSGS110033	C
Steel and Concrete Partial Safety Factors for EPR Fastening Systems	ENGSGC100395	B
EPR UK – Shear design proposal	ENGSGC100410	D
Determination of the αcc Coefficient Used in the Formula for the Design Value of the Compressive Strength in Eurocode 2	ENGSGC100384	B
PRESTRESSING SYSTEM FOR UK EPR INNER CONTAINMENT	ENGSGC080361	A
New Codification Approach for the Design of NPP Containments with Metallic Liner	ICONE 16 - 48953	
Shear design concept for structural concrete based on strut and tie models		
Local Behaviour of Reinforced Concrete Walls Under Missile Impact		
Test and Calculation of the Local Behaviour of Concrete Structures Under Missile Impact		
Dynamic elasto-plastic analysis of reinforced concrete slabs: finite element computation and validation by experimental tests"		
Recommendations of a Study Group of the New Zealand National Society for Earthquake Engineering		
HOW MANY ACCELEROGRAMS TO USE AND HOW TO DEAL WITH SCATTERING FOR TRANSIENT NON-LINEAR SEISMIC COMPUTATIONS?	SMiRT18-K09-2	
Improved Generation and Application of Artificial Time Histories and PSD Functions		
UK – GDA – Seismic rules for EPR Civil Works – ETC-C vs. EC8		
About clause 6.2.2 – Members not requiring design shear reinforcement and particularly vmin and punching. 2nd part – Answer to TQ-EPR-284		

Rapport d'expertise sur le risque de corrosion des câbles de précontrainte des BR 900 MWe.	PWO6D00OO11383MGCD	3
MISS06 – Validation of MISS3D – Code_Aster Chaining for the dynamic uplift of the foundation in SSI by the soil spring method	V1.10.125-B	8.1
ADLV101 Sloshing modes of a tank filled with water	V8.21.101-A	4
FDNV100 Sloshing in a water tank with deformable elastic walls	V8.03.100-A1	7.3
SDLD25 Weight and spring system with viscous damper	V2.01.025-B1	4
SDLD30 – Spectral seismic response of a 2-mass, 3-spring model	V2.01.030-A	6.4
SDLL130 – Seismic response of a reinforced concrete beam with linear behaviour	V2.02.130-A	7.1
SDLL132 – Eigenmodes for a frame structure	V2.02.132-A	6
SDLL23 Free restrained beam subjected to seismic conditions	V2.02.023-B	7.1
SDLS106 - Modal calculation for a plate using sub-structuring method	V2.03.106-A	6.4
ZZZZ102 – Read accelerogram and oscillator spectrum functions	V1.01.102-A	4
Code_Aster independent validation plan (English version)	A2.03.01-A	4
Quality Sheet for the operating version of Code_Aster: Version 8 – BULL BAS4 and Linux	A0.02.50	9.5
Supplementary qualification actions for Code_Aster	A4.01.03	9.5
CTICM Recommendations for the Design and Construction of Overhead Crane Runways: Construction measures		
CTICM Recommendations for the Design and Construction of Overhead Crane Runways: Loads to be considered and determination of forces		
CTICM Recommendations for the Design and Construction of Overhead Crane Runways: Stresses and verification of stability		
MISS02 – Validation of MISS3D-Code_Aster Chaining for Structure – Soil – Structure Interaction (SSSI) at the Civaux Nuclear Island	v1.10.121	6.3
MISS03 - Validation of MISS3D - Code_Aster Chaining for the FB - PWR1300 P'4	v1.10.122	6.4
MISS04 – Incorporation of Embedment for a Structure in interaction with the Soil	v1.10.123	6.4
MISS05 – Implementation of the Modal Reduction Method in the MISS3D – Code_Aster Chaining	v1.10.124	6.4
Validation file Shallow, rigid and massless foundation impedance cases: Foundations on a layer resting on a half-space	v2.1.001.03	
Validation file Shallow, rigid and massless foundation impedance cases: Foundations on a layer resting on a rigid substratum	v2.1.001.02	
Validation Manual Impedance of embedded, rigid, massless foundations: Foundation in a layer resting on a rigid substratum	v2.1.002.02	
Validation Manual Impedance of embedded, rigid, massless foundations: Foundations in a homogeneous half-space	v2.1.002.01	
Validation of MISS3D-Code_Aster Chaining for the EPR Nuclear Island	v1.10.120	6.3
Validation Report Basic solutions: Green's functions: Displacements	v2.1.003.02	

Validation Report Basic solutions: Green's functions: Stresses	v2.1.003.03	
Validation Report Basic solutions: Homogeneous and layered cases	v2.1.003.04	
Validation Report: Soil-Fluid-Structure Interaction	v2.1.007.01	
Validation file Shallow, rigid and massless foundation impedance cases: Foundations in a homogeneous half-space	v2.1.001.01	
French National Annex to Eurocode 3 Part 1-10	NF EN 1993-1-10/NA	
HSLS01: Thin square plate subjected to a thermal gradient in the thickness	V7.11.001	9.5
SSLS100: Embedded circular plate subjected to an uniform pressure	V3.03.100	9.5
SSLS114: Non-regression test on cylindrical shell	V3.03.114	9.5
SSNP15: Plate in tension-shear-Von Mises (isotropic strain hardening)	V6.03.015	9.5
Extracts from the independent Code_Aster validation summaries supporting the response to TQ-515 EPR UK	ENGSDS090309	
Bâtiment réacteur - Calculs détaillés de l'enceinte interne - complément d'étude des traversées standards	11815 28B03 NT 0046	A
Bâtiment réacteur - Calculs détaillés de l'enceinte interne - Phasage détaillé de mise en tension des câbles	11815 28B03 NT 0040	B
Divisions 1 and 4 - Detailed calculation note - RIS Equipement Level - 5.00m (RIS 1250EX / RIS 2250EX / RIS 3250EX / RIS 4250EX)	10439-NT-28B01-0441	D
Enceinte interne - Analyse de la peau métallique en situations de fonctionnement - Note de calcul autour de l'ouverture de l'accès site [Inner containment wall. Analysis of the steel liner under operation conditions. Design report on the steel liner around the site access opening]	892 CD 01010	B
FA3 EP3 - Definitions of Loads and Load Combinations to be Considered for the EPR DWN Stack	ECEIG080033	
Ilot nucléaire - Modèle d'ensemble 3D - Jeux de données [Nuclear Island - 3D Overall Model - Set of data]	10439-NT-28B01-0115	B
Note d'hypothèses de dimensionnement du génie civil de la tour d'accès de FA3	SFLEYRC0030021	D
NUCLEAR ISLAND - 3D OVERVIEW OF THE REINFORCEMENT OF THE APC SHELL AND THE WALLS OVER THE COMMON FOUNDATION RAFT UNDER THE PLATFORM UNDER STATIC AND DYNAMIC LOAD COMBINATIONS	10439-NT-28B01-0200	D1
Principes généraux de conception des drains et puisards du système RPE de l'EPR	ECEIG061031	A
NUCLEAR ISLAND – 3D OVERVIEW MODEL DETERMINATION AND LOCATION OF EQUIPMENT MASSES AND OPERATING LOADS	10439-NT-28B01-0103	B1
NUCLEAR ISLAND 3D MODEL USED TO DETERMINE SOIL-STRUCTURE INTERACTION PARAMETERS	10439-NT-28B01-0104	D1
NUCLEAR ISLAND 3D MODEL MODAL ANALYSIS – CALCULATION OF SPECTRAL RESPONSES	10439-NT-28B01-0105	D1
Nuclear Island-OVERALL 3D MODEL Results of basic seismic load combinations – displacements – accelerations – reactions	10439-NT-28B01-0106	C1
Nuclear island-3D GENERAL ARRANGEMENT MODEL DESCRIPTION OF BASIC STATIC LOAD CASES - REACTIONS, FORCES, DISPLACEMENTS	10439-NT-28B01-0107	C1
Answer to HSE Technical Query n° 629	12 391 4310 RP 001-1	B

Analysis of the defect-free steel liner under operating conditions Gusset Calculation Report (step and conical section)	892 CD 01007	B1
Design of the (unflawed) steel liner under operating conditions Calculation report on steam line penetration sleeves	892 CD 01011	A1
Design of the steel liner under construction conditions Calculation report on the steel liner of the foundation raft and gusset	892 CD 01016	B1
Design of the steel liner under construction conditions Calculation report on the steel liner of the cylindrical section	892 CD 01017	B1
Analysis of the Steel Liner During Construction Dome Steel Liner Calculation Note	892 CD 01018	B1
FUEL BUILDING - DESCRIPTION OF STATIC LOAD CASES AND COMBINATIONS	10439-NT-28B01-0304	B1
FUEL BUILDING - DESCRIPTION SEISMIC LOAD CASES AND COMBINATIONS	10439-NT-28B01-0305	B1
FUEL BUILDING - ENVELOPE REINFORCEMENT DIAGRAMS FOR BASIC AND ACCIDENT SITUATIONS	10439-NT-28B01-0306	B1
REACTOR BUILDING INTERNAL STRUCTURES CHECKING OF INTERNAL STRUCTURES ANCHORING IN SEISMIC CONDITIONS	11787 YR 1221 NT 28B010038	B1
Reactor Building Impact of application of differential shrinkage on design basis for reactor building internal structures	11787 YR1221 NT 28B01 NT 0039	B1
REACTOR BUILDING – FLAMANVILLE SITE ANALYSIS OF PRE-STRESSING PHASES WITH PARTIALLY COMPLETED DOME	11815–28B03–NC 043	B1
CALCULATION OF LOCAL STIFFNESS OF COMPONENTS OF REACTOR BUILDING INTERNAL STRUCTURES	11787 YR1221 NT 28B01 0031	C1
Fuel Building - DETAILED DESIGN - FLOORS, Level 19.50M/24.20M/25.20M	10439-NT-28B01-0321	D1
REACTOR BUILDING - CONTAINMENT INTERNALS DETAILED REPORT INNER CONTAINMENT SHIELD WALL CYLINDRICAL WALLS BETWEEN -2,30M AND +1.50M	11787 YR 1221 NT 28B010013	E1
FUEL BUILDING - DETAILED 3D MODEL - DESCRIPTION OF SEISMIC MASSES	10439-NT-28B01-0303	B1
EPR - FLAMANVILLE NPP REACTOR BUILDING - DETAILED CALCULATIONS FOR THE PRESTRESSED CONCRETE INNER CONTAINMENT WALL – VERIFICATION OF REINFORCEMENT AROUND HIGH-ENERGY AND SPARE CAPACITY PENETRATIONS	11815–28B03–NT 045	B1
EPR – FLAMANVILLE NPP REACTOR BUILDING INNER CONTAINMENT WALL DETAILED DESIGN CALCULATIONS FOR S, ELEC, HVAC AND TRANSFER TUBE PENETRATIONS – IMPACT OF THERMAL LOAD OF SLEEVE ON WALL REINFORCEMENT	11815–28B03–NT 042	C1
REACTOR BUILDING – CONTAINMENT INTERNALS DESIGN OF REACTOR VESSEL SHOCK ABSORBING LUGS IN CASE OF ACCIDENT	11787 YR1221 NT 28B01 0021	C1
EPR – FLAMANVILLE NPP REACTOR BUILDING INNER CONTAINMENT WALL DETAILED DESIGN STUDIES THERMO-ELASTIC CALCULATION NOTE	11815–28B03–NT 009	C1
EPR – FLAMANVILLE NPP REACTOR BUILDING INNER CONTAINMENT WALL DETAILED DESIGN CALCULATIONS OF PRESTRESSED CONCRETE INNER CONTAINMENT WALL STEADY-STATE THERMAL FIELDS CALCULATION NOTE	11815–28B03–NT 0010	B1
EPR – FLAMANVILLE NPP REACTOR BUILDING INNER CONTAINMENT WALL DETAILED DESIGN CALCULATIONS FOR PRESTRESSED CONCRETE INNER CONTAINMENT WALL ACCIDENT THERMAL FIELD CALCULATION NOTE	11815–28B03–NT 0011	B1
EPR – FLAMANVILLE NPP REACTOR BUILDING INNER CONTAINMENT WALL DETAILED DESIGN CALCULATIONS OF PRESTRESSED CONCRETE INNER CONTAINMENT Results: DISPLACEMENTS AND DEFORMED SHAPES FOR ELEMENTATRY	11815–28B03–NT 012	C1

LOAD CASES		
NUCLEAR ISLAND - 3D OVERALL MODEL SENSITIVITY CALCULATION WITH GROUND/STRUCTURE INTERACTION PARAMETERS VALIDATION OF STATIC AND DYNAMIC LOAD CASES	10439 NT 28B01 0114	A1
CONTAINMENT INTERNALS IN THE REACTOR BUILDING – REFUELLING POOL WALLS AND SLABS BETWEEN LEVELS +6.16 AND +19.5M	11787 YR 1221 NT 28B010024	D1
Answer to HSE Technical Query n° TQ-EPR-711	12 680 RP 001-7	A
Seismic response by transient analysis (extract)	R4.05.01	
EPR project with liner Detailed Design Margin analysis Maximum pressure	11 256 – NC 104	B1
EPR With Liner Project Detailed Design Modelling Report	11 256 – NC 105	A1
FA3 project instruction Surveillance of subcontracted design	SFL-PF-00020	A1
Organisation report on surveillance of civil engineering design for the Flamanville 3 nuclear island	SFL-EYRC-03024	B
Surveillance programme for studies of contract YR 1223	SFLEYRC03026	B1
Improved generation of PSD functions, artificial accelerograms and spectra, fully compatible with a design response spectra	SMIRT-12	
EPR-CNPE DE FLAMANVILLE BATIMENT REACTEUR - CALCULS DETAILLES DE L'ENCEINTE INTERNE PRINCIPE DE CONSTRUCTION DE L'ENCEINTE INTERNE EN PARTIE HAUTE [Reactor building – Inner containment wall detailed design. Principle of construction of the upper part of the inner containment wall]	11815-28B03-NT 039	B
EPR-CNPE DE FLAMANVILLE BATIMENT REACTEUR - ANALYSE DES CONTRAINTES DANS LA PAROI DE L'ENCEINTE INTERNE, A 25 ANS, 45 ANS ET EN FIN DE DUREE D'EXPLOITATION [Reactor building – Analysis of stresses in the inner containment wall at the end of the operating period]	11815-28B03-NT 028	D
EPR-CNPE DE FLAMANVILLE BATIMENT REACTEUR - CALCULS DETAILLES DE L'ENCEINTE INTERNE FERRAILLAGE AUTOUR DES TRAVERSEE DES LIGNES DE VAPEUR ET EAU ALIMENTAIRE [Reactor building – Prestressed concrete inner containment wall detailed design. Reinforcement around the steam line penetrations]	11815-28B03-NT 021	D
EPR-CNPE DE FLAMANVILLE BATIMENT REACTEUR - CALCULS DETAILLES DE L'ENCEINTE INTERNE FERRAILLAGE AUTOUR DU SAS PERSONNEL ET DE L'ACCES CHANTIER [Reactor building – Prestressed concrete inner containment wall detailed design. Reinforcement around personnel airlocks and site access airlock]	11815-28B03-NT 019	F
Etude de la peau metallique en situation de construction Note de calcul de la peau metallique au droit des traversees RIS et EVU [Inner containment wall. Analysis of the steel liner under construction conditions. Design report on the steel liner at the RIS [SIS] and EVU [CHRS] penetrations]	892 CD 01021	B
Etude de la peau metallique etude des fourreaux de traversees en situation de fonctionnement Note de calcul du fourreau de la traversee de sas (ID = 3,3 m)	892 CD 01024	C

Etude de la peau metallique etude des fourreaux de traversees en situation de fonctionnement Note de calcul du fourreau de la traversee de l'acces site	892 CD 01025	C
Etude de la peau metallique etude des fourreaux de traversees RIS et EVU situees en partie basse de l'enceinte (diametre 711) [Inner containment wall. Analysis of the steel liner. Design report on the RIS [SIS] and EVU [CHRS] penetration sleeves located at the bottom of the containment wall (Ø 711)]	892 CD 01027	C
Etude de la peau metallique etude des fourreaux des traversees de diametres 273 a 711, de type <<Standard>>, <<Haute Energie>> et <<Reserve>> (hors RIS et EVU entre Z= -7,02 et -6,93m) [Inner containment wall. Design report on standard, high-energy and standby penetration sleeves of diameter 273 to 711 mm (excluding RIS [SIS] and EVU [CHRS] between Z = - 7.02 and – 6.93 m)]	892 CD 01028	D
Etude de la peau metallique Note de calcul des puisards des traversees RIS et EVU	892 CD 01029	C
Etude relative aux tuyauteries RIS et EVU Etude de la mise en place des ensembles fourreaux - puisards et de leur comportement en phase de betonnage	892 CD 01030	B
Etude relative aux tuyauteries RIS et EVU Dimensionnement des appuis support des fourreaux et des couvercles de protection des puisards (phase de construction de l'enceinte)	892 CD 01031	B
Etude de la peau metallique Calcul des platines pre-scellees. Dimensionnement des platines du liner. [Analysis of the steel liner. Pre-embedded plate design. Liner plate dimensioning]	892 CD 01034	D
Dimensionnement des pieces pre-scelles Determination des efforts admissibles des fourreaux points fixes	892 CD 01035	B
Calcul des platines pre-scellees. Dimensionnement des platines rectangulaires (<<Flat bars>>)	892 CD 01036	B
Etude relative au recuperateur de corium Dimensionnement des ancrages du canal de decharge et des plaques de jonction avec la chambre d'etalement	892 CD 01037	B
Etude relative au recuperateur de corium Calcul du canal de decharge en phases du manutention et de construction	892 CD 01038	B
Etude relative au recuperateur de corium Dimensionnement des ancrages du caniveau central de distribuiton d'eau de la zone d'etalement	892 CD 01039	B
Etude de la peau metallique Principes de conception et de dimensionnement d'une platine elargie dans la partie cylindrique du fut. Note de calculs analytique	892 CD 01040	B
Etude de la peau metallique Platine elargie dans la partie cylindrique du fut: Note de calculs aux elements finis	892 CD 01041	A
Etude de la peau metallique Etude des platines d'ancrage du Tampon acces materiel [Analysis of the steel liner. Analysis of the equipment hatch anchor plates]	892 CD 01043	C
Etude de la peau metallique en situations de construction: Calculs completementaires sur le fut compte-tenu de la presence de singularities (zones a concentration de platines)	892 CD 01044	A
Etude de la peau metallique en situations de construction: Note de calcul de la peau metallique du fut compte-tenu de la presence de singularities et de chargement specifique du Constructeur [Analysis of the steel liner under construction conditions. Cylindrical part steel liner design report taking account of singularities and specific contractor loads]	892 CD 01046	C
Methodology for consideration of shrinkage for EPR concrete structures	ENGSGC080086	B1

Etude du comportement de l'enceinte interne EPR au-dela des conditions de dimensionnement - Note de cas de charges et combinaisons'	11854 431 NC 002	E
Etude du comportement de l'enceinte interne EPR au-dela des conditions de dimensionnement - Calculs des contraintes dans les sections courantes	11854 431 NC 003	D
Etude du comportement de l'enceinte interne EPR au-dela des conditions de dimensionnement - Calculs des contraintes locales autour du sas material	11854 431 NC 004	D
Etude du comportement de l'enceinte interne EPR au-dela des conditions de dimensionnement - Etude des pressions de rupture de l'enceinte interne	11854 431 NC 005	C
Etude du comportement de l'enceinte interne EPR au-dela des conditions de dimensionnement - Calcul non lineaire de l'enceinte interne - Cas de charges et results principaux efforts	11854 431 NC 008	E
Etude du comportement de l'enceinte interne EPR au-dela des conditions de dimensionnement - Calcul no lineaire de lenceinte interne - Displacement du bord interieur de la virole	11854 431 NC 009	C
Etude du comportement de l'enceinte interne EPR au-dela des conditions de dimensionnement - Calcul non lineaire de l'enceinte interne - Deformations de la peau metallique	11854 431 NC 010	C
Answer to HSE Technical Query n° - TQ-710 - "Coyne et Bellier's Level of Verification"	12 680 RP 001-09	A
Generating additional accelerograms for the EPR project	ENGSDS070134	
EPR – Review of design reports on areas of discontinuity on the inner containment wall of the Reactor Building	ECEIG091195	
Répartition des contraintes dans les armatures actives et passives d'une poutre en béton partiellement précontraint par post-tension, avec câbles injectés au coulis de ciment. Modélisation, évaluation réglementaire	4471-pp-71-83	
Cracking of pre-tensioned structures - 3rd fib International Congress		
Answer to HSE Technical Query No - TQ 847 - "Calculation of slab reinforcement in SAB"	12 680 RP 001-10	A
Answer to HSE Technical Query n° - TQ- 854 - "Computer Software COBEF"	12 680 RP 001-08	A
Pre-design report of TEP 3501 ZE Bolt Anchorage	EZC/2008/en/0036	B
Seismic motion response of a building on an elastic soil. A. DELEUZE G.		
Use of generalised interfaces degrees of freedom in component mode synthesis. E. BALMES		
Structure-Soil-Structure Interaction calculation for the Civaux site with implementation of the Code_Aster-Pro_MISS3D Interface. G. DEVESA	HP-52/98/027/A	
EPR Basic Design Dynamic Analyses - modelling and modal analysis of NI buildings. C. DUVAL	ENT-MS/96/052/A	
Influence of Structure-Soil-Structure-Interaction at the Civaux NPP. A. GOBERT	ENT-MS/93/038	A
Présentation et exemples d'utilisation de CLASSI : Code de calcul d'analyse des effets de l'interaction sol – structure sur la réponse sismique des bâtiments. Recueil de spectres plancher des bâtiments de l'ilot nucléaire. G. QUILTON	E SE TA 82-01 SG	

Mise en oeuvre de l'interface Code_Aster Miss3D pour la modélisation du comportement de la maquette Hualien sous des sollicitations harmoniques et transitoires pour les configurations FVT1 et FVT2. A. Devesa G.	HP-52/96/024A	
Answer to HSE Technical Query n° 1013 - TQ Title: Bursting forces during tendon reanchorage	12 680 RP 01-15	A
Answer from COYNE et BELLIER to HSE Technical Query n°1017 – TQ Title: Containment Compression	12 680 RP 01-23	A
Summary report of characterization test for anchors on EPR steel liner	ENGSGC060331	A1
Internal Containment leaktightness liner and penetration sleeves. Finite Element calculations. Presentation of SYSTUS software	300 NT 10 009	A
Internal Containment leaktightness liner and penetration sleeves Presentation of facilities (internal software) used for processing of finite element calculations data and results	300 NT 10 010	A
ZZZZ200 - RIGI_PARASOL and RIGI_MISS_3D Test	V1.01.200	A
Assessment of predictive calculations and results of tests conducted on the PCCV SANDIA 2 model	ENGSGC030097	A1
Probability Safety Assessment of Typical 900 MWE French Nuclear Containment	SMIRT 14 Lyon GRANGER	
Behaviour study of EPR equipment hatch beyond design-basis conditions	ENGSGC070167	B1
Analysis of the steel liner with defect under operating conditions – Calculation and analysis note on buckling between connectors and on a failed mesh	892 CD 01014	C1
Transient Time History Analysis and Optimization of the Dynamic Behaviour of a Command Room Isolated on A-Seismic Bearing Pads		
EPR – MCR Technical Review – Seismic behaviour	ECEIG060426	
HFERCOQ: HERCULE POST-PROCESSOR CALCULATION OF SHEAR FORCE REINFORCEMENT IN PLATE OR SHELL ELEMENTS: Identification	SM-IS296-01	2.0
HFERCOQ: HERCULE POST-PROCESSOR CALCULATION OF SHEAR FORCE REINFORCEMENT IN PLATE OR SHELL ELEMENTS: User Guide	SM-IS296-03	2.0
ASTHER: ASTHERAST ASTHERASTM: Validation Test	SM-IS298-04	1.0
HERAST: HERASTORI: Validation Test	SM-IS297-04	2.0
Batiment reacteur - structures internes. Note de details. Dimensionnement des plots de beton sacrificiel vis-à-vis de la chute de la cuve. Note de modelisation	11787 YR1221 NT28B01 0042	D
Batiment reacteur - structures internes. Note de details. Etude de comportement lors de la chute de la cuve sur les plots de beton sacrificiel.	11787 YR1221 NT28B0 0044	C
Steel liner design: Calculation note in operating situations Influence of equipment hatch anchor plates on the behaviour of the neighbouring l	892 CD 01048	A1
REACTOR BUILDING – DETAILED DESIGN OF THE INNER CONTAINMENT WALL PRESENTATION OF RESULTS - "SERIES" ABSOLUTE DISPLACEMENTS	11815-28B03-NT 034	B1
Contract YR 1223. Analysis of the defect-free steel liner in operating conditions. Calculation note of an area of the liner around penetrations with diameters of 273 to 1016 mm (in the cylindrical part)	892 CD 01012	B1
Contract YR1223. Analysis of the steel liner. Analysis of impact of anchor plate on the behaviour of the steel liner in normal situations and severe accident situations.	892 CD 01022	B1

Steel liner design: Calculation note in operating situations Influence of circular plates on the behaviour of the liner according to the length of the ligament	892 CD 01045	B1
Steel liner design: Operating situations calculation note Influence of circular plates on the behaviour of the liner according to the width of the ligament Calculations on axisymmetric mass models	892 CD 01047	B1
Contract YR 1223. Analysis of the steel liner. Hypothesis Note for calculation of 273mm to 1422mm diameter penetration sleeves	892 CD 01026	C1
Steel liner design: Calculation note for electrical, HVAC, transfer tube, steam and feedwater penetration sleeves	892 CD 01032	C1
Steel liner design: Calculation note in operating situations Influence on the liner of reinforcements on the circular plates in the cylindrical part (reinforcements proposed by the main contractor)	892 CD 01042	C1
REACTOR BUILDING – DETAILED DESIGN OF THE INNER CONTAINMENT WALL DOME RING REINFORCEMENT	11815-28B03-NT 024	B1
ASTHER ASTHERAST ASTHERASTM Identification	SMIS29801	2.0
ASTHER ASTHERAST ASTHERASTM Overview	SMIS29802	1.0
ASTHER ASTHERAST ASTHERASTM User Guide	SMIS29803	1.0
HERAST-HERASTORI Identification	SMIS29701	2.0
HERAST-HERASTORI Overview	SMIS29702	2.0
HERAST-HERASTORI User Guide	SMIS29703	2.0
SIGNSOLL Identification	SMIS29901	1.0
SIGNSOLL Overview	SMIS29902	1.0
SIGNSOLL User Guide	SMIS29903	1.0
SIGNSOLL Validation Tests	SMIS29904	1.0
Calculation of strain concentrations in the liner induced by a concrete crack		
Effect of a concrete crack. Liner analysis in the gusset zone and around major singularities		
Design of the (unflawed) steel liner under operating conditions Calculation note in the area around the equipment hatch	892 CD 01008	B1
Analysis of the Steel Liner during construction Calculation note for the liner in the equipment hatch reinforced area	892 CD 01019	B1
Steel liner design Design of penetration sleeves in operating conditions Calculation note for the equipment hatch penetration sleeve	892 CD 01023	C1
TQ1031 Appendices		
MODELE DE L'ILOT NUCLEAIRE EPR DE FLAMANVILLE 2006 - DIVISION 1 [Excel spreadsheet used to convert COBEF data to HERCULE data]		
EPR project with liner Detailed Design Hypothesis note Load cases and combinations	11 256 - NC 101	C1
EPR – Flamanville NPP Reactor Building – Inner Containment Wall Detailed Design – Verification Of Inner Containment Wall Gusset Concrete Construction Joints at -8.00 m and -6.39 m	11815–28B03–NT 058	B

Manuel Qualite: Chapitre 1 Developpement du Logiciel HERCULE Introduction Generale [Quality Manual of HERCULE providing elements regarding the development team and the Quality Assurance procedure]		
Test de validation - Index des tests [List of the tests performed for the qualification of HERCULE]		
Development and approval of software	PRO 09.11	A1
Software Quality Assurance file	PRO 09.12	A1
REACTOR BUILDING - CONTAINMENT INTERNALS DETAILED REPORT FOUNDATION RAFT AND WALLS IN SPREADING AREA TO LEVEL -2.30M	11787 YR1221 NT 28B01 0010	C1
EPR - FLAMANVILLE NPP REACTOR BUILDING - DETAILED DESIGN OF THE INNER CONTAINMENT WALL BUTTRESS REINFORCEMENT	11815–28B03–NT 018	D1
Fuel Building detail design floor levels -3.95m/-3.40m/-2.30m	10439-NT-28B01-312	D1
Fuel Building detail design floors at levels 0.00m/1.50m	10439-NT-28B01-313	D1
Fuel Building detail design superstructure walls from 0.00m to +19.60m excluding spent fuel pit walls	10439-NT-28B01-314	D1
Fuel Building detailed design of floors at levels +3.70m/+4.05m/+4.45m/5.15m	10439-NT-28B01-315	C1
Fuel Building detailed design - level +3.70 REA [RBWMS] tanks 2110/3110 BA	10439-NT-28B01-336	E1
Study of the behaviour of the inner containment of the EPR beyond design-basis conditions – Non-linear modelling of the inner containment wall	11854 431 NC 007	D1
Study of the behaviour of the inner containment of the EPR beyond design-basis conditions – Thermal calculation note	11854 431 NC 001	D1
Reactor Building - Containment internals detail design note upper part of reactor pit between +1.50m and +7.44m	11787 – YR1221 NT – 28B01 0020	C1
Steel liner design. Design of embedded plates. Calculation methodology	892 CD 01033	B1
NUCLEAR ISLAND BUILDINGS METHODOLOGY FOR THE TREATMENT OF SHRINKAGE TYPE IMPOSED STRAIN	11787 YR1221 NT 28 B 99 003	C1
EPR UK – Answer to HSE Technical Query n° TQ-EPR-1334 – Bursting forces during re-anchorage	12 680 RP01-48	A
Analysis of the (unflawed) steel liner under operating conditions - Design report in the area around personnel and emergency airlock openings	892 CD 01009	B1
HFERCOQ: HERCULE POST-PROCESSOR CALCULATION OF SHEAR FORCE REINFORCEMENT IN PLATE OR SHELL ELEMENTS: Validation	SM-IS296-04	2.0 B
HFERCOQ: HERCULE POST-PROCESSOR CALCULATION OF SHEAR FORCE REINFORCEMENT IN PLATE OR SHELL ELEMENTS: Overview	SM-IS296-02	2.0 B
ETC-C Part 2: Construction Update – Mapping of Changes from ETC-C Revision B to AFCEN 2010 ETC-C (Sections 2.2 to 2.5, 2.11, and 2.12)	EDTGC110381	A
ETC-C Part 2.10 – Mapping of Changes from ETC-C Revision B to AFCEN ETC-C 2010	ETDOIG/110305	A
UK EPR – Analysis of Inner Containment – Justification for use of Linear Elastic Methods for Design of the Inner Containment of the UK EPR	10420/R-01	02
Description and Justification of Analysis Methods Used in the Design of the UK EPR Inner Containment	ENGSGC110026	A
UK EPR – GDA – Justification of AFCEN ETC-C Construction Joint Design Method	ENGSGC110222	A

UK EPR – GDA – Presentation and Justification of the Consequences of Concrete Cracking on Liner Leak-tightness	ENGSGC110046	B
THERMAL AND MECHANICAL BEHAVIOUR OF AN EXPERIMENTAL MOCK-UP OF A NUCLEAR CONTAINMENT		
Minimal reinforcement		
UK EPR - JUSTIFICATION OF ETC-C SHRINKAGE RULES	ENGSGC110345	A
CTICM: Structural Steel Design – Expert Opinion		
Hypothesis note on inner containment wall equipped with steel liner inside reactor building	SFL-EYRC-0030018	E1
Hypothesis note for the design of civil engineering structures in the FA3 Fuel Building	SFL-EYRC-00030017	F1
Fuel Building - Assumptions and methodology note	10439-NT-28B01-301	E1
SAB Div.2 and 3 Hypotheses note	10439 NT 28B01 0401	D
Nuclear Auxiliary Building - Note of hypotheses and methodology	11788 YR1222 NT 28B01 0001	D
General Assumptions for the Design of the FA3 EPR Common Foundation Raft	SFL-EYRC-0030027	C1
YR2201 – Hypothesis Note for the Bouygues design of the DWN [NABVS] stack on the Fuel Building roof	EYRC/2009/fr/0088	C1
EPR – Flamanville 3 – General Hypothesis Notes for Turbine Hall Design	ETDOIG070090	B1
AFCEN ETC-C – 2010 Edition: ETC-C EPR TECHNICAL CODE FOR CIVIL WORKS		
EPR – Flamanville Nuclear Power Plant Reactor Building – Inner Containment Wall Detailed Studies	11815 28B03 NT 005	C
Nuclear island - 3D Overall model - Hypothesis and methodology note	10439-NT-28B01-0101	F1
Reactor Building-Containment internals modelling note	11787 YR1221 NT 28B01 0002	C1
Answer from COYNE et BELLIER to HSE Action 23.14 : HSE to send list of key 'inner' structural elements (e.g. 3 x SAB, 3 x NAB, 3 x FB & 4 x Inner Containment [i.e. Polar Crane] Answer from COYNE et BELLIER to HSE Action 23.15 : EDF/AREVA response to 23.14 will provide a route map to where the supporting information can be found.	12 680 RP 01-31	B
EPR FA3 - Diesel Building - Hypothesis and Methodology Report	10439 NT 28B01 501	D1
UK EPR - GDA Assessment file of the UK companion document to AFCEN 2010 ETC-C (Section 2.2 to 2.5 ENGSGC110015 D)	EDTGC120392	A
Assessment file for the UK Companion Document of the AFCEN ETC-C 2010 (Parts 1.7 & 2.10)	ETDOIG/110422	B
UK EPR - GDA - Assessment File of the EPR Nuclear Island Civil Engineering Design process note	ECEIG121031	A
Assessment File of Revision E of UK Companion Document to the AFCEN ETC-C 2010	ENGSGC120228	A
Behaviour of the inner containment shell during severe accident	ECEIG/99 34 (TR 99/96)	B
EPR - Technology of the penetrations through the containment	ECECP/98 053 (TR 98/78)	A
Synthesis Report on Preliminary Design and Optimisation Studies for the EPR Reactor Building Containment	ENSGC/95 03 B (TR 95/20)	A

EPR - FLAMANVILLE NPP REACTOR BUILDING – DETAILED DESIGN CALCULATIONS FOR INTERNAL CONTAINMENT RESULTS PRESENTATION FILE: LOADS AND MOMENTS	11815–28B03–NT 013	B1
REACTOR BUILDING DETAILED DESIGN CALCULATIONS FOR THE PRESTRESSED CONCRETE INNER CONTAINMENT WALL - CALCULATION OF DISPLACEMENTS AT INTERFACES	11815–28B03–NT 014	B1
EPR – FLAMANVILLE NPP REACTOR BUILDING – DETAILED DESIGN CALCULATIONS OF INNER CONTAINMENT WALL CYLINDRICAL PART TYPICAL AREA REINFORCEMENT	11815–28B03–NT 016	B1
UK EPR - R.O. 43 (Action 2 – Mechanical aspects) : Justification of the adequacy of engineering standards with supplements associated to M3 requirements	ENSNDR100104	A
Safety Justification for the Classification of Class 1 Components with M3 Mechanical Design Requirements	PEPS-F DC 99	A
Analysis of a Class 1 requirment for the PTR [FPPS/FPCS] startup feature of the main cooling trains	ECECS120406	A
Main plant parameters analysis		
UK EPR – GDA EPR design and mechanical requirements - Classification and requirements applied to mechanical pressure retaining components	ENSNDR120069	B
UK EPR GDA Project: Summary of Responses to Actions in ONR Final Report on Lessons Learned From the Fukushima Event	PTS DC 9	B
UK EPR - Specification for System Design Manuals Update post GDA	UKEPR-0019-001	01
UK EPR - Handover document for GDA design changes	UKEPR-0020-001	01
Electrical Protection for the Generator and Transformers System Manual: Document2 - System Operation	ETDOFC080364	B1
D.C. SWITCHBOARDS SUPPLYING TURBINE GENERATOR SET EMERGENCY AUXILIARIES	74.C.033.02	
Rectifiers-battery chargers for nuclear power plants	76.C.001.03.01	
CONVERTERS (INVERTERS AND UNINTERRUPTIBLE POWER SUPPLIES) FOR NUCLEAR POWER PLANTS	81.C.004.03	
CONVERTERS (INVERTERS AND UNINTERRUPTIBLE POWER SUPPLIES) FOR NUCLEAR POWER PLANTS	81.C.004.03.01	
Fast Transient within one safety division Robustness		
CAE Approach Report	17074-709-000-RPT-0001	03
UK EPR - Long-term Analysis	ENFCFF110026	A
Impact Assessment of Simultaneous Connection of All Remaining Loads During Stable LOOP Operation	ECEEL120851	B
DDG Optioneering and feasibility study	ECEEL120814	A
Quantitative and Qualitative Overview of Non-Classified Emergency Loads	ECEEL120873	A
Engineering Drawings of APG7103-7113FI	CN64711D00	E
Engineering Drawings of APG8240-8280FI	CN64724D00	B
Engineering Drawings of PTR4522FI and PTR4542FI	CN65409D00	E
Engineering Drawings of PTR4526FI	CN65414D00	B

Engineering Drawings of PTR5221FI	CN65408D00	B
Engineering Drawings of TEP2301FI	CN65415D00	B
Engineering Drawings of TEU3201-7141FI	CN64733D00	B
Engineering Drawings of TEU4201-11FI	CN64854D00	B
Engineering Drawings of TEU4226FI	CN64938D00	B
Engineering Drawings of TEU8201FI	CN64974D00	B
Equipment datasheet, coolant filters, RCV213FI – RCV215FI	NEER-G/2007/en/1136	B
Schéma Mécanique Fonctionnel, Système APG	ECEF081522	A
EPR – Flamanville 3 Effluent Treatment Building floor drawings (P10): -3.90m	EYRC/2006/FR/0229	D
EPR – Flamanville 3 Effluent Treatment Building floor drawings (P10): Level +3.70m	EYRC/2006/FR/0231	C
EPR – Flamanville 3 Effluent Treatment Building floor drawings (P10): Level +7.40m	EYRC/2006/FR/0232	C
EPR – Flamanville 3 Effluent Treatment Building floor drawings (P10): Level 0.00m	EYRC/2006/FR/0230	C
EPR – Flamanville 3 Effluent Treatment Building floor drawings (P10): Level -7.50m	EYRC/2006/FR/0228	D
EPR – Flamanville 3 Effluent Treatment Building floor drawings (P10): Levels +11.00m & +15.00m	EYRC/2006/FR/0233	D
EPR: Activités volumiques dans les circuits auxiliaires TEP, REA, TEG, RPE, TEU et TES	ENTERP070070	A
Manufacturing drawings of TEU1106BA floor drain tank	HE6EP.14.06.1750	D
Manufacturing drawings of TEU1116BA floor drain tank	HE6EP.14.16.1770	A
Manufacturing drawings of TEU1206BA process drain tank	HE6EP.11.06.1450	D
Manufacturing drawings of TEU1216BA process drain tank	HE6EP.11.16.1470	D
Manufacturing drawings of TEU1306BA chemical drain tank	HE6EP.12.06.1550	D
Manufacturing drawings of TEU1316BA chemical drain tank	HE6EP.12.16.1570	A
Inner Containment Seismic Fragility (rock site)	0841301.01-C-001	0
Reactor Building Concrete Internal Seismic Fragility (rock and soil sites)	0841301.01-C-002	0
Seismic Fragility of Electrical Equipment Qualified to HN 20-E-53 (rock site)	0841301.01-C-005	0
Seismic Fragility of Electrical Equipment Qualified to HN 20-E-53 (soil site)	0841301.01-C-006	0
Seismic Fragility of Teleperm XS I&C Panels (rock site)	0841301.01-C-007	0
Seismic Fragility of Teleperm XS I&C Panels (soil site)	0841301.01-C-008	0
Seismic Fragility of Generic Piping and Pipe Supports (rock and soil sites)	0841301.01-C-009	0

Seismic Fragility of Generic Horizontal Pumps (rock site)	0841301.01-C-010	0
Seismic Fragility of Generic Horizontal Pumps (soil site)	0841301.01-C-011	0
Generic Anchorage Seismic Fragility (rock and soil sites)	0841301.01-C-012	0
Internal Structure Structural Response Factor (rock site)	0841301.01-C-013	0
Internal Structure Structural Response Factor (soil site)	0841301.01-C-014	0
DIESEL BUILDING – DESCRIPTION OF EARTHQUAKE LOAD CASES AND COMBINATIONS	10439 NT 28B01 505	B
DIESEL BUILDING–- DESCRIPTION OF THE STATIC AND DYNAMIC LOAD CASES AND COMBINATIONS	10439 NT 28B01 504	C
Nuclear Auxiliary Building - Note of presentation of 3D model load cases	11788 YR1222 NT 28B01 0003	B
Reactor Building – Containment Internals – Overall Model Design Calculation Note – Description of Load Combinations - Results	11787-YR1221-NT-28B01-0003	D
EPR - FLAMANVILLE NUCLEAR POWER PLANT REACTOR BUILDING DETAILED DESIGN. DESIGN HYPOTHESIS NOTE FOR INNER CONTAINMENT WALL	11815 28B03 NT 003	D1
Reactor Building Inner Containment Wall – Detailed Design Studies – Definition of Basic Static Load Cases	11815 28B03 NT 007	C
Primary Equipment support hydraulic snubber – Sizing Calculation Note for DA12	QRI059686	B
EPR™ FA3 – Loop Analyses Final Loads Applied to Primary Components Supports and Pipes	NEERFDC265	B
EPR – Coherence analysis of the 3D models used for civil engineering seismic studies	ENGSDS070098 (TR07/435)	B1
Presentation of approach for incorporating hazards in the EPR Project	ENSN040070 (TR04/165)	A1
EPR FA3-RCS dimensioning loads applied by RCS supports to the civil works	NEER-F DC 89 (TR 07/399)	C
Fire During Post-Accident Phase	KWU NA-T/1998/E035 (TR98/64)	B
Internal Hazards Caused by Earthquake	KWU NA-T/1998/E070 (TR98/79)	B
Internal Hazards During Shutdown	KWU NA-T/1998/E037 (TR98/65)	A
Synthesis of basic design studies on EPR containment with steel liner	ECEIG/03 1132 (TR 03/134)	A
Nuclear island - Overall 3D model - General Description - nodes, elements and thicknesses	10439 NT 28B01 0102 (TR07/373)	C1
Fuel Building detailed 3D model - General description nodes elements	10439-NT-28B01-0302 (TR 07/378)	C1
Steel Liner Design – Definition and Description of Design Hypothesis	892 CD 01001 (TR 07/373)	C1
Steel Liner Design Modelling note for load cases and boundary conditions	892 CD 01003 (TR 07/373)	B1
Detailed design of inner containment leaktight steel liner, Steel liner design, General introductory note on modelling principles	892 CD 01002 (TR 07/373)	B1
Reactor Building - Preliminary design - Consolidation of EPR detailed design of prestressing of inner containment wall span	11815-28B03-NT001 (TR07/372)	C1
Accidents in shutdown conditions	DNM 02033 (TR 97/51)	E

Breaks on RHR system outside containment - Synthesis of long term assessment	NFPE 05.211 (TR 05/244)	
Cathare 2A-LB LOCA realistic evaluation model	EPTB DC 1502 (TR 98/71)	B
Criticality risk in case of control rod withdrawal - UOx and MOX (FRENCH)	ENPRNC080045 (TR 08/584)	A
ECC methodology-PCC 3 and 4	EPTA DC 1466 (TR 97/53)	D
Large break LOCA (2A) design guidelines	DNM 01867 (TR 96/50)	C
LOCA 2A: methodology for pin failure estimate	NEPD-F DC 52 (TR 07/363)	A
LOCA 45 cm2 combined with loss of LHSI - A state (FRENCH)	NFPSR DC 1105 (TR 05/231)	B
Methodology for P&T calculation using CONPATE4 code	NFPSR DC 1079 (TR 05/249)	B
Mitigation of Small Break LOCA with failure of the Protection System	neprf 08.1347 (TR 04/142)	
Operating domains - Identification of their limits (FRENCH)	ENFCRI050029 (TR08/521)	C
RRA breaks: definition of automatic actuation for safety injection and shutdown of non affected RRA trains (FRENCH)	NEPR-F DC 320 (TR 08/569)	A
Science V2 qualification: suitability for EPR FA3 (FRENCH)	NEPDF DC 65 (TR 07/431)	D
Technical Report on Feed & Bleed	NFPSR DC 1123 (TR 06/268)	B
Pressurizer: Lower Dome - Details	NEER-F DB 1224	E
RCPs modelling		
Routine of Main VVP Lines Inside Reactor Building	NESPFE DB 1006	C
Routine of Main VVP Lines Outside Reactor Building	NESPFE DB 1007	C
EPR UK- Detailed burn-up distribution	nepcf.09.1365	
Answer to TQ527 – Appendix to letter ND(NII) EPR00235N		
PCI TECHNOLOGICAL LIMIT FOR FUEL RODS WITH M5® CLADDING	FS1-0000675	1
Answer to TQ-EPR-568 - Radial Form Factor in CHF calculations	nepcf.10.0185	
Enclosed comparison plots		
UK EPR – TQ-EPR-605 QUERY 2 – NEUTRON SOURCES	D-FDW-10-00239	
C R G A COLUMN	NEER-F DB 1282	E
UPPER SUPPORT PLATE DEATIL DRAWING	NEER-F DB 1311	C
UPPER CORE PLATE DEATIL DRAWING	NEER-F DB 1312	D
NORMAL COLUMN	NEER-F DB 1314	C
LEVEL MEASUREMENT PROBE COLUMN (LMP COLUMN°	NEER-F DB 1315	F

RPV Internals: Typical Slab	NEER-F DB 1299	G
RPV Internals: Upper Slab	NEER-F DB 1300	G
RPV Internals: Slab N°II	NEER-F DB 1301	G
DESCRIPTION OF THE SCIENCE V2 PHYSICAL MODELS	NFPS-D DC 87	A
VALIDATION OF SCIENCE V2 CALCULATIONS FOR EPR CORE MODEL WITH HEAVY REFLECTOR BY COMPARISON WITH MCNP4	NFPS-D DC 153	A
UK EPRTM GDA - Answer to TQ 667	pepcf.10.0462	
Answer to TQ667 - Gadolinium inventories & reflector modelling methodologies - effective delayed neutron fraction and reactivity coefficients calculation	PEPDF10_0146	
Generalized Statistical DNBR method (MSG) Topical Report	EPD DC 291	A
Technical description of FLICA III-F V 2.5.1 Justification for choice of basic models	EPTCDC1470	A
Star-CD version 3.20 Qualification Synthesis Report	NEPD-F DC 10049	A
PKL III Tests on Heterogeneous Boron Dilution following SB-LOCA (Cold Leg Break / Cold Leg SI) - Applicability to Reactor Scale	NTCTP-G/2008/en/0003	C
International Standard Problem ISP-47 on Containment Thermal-hydraulics	NEA/CSNI/R(2007)10	
Response to HSE, Action 12.FS.2: Effect of anchorage design on steel liner behaviour	ENGSGC100255	A
Steam line break: break spectrum at 0% nominal power	PEPR-F DC 18	A
Steam Line Break: Break Spectrum at Power (4500MWth)	PEPR-F DC 17	A
UK GDA - Transmission of UK EPR™ steam generator steady state conditions at full power – heat flux in SG	PEPRF-10.1405	
EPR™ UK - GDA - Modification of the Average Coolant Temperature (ACT) LCO at Low Power level (below 25%) and the PZR level LCO at state A	PEPR-F.10.1701	
Monitoring Reactor coolant / secondary side leaks in PWRs	D4550.15-11/0406	01
Q1 – SIS sizing process		
Q2 – plots for IB LOCA		
Procedure MIP EPR N°ENG 2-01 Functional Identification (IG1)	ECECI030651	C1
CATHARE 2 V2.5_1 : Description of the base revision 6.1 physical laws	CEA SSTH-LDAS-EM-2005-038	0
UK EPR™ - GDA - Calculations to answer Technical Query n°683	PEPR-F 10.0574	
List of PAS / SAS functions diverse from TXS		
Justification of the internal events list for the design conditions of the NSSS	Fratec 01	E
Appendix 6 to Sec. 4 of the Incident Calculation Bases: Method for the Performance of Probabilistic Calculations		
Evaluation of dispersion using ADMS v.4 for accidental radiological consequences assessments	15395/TR/0002	4

Lower support plate – RPV Internals – Core Barrel Assembly	NEER-F DB 1307	H
FLICA III F-V 2.5.1 QUALIFICATION REPORT	EPTC DC 1469	A
Physical Models of Manta Code	NFEPD DC 50	C
MANTA MIXING MATRICES FOR THE EPR BASED ON JULIETTE TESTS INTERPRETATION	NEPD-F DC 74	C
Qualification of MANTA Code on a Transient of a spurious opening of a MSRT Valve on Paluel 3 Reactor	PEPR-F DC 13	A
UK EPR – Answer to the TQ-EPR-1302 : Uncertainties in DNBR design limits for low pressure and high quality conditions.	PEPDF10_401	
Multi-purpose description of ventilation systems contributing to containment in normal operating conditions	ECEF080078	A1
Multi-purpose description of ventilation systems contributing to containment in accident operating conditions	ECEF080346	A1
FC2002 - CHF CORRELATION AFA-2G-AFA-3G-FUEL	NEPD-F DC 10306	A
LBLOCA spreading compartment condensate data		
Batiment reacteur - structures internes note de details etude locale de comportement lors de la chute de la cuve sur un plot de beton sacrifieciel	117878 YR1221 NT 28B01 0070	C
Central Supply Duct – single part drawing	NEER-G-100571	A
Analyse de la tenue mécanique des plaques entretoises et de l'interaction tube plaque en situation accidentelle – Analysis of support plate spacers plastic & elastic stress analyses, mechanical behaviour and tube / plate interactions under accident and seismic conditions.	NEEG-F DC 902	C
Steam Generator internals dimensioning report – feedwater rings – AVB's supporting system	NEEG-F DC 36	C
Exploitation des résultats d'essais d'éclatement des tubes des GV - SG tube bundles break test analysis	TEG DC 0572	A
Review of PWR Steam Generator Tubing Materials selection, performance and manufacturing routes	PMSCC.09.041	
GDA UK - Demonstrate the provision of diverse protection against loss of CVCS following a normal reactor trip and xenon decay including demonstration of diversity to operator action	peprf.11.0956	
UK EPR GDA – GDAI-FS02 - ATWS by loss of TXS –RCCA misalignment up to Rod drop	pepcf.11.1467	A
GDA UK - Demonstrate the provision of diverse protection against loss of CVCS following a normal reactor trip and xenon decay including demonstration of diversity to operator action	peprf.11.0956	1
UK EPR - FS 03 - Faliure Modes and Effects Analysis for the Spent Fuel Mast	ECESN120111	A
Identification of single and common modes of failure for the electrical systems for the UK EPR issue FS-05	ENFCFI120092	A
UK GDA – Answer to TQ 1539 related to loss of CVCS faults	PEPR-F 12. 0139	
Dilution tests on Juliette mock-up description of facility and tests grids	PEPD-F DC 17	B
Probabilistic assessment of the initating events relative to the loss of DVL and DEL trains in the frame of the GDA issue GI-EPRUK-FS-05	ECEN120408	A
UK EPR FS03 – Design Basis Analysis of faults associated to the spent fuel pool safety case	ECESN120587	A

[EPR UK GDA] – TQ 1567 – Spurious opening of a Pressuriser Safety Valve, in case of loss of computerized I&C	peprf12.0966	
UK GDA - Response to Question 3 of TQ-EPR-1593	PEPRF 12.1220	
GDA issue FS05 – TQ 1621 – Partial answer to TQ	peprf12.1371	
Active cooling of the core catcher with the containment heat removal system	NEPSG/2007/en/1001 (TR 07/482)	A
Containment heat removal system	MN/S-93-2661 (TR 94/13)	
Experimental demonstration of the heat removing capability of the EPR core catcher, considering the impact of boron and insulation material	NGTT1/2005/en/1001 TR 05/221	A
FA3 - EPR licensing France - Severe accident - Sensitivity study on H2 production and AICC pressure in the containment	NFPSD DC 1012 (TR 05/233)	B
FA3 – Severe accidents – Justification of the vessel support behaviour during the temporary corium retention in the reactor pit	NEPS-F DC 132 (TR 07/434)	B
Fission product deposition in the containment vertical walls, ceilings and floors in severe accidents	NGPS4/2006/en/1012 (TR 06/273)	C
H2 source term	EPDA DC 0075 (TR 98/74)	A
Hydrogen in vessel source term	EPD DC 306 (TR 99/112)	B
Justification of the choice of sacrificial concrete in the reactor pit	NGPS4/2005/en/1054 (TR 05/243)	B
Melt water interaction	EPD DC 203 (TR 99/88)	A
Practical elimination of core melt sequences with containment by-pass Status of the EPR Project at end 2004	ECEF041013 (TR 05/172)	A
Severe Accident relevant Instrumentation	NEPRG/2007/en/1009 (TR 08/538)	B
Severe accidents - improved spreading concept	DNM 01469 (TR 96/31)	A
Thickness and concrete type of the sacrificial layer in the core catcher	NGPS4/2005/en/1037 (TR 05/220)	A
EPR Project - R&D for severe accidents	ECEF031269 (TR 03/131)	E
Paper of principle on the validation of the core Melt Stabilization function		
Proceedings of the International Youth Nuclear Congress 2008	IYNC 2008 Paper No. 189	
Presentation on Hydrogen Analyses in the EPR presented at the International Youth Nuclear Congress 2008		
Conference on Concrete Behaviour under Severe Accident Conditions		
Computational validation method for the combustible gas control system of the EPR containment	PEPA-G/2010/en/1015	A
COCOSYS: Short Description		
Stability of cavity pit structural concrete	ECEP072031	
Re-assessment of Melt Spreading in the EPR Core Catcher		

EPR reference framework for studies on filtration of the debris upstream of the recirculation pumps	ENGSIN050001	D1
FA3 Reactor Pressure Vessel Cavity Seal Ring	NEER-F DB 1321	E
List of plant conditions categories - Justification of relevant events	ENSN010010761 (TR 01/118)	A
Core damage extent analysis during doubleended break LOCA for FA3 EPR reactor"	NEPD-F DC 159	A
Cathare V2.5-1 User's manual	SSTH/LDAS/EM/2005-035	0
EPR™ FA3 – Elaboration of the reference CATHARE input deck – Internal use	NEPR-F DC 417	A
Geometry files and CFD modelling of the case where the two spacer grids are in contact	Data_for_contact_case	
Geometry files and CFD modelling of the case where the two spacer grids are in nominal deposition	Data_for_nominal_gap_zie_case	
EPR™ GDA – explanations related to the data provided to HSE for 100cm² IB-LOCA	PEPD-F 10_0151	
Used-up RCCAs characteristics for NDA disposability assessment	NEPCF.09.1014	A
Description and Qualification of the COPERNIC / TRANSURANUS code (Update May 2000) – Fuel Rod Design Code	TFJC DC 1556	A
COPERNIC V2.4 – Addendum to Qualification Report	FF DC 0461	A
Enhanced corrosion of 1-cycle M5 HTP fuel rods - summary of root cause analysis results	A1C-1338315-0	
EPR FA3 - Fuel Assembly and Main Components – List of drawings and Specifications	FS1-0001578	1.0
Design and Conception Rules for Fuel Assemblies of Nuclear Power Plants of Nuclear Island –RCC-C		
Modelling of Heavy Reflector – follow up	PEPD-F.10.0198	
Delayed Hydride Cracking : Response to TQ1184	HT25-C2010 - 86/PBF	
Identification and Substantiation of Key Claims on Operator Reliability in the UK EPR PSA Level 1	NEPS-F/10.173	
Analysis of local maintenance and operating activities – Selection and disciplines involved	ECEPEP040070	
Identification and Substantiation of Key Claims on Operator Reliability in the UK EPR PSA Level 2	NEPS-F/10.273	A
EDF/AREVA GDA Task Analysis Method Statement	16474/TR/0001	01
EDF/AREVA GDA Task Analysis of Example Claim 1: Start Up of the Station Blackout Diesel Generators following a Loss of Offsite Power	16474/TR/0002	2.0
Target Audience Description	D 4550 15 10 5225	
Guide de Rédaction des Consignes CIA EPR [Incidental and Accidental Operations Procedures Writing Guide	ECEF090533	A
Contrôle des accès dans les locaux de sauvegarde de l'EPR Cahier des charges fonctionel du système KKX [Functional Specification for the KKX system, Access Control for EPR Safeguard Buildings Rooms]	D4002.92-06/CN003	0
Niveaux d'eclairement dans les locaux industiels de l'EPR [Lighting levels in EPR industrial rooms]	CDH-MDU-SMaRT 2006-0020	
Prescriptions Technologiques pour l'éclairage de l'EPR [Technological requirements for EPR lighting]	ECEIG060161	A

Cahier Des Charges Du Système DNX "Distribution des prises de courant" [Specification for the DNX System, Electrical plugs Distribution]	ECEIG061188	A
Cahier des charges du systeme RRI (Systeme de refroidissement intermediare) Specifications for the CCWS (Component Cooling Water System)	ECEF0100620	A
Cahier des charges du système ASG EPR Specifications for the Emergency Feed-water System, EFWS]	ECEF0101031	B
Extra Boration System (EBS [RBS]) System Specification	ITSR DC 177	E
SYNTHESE DES RESULTATS DES ESSAIS FH CONDUITE CONDUITE SUR LES THEMES DOCUMENTATION ET IHM MENES SUR LE SIMULATEUR PHASE 2 [Synthesis of the results of HF tests carried-out on Phase 2 Simulator on the themes of Documentation and HMI]	ECEP101809	A
Report on recorded differences between Operator Workstation Panel HMI on Plant Unit and Operator Workstation Panel HMI on Phase 2 Simulator	ECECC090517	B1
EPR Graphic Objects Library	ECEF050861	F1
EDF/AREVA GDA Task Analysis: Feed and Bleed Recovery Strategies [OP_BLEED_120MN] & [OP_BLEED_30MN]	16895-707-000-RPT-0001	E
EDF/AREVA GDA Human Factors: PCSR Sub-Chapter 18.1 Revised Structure	16895-707-000-RPT-0018	B
Task Analysis (Human HAZOP) Programme for Type A/B Human Failure Events Modelled in the PSA	17163-707-000-RPT-0003	F
EPR Control Room Layout - Programme Document	04088-300-DE004	A1
EPR UK GDA - Human Factors - Time estimate for transfer to NCSS	ECUK121139	A
Teleperm XS Based I&C System Quality Plan	NLE-F DC 113	C
UK NII/HSE Safety Assessment Principles comparison with EURs	ENSN070068	B
Fire protection for EPR technical buildings – Main practical measures in design	ECEIG050560	C1
Analysis Network Qualification Part 1: SAS Network	DN2.2.09	2
Simplified Failure Modes and effects analysis STT1 (FMEA)	NLTCG2008EN1011	B
SL22 and SLM2 Failure Modes (FMEA)	NLTHG2008EN1001	C
Software tests	FAW TXS - 4.1	A
Scalance F1B	DN2.2.12	2
Operating Principles and Failure Modes of the SAA1 Analog Signal Conditioning Module (FMEA)	NLTCG2008EN1002	C
Justification of diversity between the SPPA-T2000 / S5 (F2 automation part) and TELEPERM XS (F1A part)	ECECC050092	D1
Justification of non-interference with Standard Instrumentation and Control F1B functions (Subject of Standing Group P/A No. 7 of the 1st December 2005)	ECECC071061	A1
TXS Hardware Modules with Complex Components	NLTD-G/2009/en/0181	A
Justification of the C&I Compact Model for the EPR PSA	H-T52-2009-01596-EN	1

Protection System detailed specification file	NLE-F DC 38	F
Engineering Procurement ENG 2-28: Design of automatic logic processing	ECEF050477	A1
Probabilistic justification of Non Computerised Safety System (NCSS)	NEPS-F DC 192	A
Protection System, Severe Accident I&C, Reactor Control Surveillance and Limitation System V&V and Test Plan	NLE-F DC 222	E
T2000 System FAT - Test Report Performance and Others Functions	NLF-F DC 287	A
Design Report Predictability Model	QU628	7.0
Approval Program for standard system features	NLF-F DC 80	E
Design rules for software basic engineering	NLF-F DC 81	C
AnalysisNetwork Qualification Part 3: F2 - Terminal Bus	DN 2.2.19	2.0
Type test procedure SPPA T2000	QU639	2
IEC 61513 and 62138 justification for PICS	DN2.2.23	2
FA3 Standard I&C Changes and configuration management	NLF-F DC 154	D
Principles for the functional testing of the standard EPR FA3 Instrumentation and Control System programmed data	ECEF072211	A1
Process Instrumentation Pre-processing system detailed specification	NLE-F DC 173	C
RIC Specification file	NLE-F DC 27	D
Rod Position Instrumentation (RPI) System	NLLP-G/2006/en/1001	D
Safety Information and Control System (SICS) Analysis of technical data	WR2.5.02	1
Cartography OM 690 development documentations	DN 2.2.13	5.0
P/A 1 Justification qualification F2 for OM690 and terminal bus. Action 8 - justification pre-developed software (COTS) in OM690	DN 2.2.22	2.0
F1B Qualification. Cartography of Development Documentation	DN 2.2.11	3.0
Justification of Non-Perturbation for OM690 and the Terminal Bus by OM690-XU, ES680, DS670 or ES685	DN 2.2.21	3.0
Switch Modules : Scalance X408-2 Justification for F2-Qualification	DN 2.2.15	4.0
Switch Modules : Scalance X307-3 Justification for F2-Qualification	DN 2.2.16	4.0
Definition of the Hardware configuration for Seismic tests and EMC tests	QU629	10.0
Seismic test procedure SPPA T2000	QU630	3.0
Seismic test report SPPA T2000	QU631	2.0
EMC test procedure SPPA T2000	QU637	3.0
EMC Test Report SPPA-T2000	QU638	3.0

Reliability analysis SPPA-T2000	QU627	12.0
Electronic Modules dependability analysis	QU626	9.0
Type Test Report SPPA T2000	QU640	2.0
PS, SA I&C and RPI - Level Test Specification T-5	NLN-F DC 116	B
Typical System Quality Plan for the Engineering of a TELEPERM XS I&C System	NLE-F DM 10007	D
Production excellence and independent confidence building for EPR UK safety C&I	ENSECC090137	B
Network Qualification - Justification of Non-Perturbation of SAS, PAS, Plant Bus and Island Bus by Lower Classified Components	DN 2.2.25	3.0
Summary Qualification Report: Qualification of the Power Supply Modules SPS1 (24V) - 6FK5542-8AA, SPS2 (5V) - 6FK5543-8AA, SPS3 (12V) - 6FK5544-8AA	NLTC-G/2007/en/0044	A
YR4101 – Analysis of PS source code V1 (TXS) – Supplement to monitoring programme EDESEL070143 A	EDESFR101459	A1
Objectives of monitoring operations performed by SEPTEN to validate 1E- or F1A-class software	ENSECC060060	A1
Content of monitoring operations performed by SEPTEN to validate 1E- or F1A-class software	ENSECC060061	A1
TELEPERM XS User manuals: Engineering System SPACE (TXS Core Software 3.4.x)	TXS-2100-76	4.0
Istec Certificate for the Digital Safety Instrumentation and Control System TELEPERM XS Software: Environment (RTE) V2.7.2	TXS-RTE-1109-10	
TELEPERM XS ENGINEERING PROCEDURE METHODS AND RULES FOR ENGINEERING OF TELEPERM XS APPLICATION SOFTWARE WITH SPACE	NFLE DM 4	D
FAT Organization application data test platform	NLE-F DC 282	A
Detailed Hardware Design Documentation of RPR PIPS Cabinet 3RPR4001AR	NLDH-G/2009/en/1195	B
TXS plant-spec - Qualification synthesis evaluation report	NLTQ-G/2009/en/1018	G
Plans du MCS	ECEF082964	E
OASIS Concept and Tools	NFLS DC 165	C
OASIS Safety Principles	NFLS DC 168	C
Justification Note for NCSS Platform Selection	PTI DC 5	A
Justification of compliance with safety requirements governing organisation of the different types of safeguard functions within the reactor protection system (PS) and interfaces between this system and safeguard actuators	ECECC070578 (TR07/393)	C1
CCE Project : Common Cause Failure Analysis of FA3 I&C architecture	H-P1A-2007-02803-FR (TR07/479)	1.0
AREVA NP PLANTS SECTOR INTEGRATED MANAGEMENT MANUAL	QM DC 55	J
Compliance of the TXS Engineering process with IEC60880 Ed.2	NLTC-G/2007/en/0017	A
Istec Test Certificate for the Digital Safety Instrumentation and Control System TELEPERM XS Software	TXS-DRV2GEN-0707-02	

Configuration Management Plan for the TELEPERM XS System Platform	FAW TXS-1.5	D
Programming Guidelines	FAW TXS-2.1	E
Reviews	FAW TXS-4.2	C
Software Requirement Specifications	FAW TXS-6.3	A
Software Design Descriptions	FAW TXS-6.4	A
Software Implementation Descriptions	FAW TXS-6.6	A
P/A 2.2: Non-Perturbation of SAS - SAS-RCC-B By Lower Classified Equipments	DN 2.5.04	V2.0 INF
Summary Qualification Report for the TELEPERM XS modules SAI1, SAO1 and SGPIO1	NLTC-G/2007/en/0029	B
Softwareprüfplan für sicherheitsrelevante Produkte [Software test and verification plan for safety relevant products]	ISTec–A–1068	00
Istec Certificate for the Digital Safety Instrumentation and Control System TELEPERM XS Software: Program Structure of FD/FDG	TXS-FPG-0306-07	
Istec Certificate for the Digital Safety Instrumentation and Control System TELEPERM XS Software: FDG Code Generator v2.8.0	TXS-FDGCG-0808-10	
Istec Certificate for the Digital Safety Instrumentation and Control System TELEPERM XS Software: RTE Code Generator v2.8.0	TXS-RTECG-0808-10	
Istec Certificate for the Digital Safety Instrumentation and Control System TELEPERM XS Software: Operating system MICROS, V1.04 dated 01.08.2001	TXS-MIC-1201-05	
Istec Test Certificate for the Digital Safety Instrumentation and Control System TELEPERM XS Software: MicroNET version 2.1.0	TXS-MNET-0209-1	
TELEPERM XS Service Letter, Component version ES07 of modules SVE2, SCP2 and SCP3 qualified		
TELEPERM XS Product Information 2009/01: New Release 3.5.0 of the 'TXS CORE Software' for LINUX		
Engineering procedure ENG2-26: functional allocation rules for I&C processing; I&C, electrical sources and flud supply systems loss and restoration management rules	ECEF070582	A1
FA3 standard instrumentation & control system qualification synthesis evaluation report	PELL-F DC 52	A
Outline of Content of Basis of Safety Case for the Protection System Operator Terminal	ECECC111181	A
UK EPR SMART Devices – Trial Applications	ECECC111184	B
UK EPR Protection System – Scope and Programme of Work to Address Functional Static Analysis and Compiler Validation	ENSECC110123	A
Outline of Basis of Safety Case of Non-Computerized Safety System	PEL-F/11.0309	A
Protection System Failure Mode and Effect Analysis - System Level	NLN-F DC 83	D
Justification of PS reliability	PELL-F DC 233	A
Diversity Implementation plan for Sensors & Conditioning	PELA-F DC 3	A
UK EPR Outline BoSC - Qualification Document - Selftest coverage analysis	Ev1-Key Cl 3b	0

Outline of Basis of Safety Case for the SPPA-T2000 based I&C systems (SAS, PAS, SAS RRC-B, PICS and Plant Bus) and SPPA-T2000 platform	PEL-F/11.0353	
UK EPR Protection System - Overall scope of Independent Confidence Building Measures	ENSECC110173	A
UK EPR GDA – TATS Action GI 4-I&C-1 - EPR overall I&C design process	PELA-F 12.1004	B
Progress Report on Class 2 Smart Instrument Qualification Trial	ECECC120227	A
General approach to Failure Rate calculation and FMEA of TXS-modules	PTLSC-G/2012/en/0010	A
Qualification of the standard-signal multiplier module SNV1-2.5 6FK5250-8AA01 ES02 and SNV1-10 6FK5250-8AA02 ES01	NLTC-G/2007-en/0051	A
Qualification of the Binary Signal Conditioning module SBC1 6FK5326-8AA00	NLTC-G/2007/en/0032	C
SNV1 Failure mode and effects analysis (FMEA)	NLTC-G/2008/en/0043	D
Failure modes and effects analysis (FMEA) for SBC1	NLTC-G/2008/en/0059	D
Ausfallraten-Prognose	EK31/4.529A	D
Ausfallratenberechnung zur Baugruppe Binärsignalaufbereitung SBC1 6FK5326- 8AA00	NLTD-G/2006/de/0174	A
Field Failure Rate Calculation and Statistics of TELEPERM XS; Status 2011-06-30	PTLD-G/2011/en/0302	A
SVE2 Failure mode, failure effects and failure detection (FMEA)	NLTC-G/2008/en/0039	D
SAI1 Failure mode and effect analysis (FMEA)	NLTC-G/2008/en/0056	F
SDIx Failure mode and effect analysis (FMEA)	NLTC-G/2008/en/0049	F
SGPIO1 Failure mode and effect analysis (FMEA)	NLTC-G/2008/en/0062	D
SAO1 Failure mode and effect analysis (FMEA)	NLTC-G/2008/en/0058	F
SDO1 Failure mode (FMEA)	NLTC-G/2008/en/0006	E
SL22 and SLM2 failure modes (FMEA)	NLTC-G/2007/en/0071	C
SDM1 Failure modes and effect analysis (FMEA)	NLTC-G/2008/en/0014	B
SOBx-y Failure modes and effect analysis (FMEA)	NLTC-G/2008/en/0008	B
Subrack with power supply, fans and backplane - failure mode and effect analysis (FMEA)	NLTC-G/2008/en/0054	C
PS Reliability, Availability and Maintenance Studies	NEPS-F DC 29	G
NCSS System Verification and Validation Plan	TA-2065953	B
Proposal for Research Programme on Simulation-Based Statistical Testing	ECECC111572	B
Feasibility Study into the use of MALPAS for UK EPR	5094205-rep-01	3.0

Feasibility Study into Compiler Validation for Teleperm XS	5098073-rep-02	4.0
RS/PTL Organisation Note for I&C Platforms Diversity Management	PTL-F DC 4	A
Design of the NCSS system - Principles of selection of actuator orders and information for operators	ECECC100555	B
PSOT System Requirements Specification – Feasibility Study	ECECC110951	A
QDS Operation Principles	NLS-F DC 10143	B
Software Identification File - Document A	QU004A	0.2
System specification file (DSS)	QU014	0.1
Equipment Identification File	QU021	0.2
DX1000 Emphasis assessment database	ECECC121338	A
Emphasis license agreement		
Yokogawa DX1000 Operational Experience Report	ECECC120781	A
GDA – EPR UK – Report of the audit held onthe 15th and 16th of February, 2012, in Yokogawa's premises, in Amersfoort (Netherlands), concerning the software development of the recorder DX1000	EDESFR120956	A
Seismic qualification report for Yokogawa DX Advances series recorders	TR90725-06N	2
Electromagnetic Interference (EMI) qualification report for Yokogawa DX Advances series recorders	TR90725-06N-1	2
Test report for Software/Firmware Validation of Yokogawa DX Advances series recorders	TR90725-06N-2	2
Engineering Assessment Report for the substantiation of the Yokogawa Daqstation DX 1000/200 for use in Safety applications	RP_DES-CAP_SYST_00377	A
CINIF EMPHASIS Phase 2 DXA_Daqstation YHQ R2		
CINIF EMPHASIS Phase 3 DXA_Daqstation YHQ R2		
Technical specification for Enhanced life sign for EPR Flamanville FA3	DE 3020	4.0
SAS application for Enhanced life sign OM690 for EPR Flamanville FA3	DN2-5-10	3.0
QDS System Configuration Management Plan	NFLS DC 186	E
Progress Report on Class 1 Smart Device Trial Assessment	ECECC121403	A
Definition of I&C architecture design requirements in the UK context	ECECC120414	A
Exclusion of CCF between SPPA T2000(S7) and TELEPERM XS by using diversity	NLTC-G/2009/en/0018	B
TATS Action GI 14-I&C-1 related to TQ-EPR-1607 : Spurious actuation challenging category A functions	ECECC121715	A
Current Diversity Analysis between SPPA-T2000(S7) and TELEPERM XS – Corrective action plan	PTI12.1071	A
Key Elements for Diversity Management Methodology Improvement	PTI12.1072	A

QDS Software Quality Assurance Plan	NLS-F DC 10067	B
QDS System Software Development Rules, Recommendations and Guidelines	NFLS DC 119	C
QDS Software Verification and Validation Plan	NFLS DC 177	E
[EPR UK] – GDA TQ 1567 – Comparison of the NCSS functions and SAS diversified	peprf12.1062	1
UK GDA - Response to TQ-EPR-1624 : Elements of protection system – Primary/Secondary (P/S) related	peprf.12.1121	
UK GDA - Allocation of sensors 1 conditioning when 3 lines of defence are involved	PEPS-F DC 148	A
Fire Protection Reactor Building plan view level +1.50m	SFL-EYRL-11-031211 (TR07/441)	
Fire Protection Reactor Building plan view level +13.80m	SFL-EYRL-11-031214 (TR07/441)	
Fire Protection Reactor Building plan view level +19.50m	SFL-EYRL-11-031215 (TR07/441)	
Fire Protection Reactor Building plan view level +24.10m	SFL-EYRL-11-0312016 (TR07/441)	
Fire Protection Reactor Building plan view level +28.50m	SFL-EYRL-11-031217 (TR07/441)	
Fire Protection Reactor Building plan view level +5.15m	SFL-EYRL-11-031212 (TR07/441)	
Fire Protection Reactor Building plan view level +8.70m	SFL-EYRL-11-031213 (TR07/441)	
Fire Protection Reactor Building plan view level -2.30m	SFL-EYRL-11-031210 (TR07/441)	
Fire Protection Reactor Building plan view level –6.15m	SFL-EYRL-11-031209 (TR07/441)	
Input data for identifying the EPR rooms and areas requiring fire protection by fixed spray header systems by fixed spray header systems	ECEIG060418 (TR 07/440)	B1
Avoidance of PCC-3 and 4 events as a consequence of independent internal hazards	NA-T/1998/EO46 (TR 98/67)	A
Incorporation of Feedback Experience into fire studies for EPR sites in France	ECEIG060536 (TR 07/494)	B1
Test specification for electrical cable raceway protection systems	ENGSIN040526	A
Test specification for fire break cases	ENGSIN040476	A
Systems at risk of internal explosion – EPR	ECEF071213	A1
Sectorisation Incendie Batiment Electrique et des Auxiliaires des Sauvégarde (HLF) NIV, -9.60 M [Fire Protection Drawing Safeguard Building HLF (Div 1) Niv, -9.60 M]	SFLEZL00006888	F
Sectorisation Incendie Batiment Electrique et des Auxiliaires des Sauvégarde (HLF) NIV, -5.00 M [Fire Protection Drawing Safeguard Building HLF (Div 1) Niv, -5.00 M]	SFLEZL00006889	F
Sectorisation Incendie Batiment Electrique et des Auxiliaires des Sauvégarde (HLF) NIV, +0.00 M [Fire Protection Drawing Safeguard Building HLF (Div 1) Niv, +0.00 M]	SFLEZL00006890	F
Sectorisation Incendie Batiment Electrique et des Auxiliaires des Sauvégarde (HLA) NIV, +4.70 M [Fire Protection Drawing Safeguard Building HLA (Div 1) Niv, +4.70 M]	SFLEZL00006891	F

Sectorisation Incendie Batiment Electrique et des Auxiliaires des Sauvégarde (HLA) NIV, +8.10 M [Fire Protection Drawing Safeguard Building HLA (Div 1) Niv, +8.10 M]	SFLEZL00006892	F
Sectorisation Incendie Batiment Electrique et des Auxiliaires des Sauvégarde (HLA) NIV, +12.00 M [Fire Protection Drawing Safeguard Building HLA (Div 1) Niv, +12.00 M]	SFLEZL00006893	F
Sectorisation Incendie Batiment Electrique et des Auxiliaires des Sauvégarde (HLA) NIV, +16.80 M [Fire Protection Drawing Safeguard Building HLA (Div 1) Niv, +16.80 M]	SFLEZL00006894	F
Sectorisation Incendie Batiment Electrique et des Auxiliaires des Sauvégarde (HLA) NIV, +21.00 M [Fire Protection Drawing Safeguard Building HLA (Div 1) Niv, +21.00 M]	SFLEZL00006895	F
Sectorisation Incendie Batiment Electrique et des Auxiliaires des Sauvégarde (HLA) NIV, +24.70 M [Fire Protection Drawing Safeguard Building HLA (Div 1) Niv, +24.70 M]	SFLEZL00006896	F
Sectorisation Incendie Batiment Electrique et des Auxiliaires des Sauvégarde (HLF/HLA) Section A-A [Fire Protection Drawing Safeguard Building (HLF/HLA) Section A-A]	SFLEZL00006897	F
Sectorisation Incendie Batiment Electrique et des Auxiliaires des Sauvégarde (HLB/HLC) NIV, +23.90 M [Fire Protection Drawing Safeguard Building HLB/HLC (Div 2/3) Niv, +23.90 M]	EZL2007EN0145	D
Sectorisation Incendie Batiment Electrique et des Auxiliaires des Sauvégarde (HLG/HLH) NIV, -9.60 M [Fire Protection Drawing Safeguard Building HLG/HLH (Div 2/3) Niv, -9.60 M]	SFLEZL00006900	F
Sectorisation Incendie Batiment Electrique et des Auxiliaires des Sauvégarde (HLG/HLH) NIV, -5.00 M [Fire Protection Drawing Safeguard Building HLG/HLH (Div 2/3) Niv, -5.00 M]	SFLEZL00006901	F
Sectorisation Incendie Batiment Electrique et des Auxiliaires des Sauvégarde (HLG/HLH) NIV, +0.00 M [Fire Protection Drawing Safeguard Building HLG/HLH (Div 2/3) Niv, +0.00 M]	SFLEZL00006902	F
Sectorisation Incendie Batiment Electrique et des Auxiliaires des Sauvégarde (HLB/HLC) NIV, +4.70 M [Fire Protection Drawing Safeguard Building HLB/HLC (Div 2/3) Niv, +4.70 M]	SFLEZL00006903	F
Sectorisation Incendie Batiment Electrique et des Auxiliaires des Sauvégarde (HLB/HLC) NIV, +8.10 M [Fire Protection Drawing Safeguard Building HLB/HLC (Div 2/3) Niv, +8.10 M]	SFLEZL00006904	F
Sectorisation Incendie Batiment Electrique et des Auxiliaires des Sauvégarde (HLB/HLC) NIV, +12.00 M [Fire Protection Drawing Safeguard Building HLB/HLC (Div 2/3) Niv, +12.00 M]	SFLEZL00006905	F
Sectorisation Incendie Batiment Electrique et des Auxiliaires des Sauvégarde (HLB/HLC) NIV, +16.30 M [Fire Protection Drawing Safeguard Building HLB/HLC (Div 2/3) Niv, +16.30 M]	SFLEZL00006906	F
Sectorisation Incendie Batiment Electrique et des Auxiliaires des Sauvégarde (HLB/HLC) NIV, +21.00 M [Fire Protection Drawing Safeguard Building HLB/HLC (Div 2/3) Niv, +21.00 M]	SFLEZL00006907	F
Sectorisation Incendie Batiment Electrique et des Auxiliaires des Sauvégarde (HLB/HLC) NIV, +28.90 M [Fire Protection Drawing Safeguard Building HLB/HLC (Div 2/3) Niv, +28.90 M]	SFLEZL00006908	F
Sectorisation Incendie Batiment Electrique et des Auxiliaires des Sauvégarde (HLG/HLH) Section B-B [Fire Protection Drawing Safeguard Building HLG/HLH/HLB/HLC (Div 2/3) Section B-B]	SFLEZL00006909	F
Sectorisation Incendie Batiment Electrique et des Auxiliaires des Sauvégarde (HLI) NIV, -9.60 M [Fire Protection Drawing Safeguard Building HLI (Div 4) Niv, -9.60 M]	SFLEZL00006911	F

Sectorisation Incendie Batiment Electrique et des Auxiliaires des Sauvégarde (HLI) NIV, -5.00 M [Fire Protection Drawing Safeguard Building HLI (Div 4) Niv, -5.00 M]	SFLEZL00006912	F
Sectorisation Incendie Batiment Electrique et des Auxiliaires des Sauvégarde (HLI) NIV, +0.00 M [Fire Protection Drawing Safeguard Building HLI (Div 4) Niv, +0.00 M]	SFLEZL00006913	F
Sectorisation Incendie Batiment Electrique et des Auxiliaires des Sauvégarde (HLD) NIV, +4.70 M [Fire Protection Drawing Safeguard Building HLD (Div 4) Niv, +4.70 M]	SFLEZL00006914	F
Sectorisation Incendie Batiment Electrique et des Auxiliaires des Sauvégarde (HLD) NIV, +8.10 M [Fire Protection Drawing Safeguard Building HLD (Div 4) Niv, +8.10 M]	SFLEZL00006915	F
Sectorisation Incendie Batiment Electrique et des Auxiliaires des Sauvégarde (HLD) NIV, +12.00 M [Fire Protection Drawing Safeguard Building HLD (Div 4) Niv, +12.00 M]	SFLEZL00006916	F
Sectorisation Incendie Batiment Electrique et des Auxiliaires des Sauvégarde (HLD) NIV, +16.80 M [Fire Protection Drawing Safeguard Building HLD (Div 4) Niv, +16.80 M]	SFLEZL00006917	F
Sectorisation Incendie Batiment Electrique et des Auxiliaires des Sauvégarde (HLD) NIV, +21.00 M [Fire Protection Drawing Safeguard Building HLD (Div 4) Niv, +21.00 M]	SFLEZL00006918	F
Sectorisation Incendie Batiment Electrique et des Auxiliaires des Sauvégarde (HLD) NIV, +24.70 M [Fire Protection Drawing Safeguard Building HLD (Div 4) Niv, +24.70 M]	SFLEZL00006919	F
Sectorisation Incendie Batiment Electrique et des Auxiliaires des Sauvégarde (HLI/HLD) Section A-A [Fire Protection Drawing Safeguard Building HLI/HLD (Div 4) Section A-A]	SFLEZL00006920	F
Sectorisation Incendie Batiment Electrique et des Auxiliaires des Sauvégarde (HLD) NIV, +29.30 M [Fire Protection Drawing Safeguard Building HLD (Div 4) Niv, +29.30 M]	SFLEZL00006921	F
Safety Requirements for defining FB fire zones	ECEF071646	B1
Safety requirements for establishment of NAB fire zoning	ECEF071472	A1
Safety requirements for establishment of WTB fire zoning	ECEF071528	A1
Safety Requirements for the Establishment of Fire Zoning in the RB	ECEF071591	B1
Safety Requirements for defining Safeguard Auxiliary and Electrical Building Fire Zones	ECEF070601	B1
Identification of the high energy pipes of FA3 fuel building.	EYRL/2008/fr/0054	C
Consideration of High Energy Pipes Break in Safeguard Buildings	PF/2008/en/0002	E
Consideration of High Energy Pipes Breaks in the Nuclear Auxiliary	EZT/2009/en/0004	B
Study of high-energy line break in the effluent treatment building (HQ-)	EYRT/2009/fr/0047	B1
Study of high-energy line breaks in the EPR FA3 diesel generator buildings	EYRT/2009/fr/0022	A1
Identification of F1 and F2 equipment potentially aggressed by high energy pipe break inside fuel building of EPR FA3	EYRL2008/fr/0094	B
Procedure ENG 2-04: EPR cabling principles	ETDOFC/050261	B1
Test program for the qualification of the FA3 IRWST filtration system	NESS-F DC 373	E

Identification of the high energy pipes of FA3 Reactor building.	EYRL/2008/fr/0055	C
Definition of requirements for EPR openings	ECEIG070950	A0
Fire resistant cable wraps and cases in thermal and nuclear power plants	CRT 62-C-010-01	
Procedure ENG 2-0B: Marking Principle for Cable Trays, Cables, Conductors and Wiring for the EPR	ECEMA/04.1001	A1
Engineering Rule ENG2-51 : HV and HL cable sizing calculations	ECEMA060630	B1
Batiment Diesel HDA-HDB Plan Guide de genie civil P10 vue en plan niveau - 10.80	ECEIG061163	B
Batiment Diesel HDA-HDB Plan Guide de genie civil P10 vue en plan niveau -3.78	ECEIG061164	B
Batiment Diesel HDA-HDB Plan Guide de genie civil P10 vue en plan niveau +/-0.00 & +3.24	ECEIG061165	B
Batiment Diesel HDA-HDB Plan Guide de genie civil P10 vue en plan niveau +8.10m	ECEIG061166	B
Batiment Diesel HDA-HDB Plan Guide de genie civil P10 vue en plan niveau +13.86	ECEIG061167	B
Batiment Diesel HDA-HDB Plan Guide de genie civil P10 vue en plan niveau +19.44	ECEIG061168	B
Batiment Diesel HDA-HDB Plan Guide de genie civil P10 vue en plan niveau +24.72	ECEIG061169	B
Batiment Diesel HDA-HDB Plan Guide de genie civil P10 coupe A-A	ECEIG061170	B
Batiment Diesel HDA-HDB Plan Guide de genie civil P10 coupe B-B	ECEIG061171	B
Batiment Diesel HDA-HDB Plan Guide de genie civil P10 coupe C-C	ECEIG061172	B
Batiment Diesel HDA-HDB Plan Guide de genie civil P10 coupe D-D	ECEIG061173	B
General arrangement drawing safeguard building HLF (Div.1) plan view +4,70m	SFL-EZL-00-006856	E
General arrangement drawing safeguard building HLF (Div.1) plan view +8,10m	SFL-EZL-00-006857	E
General arrangement drawing safeguard building HLF (Div.1) plan view +12,00m	SFL-EZL-00-006858	E
General arrangement drawing safeguard building HLF (Div.1) plan view +16,80m	SFL-EZL-00-006859	E
General arrangement drawing safeguard building HLF (Div.1) plan view +21,00m	SFL-EZL-00-006860	E
General arrangement drawing safeguard building HLF (Div.1) plan view +24,70m	SFL-EZL-00-006861	E
General arrangement drawing safeguard building HLF (Div.1) plan view +29,30m	SFL-EZL-00-006862	E
General arrangement drawing safeguard building HLF/HLA (Div.1) plan view Section A-A	SFL-EZL-00-006863	E
General arrangement drawing safeguard building HLG/HLH (Div.2/3) plan view +4,70m	SFL-EZL-00-006868	E
General arrangement drawing safeguard building HLB/HLC (Div.2/3) plan view +8,10m	SFL-EZL-00-006869	E
General arrangement drawing safeguard building HLB/HLC (Div.2/3) plan view +12,00m	SFL-EZL-00-006870	E
General arrangement drawing safeguard building HLB/HLC (Div.2/3) plan view +16,30m	SFL-EZL-00-006871	E

General arrangement drawing safeguard building HLB/HLC (Div.2/3) plan view +21,00m	SFL-EZL-00-006872	E
General arrangement drawing safeguard building HLB/HLC (Div.2/3) plan view +28,90m	SFL-EZL-00-006873	E
General arrangement drawing safeguard building HLG/HLH (Div.2/3) section B-B	SFL-EZL-00-006874	E
General arrangement drawing safeguard building HLD (Div.4) plan view +4,70m	SFL-EZL-00-006879	E
General arrangement drawing safeguard building HLD (Div.4) plan view +8,10m	SFL-EZL-00-006880	E
General arrangement drawing safeguard building HLD (Div.4) plan view +12,00m	SFL-EZL-00-006881	E
General arrangement drawing safeguard building HLD (Div.4) plan view +16,80m	SFL-EZL-00-006882	E
General arrangement drawing safeguard building HLD (Div.4) plan view +21,00m	SFL-EZL-00-006883	E
General arrangement drawing safeguard building HLD (Div.4) plan view +24,70m	SFL-EZL-00-006884	E
General arrangement drawing safeguard building HLD (Div.4) plan view +29,30m	SFL-EZL-00-006885	E
General arrangement drawing safeguard building HLI/HLD (Div.4) section A-A	SFL-EZL-00-006886	E
Nuclear auxiliary building HNX general arrangement drawing plan view roof	SFL-EZL-00-200026	F
Application of the MTE175 « Breaches of High Energy pipes, load caused on the structural steelworks	ECEIG091393	A1
Dropped Loads – Summary of Design Basis and Principles	ECEIG111683	A
Identification of representative Drop Load Cases from the Safety Class 2 cranes	ECEIG111791	A
Identification of Representative Drop Load Cases from the Safety Class 1 Polar Crane in the Reactor Building	PEPS-G/2011/en/1060	A
Internal Missiles – Selection of the RCC-M Components for which a Detailed Analysis is performed	ECEIG111942	A
List of EPR HK building doors	EYRS-2009-FR-0307	D1
High Energy Pipe Break - Treatment of open points - Common mode failure and other safety related aspects in the Safeguard Buildings	EZLT2010EN0005	B
Treatment of open points from the 1st Phase HELB Study in the Fuel Building	EYRT/2010/fr/0050	B1
Application Note for a Drop Load Impact on a Reinforced Concrete Slab	EDEIG111395	A
EPR UK – RS2 cranes – Drop Load Impact Calculations	ECEIG111620	A
Consequences on the Reactor of an Accidental RPV Head Drop During it's Handling	PEER-F DC 71	B
Drop of a Reactor Cavity Cover Slab on the RPV Closure Head Analysis	PEPR–F DC 85	B
Check of Bearing Capacity of Reinforced Concrete Reactor Pool Slab subjected to Drop Load of a Concrete Cover Slab and a Multi Stud Tensioning Machine	PECS-G/2011/en/0018	B
Impact calculation on the civil structures in case of failure of the regenerative heat exchanger RCV6220EX	ECEIG112399	A
EPR KOPRA Tests – Test Synthesis for CRDM	NEER-F DC 149	C

Exterior and Interior Seismic Analysis	NEEA-G 2008 en 1092	A
FA3 CRDM Preliminary Seismic Analysis	NEEA-G 2006 en 1073	D
FA3 EM2 CW Interfaces	NECE DC 208	B
EPR RCP Synthesis note on SSSS and associated O-RINGS Qualification Tests	NEEG-F DC 160	C
Completeness Analysis of Extra Boration System	NESS-F DC 633	A
Periodic Test Instructions of Extra Boration System	NESS-F DC 632	A
Book of Technical Rules - Irradiation resistance test and thermodynamic and chemical conditions in the containment building resistance test - Generic provisions for the tests	BTR_91C113	00
Book of Technical Rules - Equipment earthquake resistance test - Generic provisions for the biaxial time history test	BTR_91C112	00
General specification for qualification of valves for accident operating conditions – Test based method	ENRECI070143	A1
Handbook for qualification by analogy of valves under accident conditions - Case of seismic resistance	ENMRE893017	C1
Review of OEF associated with UK Nuclear Lifting Operations	PEPS-F DC 9	A
Data Sheet for Medium Head Safety Injection pump sets	NEEG-F DC 47	D
Control rod drive mechanism – Latch Unit – Acceptance drawing	NEER-G-00-100277	F
FA3 Design at 4500 MWth (Dimensionnement FA3 à 4500 MWth)	NEPR-F DC 547	B
Rule for choice and codification of the valves	SFL EF MF 2006 890	B1
EQUIPMENT SPECIFICATION CONTROL ROD DRIVE MECHANISMS D143	NEER-G/2006/en/1517	D
PROCEDURE ENG 2.05 : Definition of the required RIN for the EPR Project	ECEMA060036	B1
Water hammer on CCWS	EYTS/2007/fr/0167	A
MAAP files supporting the UK EPR GDA PSA.		
UK EPR Level 2 PSA Supporting Severe Accident Analysis Calculations	NEPS-F DC 459	A
UK EPR Severe Accident Source term Calculations	NEPS-F DC 460	A
Guidance notes on applying the model and PSA model changes		
Excel file summarizing all of the basic events of the PCSR 2009 Risk Spectrum runs with a Risk Increase Factor (RIF) above 2 and/or a Fussel-Vesley (FV) above 1E-2.		
UK EPR - Response to Regulatory Observation 18	NEPSFP/10.154	
UK EPR MAAP4.0.7 Parameter File Development – Model	NEPS-F DC 517	A
UK EPR MAAP4.0.7 Parameter File Development - Geometry	NEPS-F DC 518	A
UK EPR MAAP4.0.7 Parameter File Development – Include Files	NEPS-F DC 519	A

OL3 Topical report 1004 part 1– Consequences of unrestricted 2A breaks as beyond design	NEPS-F DC 87	B
OL3 Topical report E1004 part 2– Consequences of beyond design unrestricted 2A breaks on the RPV, RPV internals and fuel assemblies	NFPMR DC 1005	D
UK EPR – Response to Regulatory Observation UK-EPR-68, Actions A1.c (i) and A1.c (ii)	PEPSPF10.793	
Result files for UK EPR PSA model for GDA step 4		
UK EPR PSA model logbook		
UK EPR PSA model for GDA step 4		
UK EPR PSA LIVING STATUS DOCUMENT – PSA model UK_HSE3		
UK EPR PSA LIVING STATUS DOCUMENT – PSA models from UK_HSE2 to UK_HSE3		
PSA SUPPORT STUDIES (ATWS)	PSRR DC 25	B PRE
Function events LUHS_AB		
Function events SLB_SO_SGTR_AB		
Function events LOCC7_D		
Function events WS_LMF_A		
Sequences LUHS_AB		
Sequences SLB_SO_SGTR_AB		
Sequences LOCC7_D		
Sequences WS_LMF_A		
Medium Loss of Coolant Accident (Event tree PBM2-_AB)		
Fire in the Switchgear Building (Event tree IH F SWGB_AB)		
Inner Containment – Detailed Design Probability Safety of the Ultimate Strength Capacity of Inner Containment	CEB 11508RP251	B
Containment potential leakage paths	NEEA-G/2007/en/1014	D
EPRtm UK: Anticipated Transient Without Scram in case of Spurious opening of one Pressuriser Safety Valve	PEPR10.1372	
Level 2 PSA – Suppression of LHSI and Impact on LRF, LERF, Individual/Societal Risk	PEPSP-F 11.403	
EPR FA3 Functional description of RRC-A functions	NEPR–F DC 52 (TR07/500)	B
Microshield 5.05 Verification and Validation Report	NPASDEREF060153	5
Collection of requirements and methodology for the realization of the studies of radiation protection	ECEIG061049	B1
EPR FA3 – Radiation protection guidelines	ECEIG051117	B
Thermal and irradiation ageing of materials foreseen for neutron protections	NEEMFDC87	B

Methodological note to define iodine risk and / or aerosol risk rooms in FA3	ECEIG061473	A
EPR FA3 – Radiological study of the handling of instrumentation lances and control rods during normal outage	NEEM-F DC 50	B
Batiment Reacteur Plan D'installation Generale Niveau -6.15m	SFL-EYRL-00-035025	E
Batiment Reacteur Plan D'installation Generale Niveau -2.30m	SFL-EYRL-00-035026	E
Batiment Reacteur Plan D'installation Generale Niveau +1.50m	SFL-EYRL-00-035027	E
Batiment Reacteur Plan D'installation Generale Niveau +5.15m	SFL-EYRL-00-035028	E
Batiment Reacteur Plan D'installation Generale Niveau +8.70m	SFL-EYRL-00-035029	E
Batiment Reacteur Plan D'installation Generale Niveau +13.80m	SFL-EYRL-00-035030	E
Batiment Reacteur Plan D'installation Generale Niveau +19.50m	SFL-EYRL-00-035031	E
Batiment Reacteur Plan D'installation Generale Niveau +24.10m	SFL-EYRL-00-035032	E
Batiment Reacteur Plan D'installation Generale Niveau +28.50m	SFL-EYRL-00-035033	E
Qualification of the PANTHERE V1.6 code	ENTERP090200	A
Modelling hypotheses and input data for calculating dose rates in the EPR-FA3 reactor building	ECEIG092367	A1
UK EPR - PRIMARY NUCLIDE SOURCE TERM DERIVATION WITHIN SYSTEMS	PEEM-F DC 27	A
ALARA – Maintenance and Radiation Protection	DNM03322	D FIN
EPR – Estimate of personnel exposure for maintenance works outside the reactor building	ECEIG021077	B
EPR RF002 - TG4 Radiation Protection - Dose assessment report and radiation protection measures inside RB	EEGDC2480	D FIN
Impact of Reactor Building Two Room Concept on HVAC Systems in RB and NAB	ECEF022210	A
Radiological classification of the Fuel Building priority rooms – level – 9.60m	EYRL/2009/fr/0054	B1
Frame and contents of the accessibility studies in post-accident situations	ECEIG091447	B1
EPR - Methods and assumptions used for assessing accessibility under severe accident conditions	ENTERP090169	A1
FUEL BUILDING GENERAL ARRANGEMENT DRAWING PLAN VIEW LEVEL -9.60m	EYRL/2008/fr/0007	B1
FUEL BUILDING GENERAL ARRANGEMENT DRAWING PLAN VIEW LEVEL -6.20m	EYRL/2008/fr/0008	B1
FUEL BUILDING GENERAL ARRANGEMENT DRAWING PLAN VIEW LEVEL -3.40M	EYRL/2008/fr/0009	B1
FUEL BUILDING GENERAL ARRANGEMENT DRAWING PLAN VIEW LEVEL +0.00m	EYRL/2008/fr/0010	C1
FUEL BUILDING GENERAL ARRANGEMENT DRAWING PLAN VIEW LEVEL +3.70m	EYRL/2008/fr/0011	B1
FUEL BUILDING GENERAL ARRANGEMENT DRAWING PLAN VIEW LEVEL +7.40m	EYRL/2008/fr/0012	B1
FUEL BUILDING GENERAL ARRANGEMENT DRAWING PLAN VIEW LEVEL +11.10m	EYRL/2008/fr/0013	B1

FUEL BUILDING GENERAL ARRANGEMENT DRAWING PLAN VIEW LEVEL +14.80m	EYRL/2008/fr/0014	B1
FUEL BUILDING GENERAL ARRANGEMENT DRAWING PLAN VIEW LEVEL +19.50m	EYRL/2008/fr/0015	B1
ESPN: Risk Analysis for RCV HP letdown assembly	NESS-G/2009/en/1022	B
EPR™ FA3 – Cricality studies for wet and dry storage - Partial Answer to the additional queries related to the TQ-EPR-1019, in relation with the TQ-EPR-595	pepcf.10.1095	
Summary report on qualification of MCNP4 code for criticality calculations	EPD DC 431	A
EPR UK - Partial Answer to the additional queries related to the TQ-EPR-1019, in relation with the TQ-EPR-595	PEER-F 101352	
Room air contamination – Reactor building	NEEM-F DC 45	B
UK EPR LEVEL 2 SUPPORTING ANALYSIS	PEPS-F DC 53	A
Methodology to calculate does rate for rooms classification	EYRL/2008/FR/0018	B
Validation of the PANTHERE Computational Core	AB04B020/DU/05011	1.0
General arrangement drawing safeguard building HLF (DIV.1) plan view -9,60M	SFL-EZL-00-006853	E
General arrangement drawing safeguard building HLF (DIV.1) plan view -5,00M	SFL-EZL-00-006854	E
General arrangement drawing safeguard building HLF (DIV.1) plan view +0,00M	SFL-EZL-00-006855	E
General arrangement drawing safeguard building HLG/HLH (DIV.2/3) plan view -9,60M	SFL-EZL-00-006865	E
General arrangement drawing safeguard building HLG/HLH (DIV.2/3) plan view -5,00M	SFL-EZL-00-006866	E
General arrangement drawing safeguard building HLG/HLH (DIV.2/3) plan view +0,00M	SFL-EZL-00-006867	E
General arrangement drawing safeguard building HLI (DIV. 4) plan view -9,60M	SFL-EZL-00-006876	E
General arrangement drawing safeguard building HLI (DIV. 4) plan view -5,00M	SFL-EZL-00-006877	E
General arrangement drawing safeguard building HLI (DIV. 4) plan view +0,00M	SFL-EZL-00-006878	E
Nuclear Auxiliary Building HNX general arrangement drawing plan view -9,60M	SFL EZL 00 200016	F
Nuclear Auxiliary Building HNX general arrangement drawing plan view -6,50M	SFL EZL 00 200017	F
Nuclear Auxiliary Building HNX general arrangement drawing plan view -3,40M	SFL EZL 00 200018	F
Nuclear Auxiliary Building HNX general arrangement drawing plan view +0,00M	SFL EZL 00 200019	F
Nuclear Auxiliary Building HNX general arrangement drawing plan view +3,70M	SFL EZL 00 200020	F
Nuclear Auxiliary Building HNX general arrangement drawing plan view +7,40M	SFL EZL 00 200021	F
Nuclear Auxiliary Building HNX general arrangement drawing plan view +10,5M	SFL EZL 00 200022	F
Nuclear Auxiliary Building HNX general arrangement drawing plan view +15,20M	SFL EZL 00 200023	F

Nuclear Auxiliary Building HNX general arrangement drawing plan view +19,50M	SFL EZL 00 200024	F
Nuclear Auxiliary Building HNX general arrangement drawing plan view +24,70M	SFL EZL 00 200025	F
Accessibility to SABs, FB in post-accident conditions (SA and PCC4 LB-LOCA) in the long term phase	ECEIG101497	A1
Nuclear Auxiliary Building Radiation Contingency Drawing Plan View -9,60m	SFL-EZL00-20044	E
Nuclear Auxiliary Building Radiation Contingency Drawing Plan View -6,50m	SFL-EZL00-20045	E
Nuclear Auxiliary Building Radiation Contingency Drawing Plan View -3,40m	SFL-EZL00-20046	E
Nuclear Auxiliary Building Radiation Contingency Drawing Plan View +0,00m	SFL-EZL00-20047	E
Nuclear Auxiliary Building Radiation Contingency Drawing Plan View +3,70m	SFL-EZL00-20048	E
Nuclear Auxiliary Building Radiation Contingency Drawing Plan View +7,40m	SFL-EZL00-20049	E
Nuclear Auxiliary Building Radiation Contingency Drawing Plan View +10,50m	SFL-EZL00-20050	E
Nuclear Auxiliary Building Radiation Contingency Drawing Plan View +15,20m	SFL-EZL00-20051	E
Nuclear Auxiliary Building Radiation Contingency Drawing Plan View +19,50m	SFL-EZL00-20052	E
Nuclear Auxiliary Building Radiation Contingency Drawing Plan View +24,70m	SFL-EZL00-20053	E
Radiation Contingency Drawing Safeguard Building HLF (Div.1) Plan View -9,60m	SFL-EZL-00-006923	E
Radiation Contingency Drawing Safeguard Building HLF (Div.1) Plan View -5,00m	SFL-EZL-00-006924	E
Radiation Contingency Drawing Safeguard Building HLF (Div.1) Plan View +0,00m	SFL-EZL-00-006925	D
Radiation Contingency Drawing Safeguard Building HLG/HLH (Div.2/3) Plan View -9,60m	SFL-EZL-00-006928	E
Radiation Contingency Drawing Safeguard Building HLG/HLH (Div.2/3) Plan View -5,00m	SFL-EZL-00-006929	E
Radiation Contingency Drawing Safeguard Building HLG/HLH (Div.2/3) Plan View +0,00m	SFL-EZL-00-006930	D
Radiation Contingency Drawing Safeguard Building HLI (Div.4) Plan View -9,60m	SFL-EZL-00-006933	F
Radiation Contingency Drawing Safeguard Building HLI (Div.4) Plan View -5,00m	SFL-EZL-00-006934	E
Radiation Contingency Drawing Safeguard Building HLI (Div.4) Plan View +0,00m	SFL-EZL-00-006935	D
Zonage Radiologique Batiment Combustile vue en Plan -9,60m [Radiation Protection Fuel Building Plan View Level -9,60m]	EYRT/2009/FR/0066	C
Zonage Radiologique Batiment Combustile vue en Plan -6,20m [Radiation Protection Fuel Building Plan View Level -6,20m]	EYRT/2009/FR/0067	C
Zonage Radiologique Batiment Combustile vue en Plan -3,40m [Radiation Protection Fuel Building Plan View Level -3,40m]	EYRT/2009/FR/0068	C
Zonage Radiologique Batiment Combustile vue en Plan +0,00m [Radiation Protection Fuel Building Plan View Level +0,00m]	EYRT/2009/FR/0069	C
Zonage Radiologique Batiment Combustile vue en Plan +3,70m [Radiation Protection Fuel Building Plan View Level +3,70m]	EYRT/2009/FR/0070	C
Zonage Radiologique Batiment Combustile vue en Plan +7,40m [Radiation Protection Fuel Building Plan View Level +7,40m]	EYRT/2009/FR/0071	C

Zonage Radiologique Batiment Combustile vue en Plan +11,10m [Radiation Protection Fuel Building Plan View Level +11,10m]	EYRT/2009/FR/0072	C
Zonage Radiologique Batiment Combustile vue en Plan +14,80m [Radiation Protection Fuel Building Plan View Level +14,80m]	EYRT/2009/FR/0073	C
Zonage Radiologique Batiment Combustile vue en Plan +19,50m [Radiation Protection Fuel Building Plan View Level +19,50m]	EYRT/2009/FR/0074	C
Zonage Radiologique Batiment Combustile vue en Plan +24,20m [Radiation Protection Fuel Building Plan View Level +24,20m]	EYRT/2009/FR/0075	C
Zonage Radiologique Batiment Reacteur en AT vue en Plan -6,15m [Radiation Protection Reactor Building in Unit Outage Plan View Level -6,15m]	EYRT/2010/FR/0018	B
Zonage Radiologique Batiment Reacteur en AT vue en Plan -2,30m [Radiation Protection Reactor Building in Unit Outage Plan View Level -2,30m]	EYRT/2010/FR/0019	B
Zonage Radiologique Batiment Reacteur en AT vue en Plan +1,50m [Radiation Protection Reactor Building in Unit Outage Plan View Level +1,50m]	EYRT/2010/FR/0020	B
Zonage Radiologique Batiment Reacteur en AT vue en Plan +5,15m [Radiation Protection Reactor Building in Unit Outage Plan View Level +5,15m]	EYRT/2010/FR/0021	B
Zonage Radiologique Batiment Reacteur en AT vue en Plan +8,70m [Radiation Protection Reactor Building in Unit Outage Plan View Level +8,70m]	EYRT/2010/FR/0022	B
Zonage Radiologique Batiment Reacteur en AT vue en Plan +13,80m [Radiation Protection Reactor Building in Unit Outage Plan View Level +13,80m]	EYRT/2010/FR/0023	B
Zonage Radiologique Batiment Reacteur en AT vue en Plan +19,50m [Radiation Protection Reactor Building in Unit Outage Plan View Level +19,50m]	EYRT/2010/FR/0024	B
Zonage Radiologique Batiment Reacteur en AT vue en Plan +24,20m [Radiation Protection Reactor Building in Unit Outage Plan View Level +24,20m]	EYRT/2010/FR/0025	B
Zonage Radiologique Batiment Reacteur en AT vue en Plan +28,50m [Radiation Protection Reactor Building in Unit Outage Plan View Level +28,50m]	EYRT/2010/FR/0026	B
H2 PAR facility in Proceedings of the OECD NES CSNI workshop on the implementation of hydrogen mitigation.	AECL-11762 NEA/CSNI/R(96)8	
Phebus fpt3 main relevant results likely to impact safety. EUROSAFE 2007		
IAEA Nuclear Energy Series - Determination and Use of Scaling Factors for Waste Characterization in Nuclear Power Plants	NW-T-1.18	
Assistance and Recommendations for Tritium Management in NPP Systems	EDEAPC080379	A1
REGULATORY OBSERVATION : RO-UKEPR-44.A5 – Item (b) Response	EDECME100870	
EPR UK: SG Tubes – Precautions and Control During Procurement and SG Tubes Manufacturing to Ensure Low Radioactivity	EFFHQ11.18043	A
Tritium Evaluation for UK EPR	PEEM-F DC 62	A
Balance Between Experience and Modelling for the Justification of a Selected Group of Chemical and Radiochemical Specifications	EDECME110117	A
The Nuclear Sampling System of the EPR NPP in Flamanville 3		
The Flamanville EPR: The Role of Chemistry in the Design and Development of a Third Generation PWR		

Baseline Change and additional supporting documents – transfer of activity in steam generators	ENTEAG090076	A
CVCS system specification	ITSR DC 179	E
Boron Meter Detailed Specification	NGLLN/2006/en/1002	H
Completeness analysis note on periodic tests for the RCV system	NESS-G/2006/en/1024	D
Design Transients for RCV System (Loading Specifications)	NESS-G/2007/en/1018	C
Loading Specifications MCP N°1 seal injection and leak off subsystem RCV7	NESS-G/2006/en/1023	C
FA3 EPR™ Preliminary Primary Coolant Chemistry Specification	PEEM-F DC 7	A
Hydrogen Detection System (KRH) System Specification	ECEMA060737	A1
T. Sevón, T. Kinnunen, J. Virta, S. Holmström, P. Koskinen, T. Kekki. - HECLA5 Experiment on Melt–Concrete Interactions	VTT-R-06418-09	
COSACO – Engineering model for the simulation of corium sacrificial concrete interactions – Part B: Validation with the aid of one-dimensional experiments.	KWU NA-T/1998/E090	A
M. Farmer, et al, S. Basu; “OECD MCCI Project - Final Report”, Argonne National Laboratory	OECD/MCCI-2005-TR06	
P. Jolivet, A. Tamba, and F. Chahma, “Material Selection According to ALARA during Design Stages of EPR™”, ISOE International Symposium		
Steam Generator Tube Material – Experience Feedback and Experimental Results of Alloy 690TT and Alloy 800 Release	NEEM-F 09 1194	A
Addition to the document "dissociation of iodine aerosols into gaseous iodine in hydrogen recombiners - impact on iodine source term" DPAMSEMIC2011353 Rev 1 IRSN	ENTEAG120129	A
Activity release during SGTR	ENTEAG100137	
Source term for SGTR	ENTEAG100144	
Evaluation of radiological consequences in accident conditions for EPR using fleet-based methodology - Comparison with FANP-GmbH design calculations	ENTEAG090044	A
State-of-the-art review of radioactivity carryover factors in the event of SGTR	ENTEAG090134	A
Realistic methods for calculating the release of radioactivity following steam generator tube rupture faults	EUR 15615	
Gas distribution in the containment during a severe accident and assessment of the potential hydrogen combustion risk	PEPA-G/2011/en/1009	A
Temperature loads from gas release, recombiner operation and slow hydrogen combustion in the dome during a severe accident	PEPA-G/2011/en/1010	A
Pressure loads from fast hydrogen combustion during a severe accident and assessment of the risk of a deflagration-to-detonation transition	PEPA-G/2011/en/1011	A
Hydrogen combustion after reactor vessel failure during a severe accident	PEPA-G/2011/en/1012	A
S. Hellmann, F. Funke, V. Lansmann and B. Friedrich; "Physico-Chemical and Material Aspects of the Core Melt Retention Concept of the EPR", Proceeding Paper of the OECD Workshop on Ex-Vessel Debris Coolability		
W. Koller, W. Steinwarz, N. Dyllong; CORESA Large-scale experiments for investigating the impact of a core melt on structural materials		

System CMSS: Justification of the type of sacrificial concrete used in the reactor pit	NEPSG/2008/en/1156	D
Passive Autocatalytic Recombiner (PAR) – Empirical equation for H2 and CO Depletion Rate calculation	NGPS5/2003/en/0040	A
Main Secondary System Overpressure Protection File Safety Accessories	NEEG-F DC 686	A
Overpressure Protection File of the CPP (Main Primary System)	NEEG-F DC 685	A
EPR-UK : Réponse à la RO-UKEPR-25.A5	nepcf.09.0486	
Validation of Neutron Propagation Calculations using the DORT and DOTSYN Codes and the Special Dosimetry Benchmark Experiment at the French St Laurent Reactor		
Fatigue Crack Growth Law Equations for Ferritic Materials Covered by RCC-M M 2110 and M 2120	NEEM-F09.0565	A
EPR-UK : Réponse à la RO-UKEPR-25.A4	Nepdf09.0305	
EPR UK : Réponse à la RO-UKEPR-25.A6	Nepdf09.0317	
RO20 Avoidance of Fracture – margins based on size of crack like defects - fracture mechanics and minimum toughness aspects	NEER-F 91.201	
Framatome view on the comparison between Class 1 and Class 2 design piping rules		0
Stress Analysis Criteria for Piping. RCC-M 2002 Rules and Validation	SMiRT 20-Division 5	
RO20 NDT manufacturing qualification - action plan	EDEEM090134	C
UK EPR selection of a prototype NDT application for NDT manufacturing qualification	PEEM-F 10 0330	
Analytical method for the calculation of J parameter on cracked pipes under thermal loading and mechanical plus loading		
Flaw analysis in the French RSE_M and RCC-MR code appendices		
French RSE-M and RCC-MR code appendices for flaw analysis: Presentation of the fracture parameters calculation—Part I: General Overview		
French RSE-M and RCC-MR code appendices for flaw analysis: Presentation of the fracture parameters calculation—Part II: Cracked plates		
French RSE-M and RCC-MR code appendices for flaw analysis: Presentation of the fracture parameters calculation—Part III: Cracked pipes		
French RSE-M and RCC-MR code appendices for flaw analysis: Presentation of the fracture parameters calculation—Part IV: Cracked elbows		
French RSE-M and RCC-MR code appendices for flaw analysis: Presentation of the fracture parameters calculation—Part V: Elements of validation		
Analytical expression of the thermal stresses in a vessel or pipe with cladding submitted to any thermal transient		
UK GDA - RPV lower internals design against gross failure	NEPS-F DC 556	A
Technical report on non destructive examination of RCP Pump Casing	PEEM-F 100738	A
Justification of the residual stress level in low alloy steel welds	PEER-F 100657	A

Implementation of break preclusion concept on the RCS and MSL pipework comparison between the EPR and previous plants (900MW, N4)	ECEMA101022	A
Analysis of SIS accumulator gross failure	ENSNDR100062	B
Improving knowledge of the metallurgical features of large forgings by optimising the experimental test program		
Hollow ingot the better way to high quality forgings for pressure vessels		
Improvements in the metallurgical quality of large forged pieces : case of EPR nozzle shell with integral flange		
Under clad cold cracking of SG tube sheet and RPV nozzles		
EPRTM UK Comments on J.F. KNOTT Report	PEEM-F 10.0776	
RO20 : Justification of FE selected areas	N°:PEER-F.10.1026	
RCC-M: Content, Working Approach and Future Evolutions		
UK EPR high structural integrity components – Non-destructive testing at the manufacturing stage - Definition and role of the expert panel	EDEEM100138	A
UK EPR Avoidance of Fracture Approach – Fast fracture analytical studies for critical defect sizes determination in unbreakable, no missile, break preclusion components	PEERF 10-0968	B
UK EPR Avoidance of Fracture Approach – Fracture mechanics prototype application	PEERF 10-0613	B
EPR UK – Justification of the residual stresses in ferritic welds and in low alloy steel base metal	PEEM-F.10.0863/A	A
UK EPR Avoidance of Fracture Approach – Fast fracture analytical studies for critical defect sizes determination in main coolant lines	PEERF 10-1286	B
UK EPR Avoidance of fracture approach - Critical defect size in Break Preclusion classified secondary piping	PESP-F.10.0460	1
UK EPR Avoidance of Fracture Approach – Fast fracture studies for critical defect sizes determination in Reactor Pressure Vessel Cover Head weld	PEERF 10-1525	
Proceeding from 7th ECNDT Conference Copenhagen 1998 – Detection and Characterisation of Defects Perpendicular to the Surface of Welds by JM Tchilian et al		
Reactor Coolant Pump of the EPR – Flywheel mechanical and fracture mechanical analysis	PEER-F 10.1674	B
UK EPR Avoidance of Fracture Approach – Fast fracture studies for critical defect sizes determination in the RPV outlet seton weld	PEER-F 10.1871	A
UK EPR End of life critical defect size determination: Specific modified RSE-M approach consistent with R6 rules to compare with the RSE-M approach	PEER-F 10.1936	A
Qualification Body Proposal for UKEPR	EDEEM100190	A
Inspection specification for NDT application on weld between upper head and upper shell of the pressuriser	EDEETC100837	E
UK EPR Qualification Proposal for UT examination of the prototype application	PEEM-F 10.2203	A
UK EPR Avoidance of Fracture Approach- Fast Fracture Analysis Specifications	PEER-F 10.0217	C
UK EPR Avoidance of Fracture Approach – Fast fracture studies for critical defect sizes determination in the RPV outlet dissimilar metal weld	PEER-F 10-2068	

Definition of the residual stress level in Inconel Dissimilar Metal Weld joints for the EPR-UK critical defect size Fracture Mechanics Analyses	NEER-F 10.2031	B
UK EPR Avoidance of Fracture Approach – Fast fracture studies for critical defect sizes determination in the Steam Generator Tubesheet welded connections (tubesheet to primary and secondary head welds)	PEEG-F 10.1395	B
Inspection Procedure – Ultrasonic Testing of pressuriser uppershell/upper head	COUSUK-NPR0200	A
Inspection Procedure - Ultrasonic Testing of pressuriser butt welds Tandem method	COUSUK-NPR0201	A
UK EPR Technical Justification for the UT examination of the prototype application	PEEM-F 10.2217	A
Minutes of Expert Meeting – Definition of relevant defect for prototype application	PEEM-F 10.1111	D
A UK EPR Avoidance of Fracture Approach – Fast fracture studies for critical defect sizes determination using surrogate R6 method (RSE-M V' Option)	PEER-F 10.2069	C
Reactor Coolant Pump casing of EPR fast fracture analysis	PEER-F 10.2038	B
Technical Manufacturing Program for Nozzle Shell With Internal Flange	EFFN-7556-13	O
Complementary Testing Program For Nozzle Shell With Internal Flange	EFFN-7556-73	L
Part Qualification Report For Nozzle Shell With Internal Flange	PQ03001	C
EPR™ FA3 : PRESSURIZER STRESS ANALYSIS SPECIFICATION	NEER-F DC 278	B
EPR™ FA3 : PRESSURIZER FAST FRACTURE ANALYSIS SPECIFICATION	NEER-F DC 317	B
EPR™ - Assessment of Fast Fracture Risk In the Most Sensitive Areas	PEEG-F DC 7	A
COMPLEMENTS TO RCC-M FOR PREVENTION OF COLD AND REHEAT CRACKING	EET DC 118	C
PZR SPN support studies - PZR surge nozzle fatigue evaluation with and without thermal sleeve	NEER-F DC 59	C
Pressuriser upper dome details	NEEG-F DB 1222	E
Pressuriser aspersion lance	NEER-F DB 1223	C
79/19TE steam generator secondary side pressure vessel details Right and Left S.G.s	NEEG-F DB 1210	G
Pressurizer supports lower brackets sections and details	NEER-F DB 1265	C
Generateur de vapeur 79/19TE boite a eau primaire percage de la plaque tubulaire GV gauche et droit	NFPMG-DB-1206	D
79/19TE steam generator Tube bundle and part list – Right and left hand SGs	NEEG-F-DB-1212	C
79/19TE Steam Generator lower internals 1/2 flow distribution baffle right and left hand SGs	NEEG-F DB 1216	G
79/19TE steam generator lower internals 1/2 tube support plate right and left hand SGs	NEEG-F DB 1217	F
79/19TE steam generator lower internals tube support plate right and left hand SGs	NEEG-F DB 1218	F
Generateur de vapeur 79/19TE SG openings closure devices right and left hand SGs	NEEG-F-DB-1233	E

Generateur de vapeur 79/19TE SG openings closure devices - part list right and left SGs	NEEG-F DB 1234	D
Generateur de vapeur 79/19TE boite a eau primaire nomenclature GV gauche et droit	NFPMG-DB-1207	J
Welding Procedure Qualification: Butt Welding of Low Alloy Steel 20MND5 by Automatic Wire/Powder Flux Process (121) in Flat Position with SRHT (Thickness=75mm)	DFHM/TSD-PQS451	D
Welding Procedure Qualification: Stainless Steel Cladding on Low Alloy Steel 20MND5 by Submerged Arc Welding (122) in Flat Position (PA) (2 Layers)	DFHM/TSD-PQS461	C
HSE Feedback about the Potential Implications of DORT and MCNP Calculations - RP Position concerning ISP anticipation factor and DPA issue	PEEM-F 11.0642	
Christian PICHON et Al., Neutron Spectrum Effect and Damage Analysis on Pressure Vessel Steel Irradiation Behaviour	ASTM STP 1366	
S. JUMEL- JC VAN DUYSEN, Simulation of Irradiation Effects in Light Water Reactor Vessel Steels – Experimental Validation of RPV-1, Journal of Nuclear Materials	366 (2007) 256-265	
EPR Steam Generator – Assessment of fatigue risk in the most sensitive areas	PEEG-F DC 24	A
UK EPR Pump Casing - Test Report (Processes, Application and Summary of Results)	EFFQM 10/17210	C
Influence of Neutron Spectrum on the Embrittlement of RPV steels - ESTEREL Programme Final Report	DRE-SRO-SIEN 95-123	
Technical Justification for UT examination of prototype application	PEEM-F.10 2217	B
EPR™ Reactor Coolant Pump - Flywheel mechanical and fracture mechanics analysis	PEER-F 10.1674	C
UK EPR Avoidance of Fracture Approach – Fast fracture studies in Reactor Pressure Vessel outlet Dissimilar Metal Weld	PEER-F 10.2068	B
UK EPR ™ Intermediate and Large Break LOCA - Feasibility study related to GDA Issue GI-UKEPR-SI01	PEPR-F 11.1421	
The Capability of Ultrasonic Inspection Procedures N1/5799/UT1 Rev 0 and N1/5799/UT2 Rev 1 Manufacturing Inspection of Reactor Coolant Loop Pipework – Narrow Gap TIG Shop Fabricated Pipe Butt Welds	TD/IRB/REP0694	1
Test report – Ultransonic Examination of the SG safe-end NGT Process	EFFQM 12 17004	B
EPR UK UT Modelling Study for MSSV welds inspection and Steam Isolation Valve weld inspection	28182	2
UK EPR Results of simulation trials of Ultrasonic examination on MSL girth welds	PEEM-F.111602	C
MAAP4 – Synthesis Assessment Report	NEPD-F DC 10213	B
Protection of the groundwater and protection from the water tables	E-N-ES-GC/00-00705	A
Air Filters and Water Filters	EPR03	
Evaporator Concentrates	EPR06	
Ion Exchange Resins	EPR01	
Maintenance and Operational Low Level Waste	EPR04	
Maintenance and Operational Very Low Level Waste	EPR04bis	

SGBD Ion Exchange Resins	EPR01bis	
Sludges	EPR05	
Spent Filter Cartridges	EPR02	
Stainless Steel Waste	EPR04ter	
UK EPR Disposability Assessments - Preliminary Information	RWMD EPR00001	
UK EPR Disposability Assessment – Preliminary Information	RWMD EPR00002	
UK EPR Disposability Assessment – Preliminary Information	RWMD EPR00004	
Spent resins (ILW) raw waste	EPR11	
Spent Cartridge Filters (ILW)	EPR12	
Spent Cartridge Filters (LLW + ILW)	EPR13	
Operational Waste > 2mSv h^{-1} (LLW + ILW)	EPR14	
Wet sludges (LLW + ILW)	EPR15	
Evaporator Concentrates (LLW + ILW)	EPR16	
Spent resins (ILW) raw waste	EPR21	
Operational Waste > 2mSv h^{-1} (LLW + ILW)	EPR24	
Wet sludges (LLW + ILW)	EPR25	
Evaporator Concentrates (LLW + ILW)	EPR26	
EPR Operational Waste and Datasheets	D4507081208	0
UK EPR spent fuel characteristics for NDA disposability assessment (ENU) fuel	NEPCF 08.1335	
RWMD Nature and Quantities Questions: Response by AREVA and EDF dated 28th July 2008 (Final Issue)		
COMPLEMENTARY Information ABOUT DECOMMISSIONING ILW WASTE		
Assessment of the safety of radioactive waste management	IAEA-CN-78/43	
NIREX 4 metre Box with 100mm concrete shielding		
NIREX 3 metre Box		
NIREX 3 metre Box with 100mm steel shielding		
ILW arising from the EPR dismantling		
Specification Document for C1 and C4 Containers	RWMD EPR00007	
Applicables à la fabrication de conteneurs béton durables au conditionnement des déchets radioactifs TYPS C1PG, C1PG PA, C4PG'	D4507-UTO-CSCT-04/0527	

UK EPR Disposability Assessment - Revised Detailed Spent Fuel Data	RWMD EPR00008	
UK EPR Disposability Assessment – Impact and Fire Accident Performance of MOSAIK casks	RWMD EPR00009	
Design, construction and inspection of the liner for the RPE [NVDS] system in the EPR (sump liner, drains and floor drains)	ECEIG070065	C1
Generic Design Assessment: Summary of Disposability Assessment for Wastes and Spent Fuel Arising from the Operation of the EPR [Note: The public version of this summary report is a Level 2 reference in Sub-chapter 11.5]	NXA/10488242	1
Mapping Document for UK EPR Radioactive Waste Management Case	R10-110(A)	1
EPR, Waste Arising over the Operating Life	D450711001253	00
GDA – UK EPR Radioactive Waste Conditioning	ECUK110016	A